

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

SECTION

PAGE

REACTOR COOLANT SYSTEM (Continued)

Figure 3.4.1.1-1 Deleted	3/4 4-3	
Jet Pumps	3/4 4-4	
Recirculation Pumps	3/4 4-5	
Idle Recirculation Loop Startup	3/4 4-6	
3/4.4.2 SAFETY/RELIEF VALVES	3/4 4-7	
3/4.4.3 REACTOR COOLANT SYSTEM LEAKAGE		
Leakage Detection Systems	3/4 4-8	
Operational Leakage	3/4 4-9	
Table 3.4.3.2-1 Reactor Coolant System Pressure Isolation Valves	3/4 4-11	
3/4.4.4 (Deleted) The information from pages 3/4 4-12 through 3/4 4-14 has been intentionally omitted. Refer to note on page 3/4 4-12	3/4 4-12	
3/4.4.5 SPECIFIC ACTIVITY	3/4 4-15	
Table 4.4.5-1 Primary Coolant Specific Activity Sample and Analysis Program	3/4 4-17	
3/4.4.6 PRESSURE/TEMPERATURE LIMITS		
Reactor Coolant System	3/4 4-18	
Figure 3.4.6.1-1 Minimum Reactor Pressure Vessel Metal Temperature Vs. Reactor Vessel Pressure	3/4 4-20	
Table 4.4.6.1.3-1 Deleted	3/4 4-21	
Reactor Steam Dome	3/4 4-22	
3/4.4.7 MAIN STEAM LINE ISOLATION VALVES	3/4 4-23	
3/4.4.8 STRUCTURAL INTEGRITY	3/4 4-24	

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>REACTOR COOLANT SYSTEM (Continued)</u>	
Figure 3.4.1.1-1 Deleted	3/4 4-3
Jet Pumps.....	3/4 4-4
Recirculation Pumps.....	3/4 4-5
Idle Recirculation Loop Startup.....	3/4 4-6
3/4.4.2 SAFETY/RELIEF VALVES.....	3/4 4-7
3/4.4.3 REACTOR COOLANT SYSTEM LEAKAGE	
Leakage Detection Systems.....	3/4 4-8
Operational Leakage.....	3/4 4-9
Table 3.4.3.2-1 Reactor Coolant System Pressure Isolation Valves	3/4 4-11
3/4.4.4 (Deleted) The information from pages 3/4 4-12 through 3/4 4-14 has been intentionally omitted. Refer to note on page 3/4 4-12	3/4 4-12
3/4.4.5 SPECIFIC ACTIVITY.....	3/4 4-15
Table 4.4.5-1 Primary Coolant Specific Activity Sample and Analysis Program	3/4 4-17
3/4.4.6 PRESSURE/TEMPERATURE LIMITS	
Reactor Coolant System.....	3/4 4-18
Figure 3.4.6.1-1 Minimum Reactor Pressure Vessel Metal Temperature Vs. Reactor Vessel Pressure	3/4 4-20
Table 4.4.6.1.3-1 Deleted	3/4 4-21
Reactor Steam Dome.....	3/4 4-22
3/4.4.7 MAIN STEAM LINE ISOLATION VALVES.....	3/4 4-23
3/4.4.8 STRUCTURAL INTEGRITY.....	3/4 4-24

TABLE 3.3.6-2
CONTROL ROD BLOCK INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. <u>ROD BLOCK MONITOR</u>		
a. Upscale ^(a)		
1) Low Trip Setpoint (LTSP)	*	*
2) Intermediate Trip Setpoint (ITSP)	*	*
3) High Trip Setpoint (HTSP)	*	*
b. Inoperative	N/A	N/A
c. Downscale (DTSP)	*	*
d. Power Range Setpoint ^(b)		
1) Low Power Setpoint (LPSP)	28.1% RATED THERMAL POWER	28.4% RATED THERMAL POWER
2) Intermediate Power Setpoint (IPSP)	63.1% RATED THERMAL POWER	63.4% RATED THERMAL POWER
3) High Power Setpoint (HPSP)	83.1% RATED THERMAL POWER	83.4% RATED THERMAL POWER
2. <u>APRM</u>		
a. Simulated Thermal Power - Upscale:		
- Two Recirculation Loop Operation	$\leq 0.66 \text{ W} + 55.2\%$ and $\leq 108.0\%$ of RATED THERMAL POWER	$\leq 0.66 \text{ W} + 55.7\%$ and $\leq 108.4\%$ of RATED THERMAL POWER
- Single Recirculation Loop Operation****	$\leq 0.66 (\text{W}-7.6\%) + 55.2\%$ and $\leq 108.0\%$ of RATED THERMAL POWER	$\leq 0.66 (\text{W}-7.6\%) + 55.7\%$ and $\leq 108.4\%$ of RATED THERMAL POWER
b. Inoperative	N.A.	N.A.
c. Neutron Flux - Downscale	$\geq 3.2\%$ of RATED THERMAL POWER	$\geq 2.8\%$ of RATED THERMAL POWER
d. Simulated Thermal Power - Upscale (Setdown)	$\leq 12.0\%$ of RATED THERMAL POWER	$\leq 13.0\%$ of RATED THERMAL POWER
e. Recirculation Flow - Upscale	*	*
f. LPRM Low Count	< 20 per channel < 3 per axial level	< 20 per channel < 3 per axial level
3. <u>SOURCE RANGE MONITORS</u>		
a. Detector not full in	N.A.	N.A.
b. Upscale	$\leq 1 \times 10^5$ cps	$\leq 1.6 \times 10^5$ cps
c. Inoperative	N.A.	N.A.
d. Downscale	≥ 3 cps**	≥ 1.8 cps**