

NOTICE OF VIOLATION

Wisconsin Electric Power Company
Point Beach Nuclear Plant

Docket No. 50-266; 50-301
License No. DPR-24; DPR-27

During an NRC inspection conducted from July 27 through September 8, 1997, violations of NRC requirements were identified. In accordance with NUREG-1600, "General Statement of Policy and Procedure for NRC Enforcement Actions," the violations are listed below:

1. 10 CFR Part 50, Appendix B, Criterion B, "Corrective Action," requires, in part, that conditions adverse to quality are promptly corrected.

Contrary to the above, in the four examples given below conditions adverse to quality were not promptly corrected:

- a. In August 1997, the licensee identified that procedures for testing of valve 1SI-852A had not been revised by May 17, 1997, to correct a previous violation of NRC requirements.
- b. On July 8, 1997, the NRC inspectors identified that the Operations Notebook in the main control room had not been updated by June 30, 1997, to include the requirements for initiating temporary changes to operations procedures. This action was necessary to correct a previous violation of NRC requirements.
- c. On July 28, 1997, the licensee identified that reactor coolant pump maintenance procedures had not been revised by May 9, 1997, to correct a previous violation of NRC requirements.
- d. On July 29, 1997, the licensee identified that a Licensee Event Report had not been submitted for a condition identified on May 15, 1997, that alone could have prevented the fulfillment of the safety function of structures or systems that are needed to remove residual heat.

This is a Severity Level IV violation (Supplement I).

2. Technical Specification 15.3.1.A.3.b requires, in part, that both residual heat removal (RHR) loops be operable when the reactor coolant temperature is less than 140 degrees Fahrenheit (°F) except when the reactor vessel head is removed and the refueling cavity flooded or to meet surveillance requirements.

Contrary to the above, a Unit 2 RHR loop was inoperable on two occasions with the reactor coolant temperature below 140 °F and the reactor vessel head installed. Specifically:

- a. On July 29, 1997, the "B" RHR loop was made inoperable when a leak from a crack in a section of one-inch diameter, component coolant water pipe to the "B" RHR heat exchanger was isolated to facilitate repair.

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- b. On August 6, 1997, the "A" RHR loop became inoperable as a result of excessive mechanical seal leakage on the "A" RHR pump (2P-10A).

This is a Severity Level IV violation (Supplement I).

- 3. 10 CFR Part 50, Appendix B, Criterion XI, "Test Control," requires, in part, that a test program be established to assure that all testing required to demonstrate that structures, systems, and components will perform satisfactorily in-service is identified and performed in accordance with written test procedures which incorporate the requirements and acceptance limits contained in the applicable design documents.

- a. Contrary to the above, as of August 12, 1997, the NRC identified that the licensee failed to ensure that acceptance criteria for inservice testing of the Unit 2 charging pumps were incorporated into Inservice Test procedure, IT 22, Revision 5, "Charging Pumps and Valves (Quarterly) Unit 2."
- b. Contrary to the above, on September 3, 1997, the NRC identified that Routine Maintenance Procedure 9046-1, "Station Battery," Revision 22, a written procedure which included testing to demonstrate that safety-related components would perform satisfactorily in-service, did not incorporate acceptance limits from design documents for cell voltage or specific gravity.

This is a Severity Level IV violation (Supplement I).

- 4. Technical Specification 15.6.11 states, in part, that a high radiation area with radiation levels such that a major portion of the body could receive in 1 hour a dose greater than 1000 millirem shall be provided with locked doors to prevent unauthorized entry.

Contrary to the above, on August 13, 1997, the licensee identified that the lower equipment hatch to the Unit 2 containment, a high radiation area with localized radiation dose rates of up to approximately 1,100 millirem per hour at 30 centimeters from the radiation source, was not locked to prevent unauthorized entry.

This is a Severity Level IV violation (Supplement IV).

Pursuant to the provisions of 10 CFR 2.201, Wisconsin Electric is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555, with a copy to the Regional Administrator, Region III, and a copy to the NRC Resident Inspector, within 30 days of the date of the letter transmitting this Notice. This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or if contested, the basis for disputing the violation; (2) corrective action taken and the results achieved; (3) corrective action to be taken to avoid further violations; and (4) the date when full compliance will be achieved. Where good cause is shown, consideration may be given to extending your response time.

Because your response will be placed in the NRC Public Document Room (PDR), to the extent possible, it should not include any personal privacy, proprietary, or safeguards information so that it can be placed in the PDR without redaction. If personal privacy or proprietary information is necessary to provide an acceptable response, then please provide a bracketed copy of your response that identifies the information that should be protected and a redacted copy of your response that deletes such information. If you request withholding of such material, you must specifically identify the portions of your response that you seek to have withheld and provide in detail the bases for your claim of withholding (e.g., explain why the disclosure of information will create an unwarranted invasion of personal privacy or provide the information required by 10 CFR 2.79 (b) to support a request for withholding confidential commercial or financial information). If safeguards information is necessary to provide an acceptable response, please provide the level of protection described in 10 CFR 73.21.

Dated at Lisle, Illinois,
this 17th day of October 1997