

Exelon Generation Company, LLC
LaSalle County Station
2601 North 21st Road
Marseilles, IL 61341-9757

www.exeloncorp.com

RA05-74

September 9, 2005

10 CFR 50.73

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

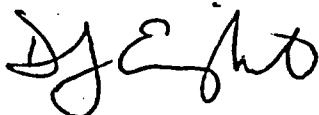
LaSalle County Station, Unit 1
Facility Operating License No. NPF 11
NRC Docket No. 50-373

Subject: Licensee Event Report

In accordance with 10 CFR 50.73 (a)(2)(i)(B), Exelon Generation Company, (EGC), LLC, is submitting Licensee Event Report Number 05-002-01, Docket No. 050-373. This supplement provides the results of the root cause investigation of the event.

Should you have any questions concerning this letter, please contact Mr. Terrence W. Simpkin, Regulatory Assurance Manager, at (815) 415-2800.

Respectfully,



Daniel Enright
Plant Manager
LaSalle County Station

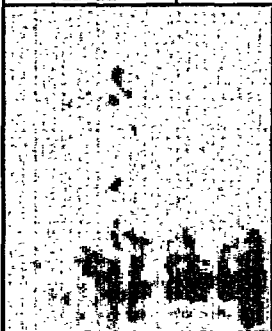

Attachment: Licensee Event Report

cc: Regional Administrator - NRC Region III
NRC Senior Resident Inspector - LaSalle County Station

IE02

LICENSEE EVENT REPORT (LER)(See reverse for required number of
digits/characters for each block)

Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME LaSalle County Station, Unit 1						2. DOCKET NUMBER 05000373			3. PAGE 1 of 3					
4. TITLE Reactor Core Isolation Cooling (RCIC) Barometric Condenser Vacuum Pump Discharge Check Valve 1E51-F028 Failed Local Leak Rate Test														
5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED					
MO	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MO	DAY	YEAR	FACILITY NAME		DOCKET NUMBER			
04	07	2005	2005	002	01	09	09	2005	FACILITY NAME		DOCKET NUMBER			
9. OPERATING MODE		1		11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)										
10. POWER LEVEL		100												
			<input type="checkbox"/>	20.2201(b)		<input type="checkbox"/>	20.2203(a)(3)(ii)		<input type="checkbox"/>	50.73(a)(2)(ii)(B)		<input type="checkbox"/>	50.73(a)(2)(ix)(A)	
			<input type="checkbox"/>	20.2201(d)		<input type="checkbox"/>	20.2203(a)(4)		<input type="checkbox"/>	50.73(a)(2)(iii)		<input type="checkbox"/>	50.73(a)(2)(x)	
			<input type="checkbox"/>	20.2203(a)(1)		<input type="checkbox"/>	50.36(c)(1)(i)(A)		<input type="checkbox"/>	50.73(a)(2)(iv)(A)		<input type="checkbox"/>	73.71(a)(4)	
			<input type="checkbox"/>	20.2203(a)(2)(i)		<input type="checkbox"/>	50.36(c)(1)(ii)(A)		<input type="checkbox"/>	50.73(a)(2)(v)(A)		<input type="checkbox"/>	73.71(a)(5)	
			<input type="checkbox"/>	20.2203(a)(2)(ii)		<input type="checkbox"/>	50.36(c)(2)		<input type="checkbox"/>	50.73(a)(2)(v)(B)		<input type="checkbox"/>	OTHER	
			<input type="checkbox"/>	20.2203(a)(2)(iii)		<input type="checkbox"/>	50.46(a)(3)(ii)		<input type="checkbox"/>	50.73(a)(2)(v)(C)		Specify in Abstract below or in NRC Form 366A		
			<input type="checkbox"/>	20.2203(a)(2)(iv)		<input type="checkbox"/>	50.73(a)(2)(i)(A)		<input type="checkbox"/>	50.73(a)(2)(v)(D)				
			<input type="checkbox"/>	20.2203(a)(2)(v)		<input checked="" type="checkbox"/>	50.73(a)(2)(i)(B)		<input type="checkbox"/>	50.73(a)(2)(vii)				
			<input type="checkbox"/>	20.2203(a)(2)(vi)		<input type="checkbox"/>	50.73(a)(2)(i)(C)		<input type="checkbox"/>	50.73(a)(2)(viii)(A)				
			<input type="checkbox"/>	20.2203(a)(3)(i)		<input type="checkbox"/>	50.73(a)(2)(ii)(A)		<input type="checkbox"/>	50.73(a)(2)(viii)(B)				
12. LICENSEE CONTACT FOR THIS LER														
NAME Bob Tjernlund, Design Engineering								TELEPHONE NUMBER (Include Area Code) (815) 415-2918						
13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT														
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX				
X	BN	ISV	Flowserve	Y										
14. SUPPLEMENTAL REPORT EXPECTED														
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)				<input checked="" type="checkbox"/> NO				15. EXPECTED SUBMISSION DATE		MONTH	DAY	YEAR		

16. ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines)

On 4/7/05, local leak rate testing (LLRT) was being performed on LaSalle County Station Unit 1 Reactor Core Isolation Cooling (RCIC) [BN] Barometric Condenser Vacuum Pump Discharge isolation valves 1E51-F069 and 1E51-F028. The procedurally required test pressure could not be achieved between these valves, and the measured leakage was therefore classified as indefinite.

Acceptable test pressure and measured leakage were achieved when the test operator mechanically agitated check valve 1E51-F028. The 1E51-F028 check valve was declared inoperable and the 1E51-F069 motor-operated globe valve was closed.

The 1E51-F028 valve was replaced and tested acceptably. A root cause investigation determined that the valve selected for this application was not optimal for this application because it was susceptible to internal corrosion and binding. Corrective actions include replacing the current piston lift check valve with a swing check valve, and modifying the pipe slope as needed to eliminate low points that accumulate water.

LICENSEE EVENT REPORT (LER)

1. FACILITY NAME	2. DOCKET NUMBER	6. LER NUMBER			3. PAGE
LaSalle County Station, Unit 1	05000373	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 of 3
		05	- 002	- 00	

17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor, 3489 Megawatts Thermal Rated Core Power

A. CONDITION PRIOR TO EVENT

Unit(s): 1 Event Date: 04/07/2005 Event Time: 1130 CDT
Reactor Mode(s): 1 Power Level(s): 100
Mode(s) Name: Run

B. DESCRIPTION OF EVENT

On 4/7/05, local leak rate testing (LLRT) was being performed on LaSalle County Station Unit 1 Reactor Core Isolation Cooling (RCIC) [BN] Barometric Condenser Vacuum Pump Discharge isolation valves 1E51-F069 and 1E51-F028. 1E51-F069 is a motor operated globe valve and the 1E51-F028 is a piston lift check valve. The LLRT was being performed as a follow up to previous failure of the 1E51-F028 valve in September 2004. The procedurally required test pressure could not be achieved between these valves, and the measured leakage was therefore classified as indefinite. Acceptable test pressure and measured leakage were achieved when the test operator mechanically agitated check valve 1E51-F028.

The 1E51-F028 valve was declared inoperable. 1E51-F069 was closed in accordance with Technical Specification (TS) 3.6.1.3 Required Action A.1 and A.2, and was controlled with an equipment status tag.

Indications were that 1E51-F028 was stuck open and had not been capable of performing its specified safety function for a period of time longer than allowed by TS. This event is therefore reportable under 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by the plant's Technical Specifications.

C. CAUSE OF EVENT

The direct cause of this event was that the 1E51-F028-valve disc (piston) was stuck in the open position. Following a similar failure in September 2004, the piston and spring were replaced, and the valve internals were cleaned. A root cause investigation determined that the valve selected for this application was not optimal for this application because it was susceptible to internal corrosion and binding. This design, however, was the best option available at the time the plant was constructed.

D. SAFETY ANALYSIS

The safety significance of this event was minimal. Isolation valve 1E51-F069 was fully operable as demonstrated with the local leak rate testing, so the primary containment integrity remained intact.

This was not a safety system functional failure.

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		05	- 002	- 00	

17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

E. CORRECTIVE ACTIONS

- Check valve 1E51-F028 was replaced on 5/6/05 and tested satisfactorily (Complete).
- The piston lift check valves currently installed for 1E51-F028 (Unit 1) and 2E51-F028 (Unit 2) are scheduled to be replaced with swing check valves (AT# 322203-28 and 322203-17).
- The slope of the Unit 1 and Unit 2 RCIC vacuum pump discharge piping will be modified to eliminate low spots that allow water to accumulate (AT# 322203-23 and 322203-24).

F. PREVIOUS OCCURRENCES

<u>LER Number</u>	<u>Title</u>
373-04-001	Invalid Containment Isolation Valve Local Leak Rate Test Due to Inadequate Procedure

The corrective actions from this LER were directed at correcting errors in LLRT methodology. 1E51-F028 failed its LLRT after the procedure was corrected, and the piston and spring were replaced, and the body internals were cleaned.

G. COMPONENT FAILURE DATA

Piston Lift Check Valve, Flowserve (Rockwell Edwards), Fig 838YT