

OCT 14 1988

Docket No. 70-36
License No. SNM-33
Amendment No. 9

Combustion Engineering, Inc.
ATTN: Dr. P. L. McGill, Vice President
Nuclear Fuel
1000 Prospect Hill Road
Windsor, CT 06095-0500

release

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Gentlemen:

In accordance with your amendment application dated October 11, 1988, and pursuant to Title 10, Code of Federal Regulations, Part 70, Materials License No. SNM-33 is hereby amended to permit storage of licensed material in Building 256. Accordingly, Condition 9 is revised to include the supplement dated October 11, 1988. This amendment is also subject to the following license condition:

35. Building 251 and Building 250 shall remain as authorized places of use.

All other conditions of the license shall remain the same. This condition was discussed and agreed to by your Mr. T. Cameron and David McCaughey of my staff on October 13, 1988.

Revised License No. SNM-33 incorporating Amendment No. 9 and our Safety Evaluation Report are enclosed.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:

Leland C. Rouse, Chief
Fuel Cycle Safety Branch
Division of Industrial and
Medical Nuclear Safety, NMSS

Enclosures:

1. Revised License No. SNM-33
2. Safety Evaluation Report

cc w/encs:

Mr. A. E. Scherer, Director
Nuclear Licensing

Mr. C. B. Brinkman, Manager
Washington Nuclear Operations

Mr. J. A. Rode, Plant Manager
Hematite Fuel Manufacturing

Mr. H. E. Eskridge, Supervisor
Licensing, Safety and Accountability

Distribution w/enc1

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NAME:DAMcCaughey:dm/mh:	VLTharpe:	GHBidinger:	LCRouse:
DATE:10/13/88	10/13/88 :	10/13/88:	: 10/14/88

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MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 40 and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee	
1. Combustion Engineering, Inc.	3. License number SNM-33 Amendment No. 9
2. P. O. Box 107 Hematite, Missouri 63047	4. Expiration date December 31, 1988
	5. Docket or Reference No. 70-36
6. Byproduct, source, and/or special nuclear material	7. Chemical and/or physical form
A. Uranium enriched to maximum 5.0 weight percent in the U-235 isotope	A. Any, excluding metal powder
B. Uranium, any U-235 enrichment	B. Any, excluding metal powder
C. Source material (Uranium and Thorium)	C. Any, excluding metal powder
D. Cobalt-60	D. Sealed sources
8. Maximum amount that licensee may possess at any one time under this license	
A. 8,000 kilograms contained U-235	
B. 350 grams	
C. 50,000 kilograms	
D. 40 millicuries, total	
9. Authorized Use: For use in accordance with the statements, representations, and conditions contained in Part I of the licensee's renewal application dated February 26, 1982, and supplements dated July 21, 1982; February 21, 1983; May 31, 1984; April 29, June 6, and October 11, 1988; and letters dated February 29, 1984, January 20, 1986, and March 30, 1987.	
10. Authorized Place of Use: The licensee's existing facilities in Hematite, Missouri, as described in the referenced license renewal application.	
11. Quarterly inspections by the Supervisor, NLS&A, or his representative shall be preplanned and shall be documented. Such documentation shall be maintained for 2 years.	

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12. A written report shall be made by the NLS&A Supervisor to the Plant Manager every 6 months reviewing employee radiation exposure (internal and external) and effluent release data to determine:
- if there are any upward trends developing in personnel exposure for identifiable categories of workers, types of operations, or in effluent releases;
 - if exposures and releases can be lowered in accordance with the ALARA commitment; and
 - if equipment for effluent and exposure control is being properly used, maintained, and inspected.
13. The licensee shall leak test sealed sources in accordance with the enclosed "License Condition For Leak Testing Sealed Byproduct Material Sources."
14. Release of equipment and material from the plant site or to clean areas onsite shall be in accordance with the enclosed "Guidelines for Decontamination of Facilities and Equipment Prior to Release for Unrestricted Use or Termination of Licenses for Byproduct, Source, or Special Nuclear Material," dated August 1987.
15. Pursuant to 10 CFR 20.302, the licensee is authorized to treat waste and scrap materials containing uranium enriched in U-235 and/or source material by incineration.
16. Within 60 days of the date of this license renewal, the licensee shall submit to the NRC a description of a proposed monitoring program to determine the quantity and environmental effects of radioactivity on spent limestone rock used as onsite fill material and to determine the environmental effects of outdoor storage of the alpha-contaminated material.
17. The licensee shall survey spent limestone rock discharge from each HF scrubber for beta contamination. Rock with beta contamination which exceeds five times the background of fresh rock shall not be used for landfill.
18. Within 60 days of the date of this license renewal, the licensee shall submit to NMSS a plan, including schedule, for the disposal of alpha-contaminated spent limestone rock.
19. The licensee shall decontaminate the two evaporation ponds such that the average residual contamination in each pond does not exceed the appropriate limit of either 250 picocuries of insoluble uranium or 100 picocuries of soluble uranium per dry gram of soil. The Tc-99 concentrations in a composite sample for each pond shall be determined.

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20. a. If the radioactivity in plant gaseous effluents exceeds 105 mCi per calendar quarter, the licensee shall, within 30 days, prepare and submit to the Commission a report which identifies the cause for exceeding the limit and the corrective actions to be taken by the licensee to reduce the release rates. If the parameters important to a dose assessment change, a report shall be submitted within 30 days which describes the changes in parameters and includes an estimate of the resultant change in dose commitment.
- b. In the event that the calculated dose to any member of the public in any consecutive 12-month period is about to exceed the limits specified in 40 CFR 190.10, the licensee shall take immediate steps to reduce emissions so as to comply with 40 CFR 190.10. As provided in 40 CFR 190.11, the licensee may petition the Nuclear Regulatory Commission for a variance from the requirements of 40 CFR 190.10. If a petition for a variance is anticipated the licensee shall submit the request at least 90 days prior to exceeding the limits specified in 40 CFR 190.10.
21. The licensee shall maintain and execute the response measures of his Radiological Contingency Plan submitted to the Commission by letter dated December 28, 1987. The licensee shall also maintain implementing procedures for his Radiological Contingency Plan as necessary to implement the Plan. The licensee shall make no change in his Radiological Contingency Plan that would decrease the response effectiveness of the Plan without prior Commission approval as evidenced by a license amendment. The licensee may make changes to his Radiological Contingency Plan without prior Commission approval if the changes do not decrease the response effectiveness of the Plan. The licensee shall furnish the Chief, Fuel Cycle Safety Branch, Division of Industrial and Medical Nuclear Safety, NMSS, U. S. Nuclear Regulatory Commission, Washington, DC 20555, a report containing a description of each change within 6 months after the change is made.
22. At the end of the plant life, the licensee shall decontaminate the facilities and site in accordance with the general decommissioning plan submitted in the enclosure to the letter dated January 12, 1979, so that these facilities and grounds can be released to unrestricted use. The financial commitment to assure that funds will be available for decommissioning in the letter dated March 8, 1979, is hereby incorporated as a condition of the license.
23. The licensee shall continue the soil sampling program for the spent limestone fill areas, as described in the letter dated February 29, 1984, until discontinuance is authorized by the Commission.
1. The report or petition should be submitted to the Director, Office of Nuclear Material Safety and Safeguards, with a copy to the Regional Administrator, Region III.

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24. The monitoring program for the spent limestone shall include:

- a. Continuous air sampling at the center of, and approximately 1 meter above, the uncovered spent limestone piles for a minimum 2-year period. The weekly samples may be composited and analyzed for uranium activity on a quarterly basis. The lower limit of detection shall be 10^{-10} mCi/ml, or
- b. Measurement of the uranium activity on the surface of the spent limestone. Prior to conducting such a program, the licensee shall submit the sampling and analytical program to the NRC for approval.

25. Processing of UF_6 in 10-ton cylinders is not authorized.

26. The 10-ton UF_6 cylinders shall be equipped with valve protectors.

27. The concrete pad for storage of UF_6 cylinders and the surrounding area shall be sloped or graded so that any spilled combustible fluids would not be confined to the storage area.

28. No combustibles shall be stored on the concrete pad.

29. A CO_2 fire extinguisher shall be readily available near the storage pad.

30. In addition to the controls in Section 4.2.3 of the enclosure to the letter dated March 30, 1987, UF_6 cylinders which are in transport and containing UF_6 heels shall be either sealed, in sealed overpacks, or in sealed vehicles.

31. Notwithstanding the statement in Section 4.2.3 of the application, the k-effective of a unit or an array of units shall not exceed 0.95 unless specifically authorized by the license.

32. Nuclear criticality safety evaluations performed by the licensee in accordance with Section 2.7, Part I of the application, shall be based on assumptions of optimum moderation and reflection of individual safe units and of arrays.

33. Nuclear criticality safety evaluations involving k-effective calculations performed by a Nuclear Criticality Specialist shall be independently reviewed and approved by an individual having, as a minimum, the qualifications of a Nuclear Criticality Specialist.

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34. For uranium enriched to more than 4.1 w/o U-235, the licensee shall limit the agglomeration/granulation process, each agglomerated powder storage location, and the pellet pressing operating to safe mass units as specified in Table 4.2.4, Part I of the application.
35. Building 251 and Building 250 shall remain as authorized places of use.



FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:

Date:

By:

Leland C. Rouse

Division of Industrial and Medical
Nuclear Safety, NMSS
Washington, DC 20555

DAW 10/13/88
VLT 10/13/88
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LRC 10/14/88

DOCKET NO: 70-36

LICENSEE: Combustion Engineering, Inc. (CE)
Hematite, Missouri

SUBJECT: SAFETY EVALUATION REPORT, AMENDMENT APPLICATION DATED
OCTOBER 11, 1988, RE STORAGE IN FACILITY ADDITION

Background

On October 11, 1988, CE submitted an application for license amendment requesting authorization to store licensed material in a facility addition (Building 256). By License Condition 10, CE is limited to authorized activities in existing facilities.

Discussion

Building 256, a newly constructed warehouse facility, will house shipping and receiving operations and store licensed material. This building, which has about 3,500 ft² of floor space is attached to the south side of Building 255.

The activities that will be authorized have been conducted in existing facilities. Therefore, radiation safety and nuclear criticality safety controls from the license will be extended to Building 256 and are adequate. CE has revised Part I of the application to include this facility addition.

CE intends to remove Building 251 and part of Building 250 to make room for new facilities. CE proposed that the old warehouse facility (Building 251) and a portion of Building 250 be removed from the license as authorized places of use. Until the staff confirms that the two buildings have been decontaminated, the buildings must remain as authorized places of use. Accordingly, the staff recommends the following license condition:

Building 251 and Building 250 shall remain as authorized places of use.

Conclusion/Recommendation

The staff concludes that the proposed storage in Building 256 should have no effect on the environment beyond that identified in the Environmental Assessment and the Negative Declaration published in the Federal Register on November 30, 1982.

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Combustion Engineering, SER

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The staff has no objection to the operation of Building 256. Subject to the condition identified above, approval of the amendment application is recommended.

The Region III Branch Chief has no objection to this proposed action.

Original Signed By:

David A. McCaughey
Uranium Fuel Section
Fuel Cycle Safety Branch
Division of Industrial and
Medical Nuclear Safety, NMSS

Approved by: **Original Signed By:**
George H. Bidinger, Section Leader

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NAME: <i>DAM</i> McCaughey: dam/mh:	VL <i>JK</i> Harpe:	GHBidinger:
DATE: 10/13/88	10/13/88	10/13/88 :

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DOCKET NO: 70-36
LICENSEE: Combustion Engineering (CE)
Hematite, Missouri
SUBJECT: CATEGORICAL EXCLUSION FOR AMENDMENT REQUEST
DATED OCTOBER 11, 1988

By letter dated October 11, 1988, CE requested an amendment to License No. SNM-33 to relocate the shipping/receiving facility. CE has constructed a new warehouse facility to house shipping/receiving operations. This new facility will also be used for storage. These activities are currently authorized for other buildings and are being relocated to the new facility. No new activities are requested. The use of the new facility will not adversely effect the public health and safety or the environment. Accordingly, pursuant to 10 CFR 51.22 (c) (11), neither an Environmental Impact Statement nor an Environmental Assessment is warranted for the proposed action.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:

Leland C. Rouse, Chief
Fuel Cycle Safety Branch
Division of Industrial and
Medical Nuclear Safety, NMSS

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NAME:MHorn:vlt/mh:	DAMcCaughey:	VLTharpe:	GHBidinger:	LCRouse:
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