



U.S. NUCLEAR REGULATORY COMMISSION
STANDARD REVIEW PLAN
OFFICE OF NUCLEAR REACTOR REGULATION

18.2 Safety Parameter Display System (SPDS)

REVIEW RESPONSIBILITIES

- Primary - Human Factors Engineering Branch (HFEB)
Secondary - Procedures and Systems Review Branch (PSRB)
- Instrumentation and Control Systems Branch (ICSB)

I. AREAS OF REVIEW

The principal purpose and function of the Safety Parameter Display System (SPDS) is to aid control room personnel during abnormal and emergency conditions in determining the safety status of the plant and in assessing whether abnormal conditions warrant corrective action by operators to avoid a degraded core. During emergencies the SPDS serves as an aid to evaluating the current safety status of the plant, executing function-oriented emergency procedures, and monitoring the impact of engineered safeguards or mitigation activities. The SPDS also operates during normal operations, continuously displaying information from which the plant safety status can be readily and reliably assessed.

The scope of the staff's review is limited to the principal function of the SPDS. The review is bounded by the minimum set of plant variables, and whatever hardware, software processing algorithms and training are needed to achieve the principal SPDS functions. Secondary functions, such as presentation of data to assist operators with diagnosis of abnormal conditions, are not part of the scope of review under this SRP Section.

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USNRC STANDARD REVIEW PLAN

Standard review plans are prepared for the guidance of the Office of Nuclear Reactor Regulation staff responsible for the review of applications to construct and operate nuclear power plants. These documents are made available to the public as part of the Commission's policy to inform the nuclear industry and the general public of regulatory procedures and policies. Standard review plans are not substitutes for regulatory guides or the Commission's regulations and compliance with them is not required. The standard review plan sections are keyed to the Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants. Not all sections of the Standard Format have a corresponding review plan.

Published standard review plans will be revised periodically, as appropriate, to accommodate comments and to reflect new information and experience.

Comments and suggestions for improvement will be considered and should be sent to the U.S. Nuclear Regulatory Commission, Office of Nuclear Reactor Regulation, Washington, D.C. 20555.

Review of secondary SPDS functions will be performed by applicants and licensees in the course of the detailed Control Room Design Review (see SRP Section 18.1).

The HFEB has lead responsibility for coordinating the review of the SPDS and specific responsibility for reviewing the incorporation of good human engineering principles in the location and accessibility of the SPDS, formatting of displays, and operator interactive devices.

HFEB, PSRB and ICSB share responsibility for the review of the applicant's verification and validation (V&V) program, including the applicant's program for SPDS design, development and installation testing. HFEB reviews the V&V program for its human performance aspects including the adequacy of applications software characteristics, system response times, verification of the design of display formats, and the ease of understanding and acting on displayed data. ICSB reviews the V&V program for hardware and operating software aspects such as system reliability and sensor accuracy. PSRB reviews the V&V program with regard to the selection and validation of the SPDS parameter set.

ICSB, in addition to reviewing the V&V program, reviews the final design for reliability and availability of SPDS hardware, and the means used to isolate SPDS signals from safety systems and to avoid propagation of electrical faults.

PSRB reviews the adequacy and basis of the parameters selected for display by the applicant to represent the critical plant functions identified in Supplement 1 to NUREG-0737, and the SPDS's relationship to and consistency with emergency operating procedures.

II. ACCEPTANCE CRITERIA

The HFEB acceptance criteria are applied in the review of all ORs, OLs, and CPs in accordance with the following:

- A. The acceptance criteria for licensees and applicants for operating licenses are based on meeting the relevant requirements of Task Action Plan Item I.D.2 of NUREG-0660 as clarified in Supplement 1 to NUREG-0737. The purpose of the review is to determine that the SPDS meets the following requirements of Supplement 1 to NUREG-0737:
 - 1. An SPDS shall be provided that is located convenient to control room operators.
 - 2. The SPDS shall continuously display information from which the safety status of the plant can be readily and reliably assessed by control room personnel responsible for the avoidance of degraded and damaged core events.

3. The SPDS shall provide a concise display of critical plant variables which at a minimum shall be sufficient to provide information to plant operators about the following critical safety functions:
 - a. Reactivity control
 - b. Reactor core cooling and heat removal from the primary system
 - c. Reactor coolant system integrity
 - d. Radioactivity control
 - e. Containment conditions

The specific parameters to be displayed shall be determined by the applicant.

4. The SPDS shall be designed to incorporate accepted human factors principles so that the displayed information can be readily perceived and comprehended by SPDS users.

Information, recommendations and guidance that provide a basis acceptable to the staff for implementing the requirements identified above are contained in Appendix A to this SRP section. Examples of acceptable approaches to meeting the SPDS requirements are contained therein. NUREG-0700 contains guidance that will be useful to reviewers on the human engineering aspects of displays, printers, systems analysis and performance validation.

- B. The acceptance criteria for construction permit applicants are based on meeting the relevant requirements of General Design Criterion 19, as it relates to the control room being designed with appropriate human factors engineering design principles to assure that the operator-machine interfaces of the control room are adequate to support safe operations of the plant. Review of applicants' incorporation of the SPDS function will be included in the review performed under Section 18.1 of the Standard Review Plan.

III. REVIEW PROCEDURES

The staff evaluation of the SPDS consists of reviews of the applicant/licensee's documentation (i.e. safety analysis report and implementation plan) and audit meetings/site visits. The procedures below are used to verify that the SPDS meets the acceptance criteria of Subsection II for three categories of applicants/licensees: A) holders of operating licenses that request a pre-implementation review and applicants for operating licenses, B) holders of operating licenses that do not request a pre-implementation review, C) applicants for construction permits that are just starting the control room design process.

- A. Holders of Operating Licenses Requesting Pre-implementation Review and Applicants for Operating Licenses.
 1. The HFEB reviewer assures that ICSB and PSRB are provided with copies of the implementation plan and safety analysis report and establishes a schedule with those branches for providing their input to the review.

2. The HFEB reviewer, in coordination with the ICSB and PSRB reviewers, evaluates the applicant/licensee's plan for verification and validation of the SPDS design (submitted as part of the SPDS implementation plan) to confirm that it is sufficient to provide reasonable assurance that the SPDS will meet the requirement that it provide a continuous display of valid and reliable information from which the plant safety status can be readily assessed. The reviewer verifies that the V&V program plan includes elements consistent with those described in Subsection 7 of Appendix A to this SRP section. Conformance with the guidance for a V&V program documented in NSAC-39 is acceptable, as are other V&V programs which the applicant/licensee demonstrates will accomplish the same goals.
3. The HFEB reviewer obtains written input from the PSRB reviewer on the safety analysis report. This input should include PSRB's evaluation of the applicant/licensee's basis on which the parameters selected for display are sufficient to assess the status of the critical safety functions identified in Supplement 1 to NUREG-0737 and are consistent with emergency operating procedures.
4. The HFEB reviewer obtains written input from the ICSB reviewer on ICSB's evaluation of the reliability and availability of the SPDS hardware and operating system software, and the means used to isolate SPDS signals from safety systems and avoid propagation of electrical faults (See Subsection 4.7 of Appendix B to SRP Section 7.1).
5. The HFEB reviewer evaluates the applicant/licensee's safety analysis report and available design documentation to confirm that means are provided to ensure that the data displayed are valid and that the display formats and operational interfaces of the SPDS have been designed to incorporate acceptable human engineering principles. Guidance for this evaluation is found in Subsections 5 and 6 of Appendix A to this SRP section. Additional human engineering guidelines are contained in Section 6 of NUREG-0700, especially Section 6.5, 6.6, 6.7, and 6.8.
6. Three separate audit meetings/site visits, as described below, may be arranged through the Division of Licensing Project Manager. As dictated by the comprehensiveness of the applicant/licensee's documentation and the schedule for design and implementation of the SPDS, the objectives of these audits may be met in fewer site visits.

Design Verification Audit. The purpose of this audit meeting is to obtain additional information required to resolve any outstanding questions about the V&V program, to confirm that the V&V program is being correctly implemented, and to audit the results of the V&V activities to date. At this meeting, the applicant should provide a thorough description of the SPDS design process. Emphasis should be placed on how the applicant is assuring that the implemented SPDS will: provide appropriate parameters, be isolated from safety systems, provide reliable and valid data, and incorporate good human engineering practice. To the extent dictated by the completeness of the V&V program plan, the HFEB reviewer will arrange for participation of PSRB and ICSB reviewers at this meeting.

Design Validation Audit: After review of all documentation, an audit may be conducted to review the as-built prototype or installed SPDS. The purpose of this audit is to assure that the results of the applicant/licensee's testing demonstrate that the SPDS meets the functional requirements of the design and to assure that the SPDS exhibits good human engineering practice.

Installation Audit. As necessary, a final audit may be conducted at the site to ascertain that the SPDS has been installed in accordance with the applicant/licensee's plan and is functioning properly. A specific concern is that the data displayed reflect the sensor signal which measures the variable displayed. This audit will be coordinated with and may be conducted by the NRC Resident Inspector.

- B. Holders of Operating Licenses That Do Not Request Pre-implementation Review
 - 1. The HFEB reviewer assures that ICSB and PSRB are provided with copies of the licensee's SPDS implementation plan and safety analysis report and establishes a schedule with those branches for providing their input to the review.
 - 2. The licensee's implementation plan and safety analysis report are reviewed to determine if a serious safety question is posed by the proposed SPDS or if the analysis is seriously inadequate. To accomplish this, the review is directed at (a) confirming the adequacy of the variables selected for display to assess critical safety functions, (b) confirming that the SPDS will be suitably isolated from electrical and electronic interference with equipment and sensors that are used in safety systems, (c) confirming that means are provided to ensure that the data displayed are valid, and (d) confirming that the licensee has committed to a human factors engineering program to ensure that the displayed information can be readily perceived and comprehended so as not to mislead the operator. The HFEB reviewer obtains SER input from the PSRB reviewer on item (a) and from the ICSB reviewer on item (b). The HFEB reviewer is responsible for review of items (c) and (d). Guidelines that will be used to identify serious safety questions or inadequate analyses are specified in Subsection 5 of Appendix A to this SRP section.
 - 3. A post-implementation audit may be conducted at the site to ascertain that the SPDS fulfills the requirements of Supplement 1 to NUREG-0737, has been installed in accordance with the licensee's plan and is functioning properly. This audit will be coordinated with and may be conducted by the NRC Resident Inspector.

C. Applicants for Construction Permits

Applicants which have not developed a control room design or are in the very early stages of control room design should incorporate the principal function of the SPDS in that design. In the course of the applicant's conduct and documentation of analyses to identify human/machine interface requirements and operator information needs, means for providing aid in determining the safety status of the plant and in assessing whether abnormal conditions warrant corrective action by operators should be established. Staff review of applicants' incorporation of this SPDS function will be included in the review performed under Section 18.1 of the Standard Review Plan.

IV. EVALUATION FINDINGS

The reviewers of an SPDS for holders of operating licenses that request pre-implementation review and reviewers of an SPDS for applicants for operating licenses confirm that sufficient information has been provided in the applicant/licensee's implementation plan, safety analysis report, and SPDS audit meetings, and that the review thereof supports conclusions of the following type to be used in the staff's safety evaluation report:

The safety parameter display system is acceptable and meets the applicable requirements of Supplement 1 to NUREG-0737. This conclusion is based on the following:

1. The variables displayed on the SPDS are sufficient to provide the minimum information required to assess the critical safety functions.
2. The SPDS is suitably isolated from electrical and electronic interference with equipment and sensors that are used in safety systems.
3. Means are provided to ensure that the data displayed are valid.
4. The applicant/licensee has demonstrated that the characteristics of the SPDS displays and other operational interfaces are sufficient to allow reasonable assurance that the information provided will be readily perceived and comprehended by its users.

The reviewers of an SPDS for holders of operating licenses that do not request a pre-implementation review confirm that sufficient information has been provided in the licensee's implementation plan and safety analysis report, and that the review thereof supports conclusions of the following type to be used in the staff's safety evaluation report:

No serious safety questions are posed by the proposed SPDS and implementation may continue. This conclusion is based on the following:

1. The variables selected for display are generally adequate to assess critical safety functions.
2. If implemented as designed, the SPDS will be suitably isolated from plant safety systems.
3. The licensee's design provides means to assure that displayed data are valid.
4. The licensee has committed to conduct a human factors engineering program which will allow reasonable assurance that the information provided will be readily perceived and comprehended by its users.

The conclusion that SPDS implementation may continue does not imply staff confirmation that the SPDS meets the requirements of Supplement 1 to NUREG-0737. Such confirmation can be made after a post-implementation audit or when the staff has otherwise obtained sufficient information.

For construction permit applicants', the SPDS function is reviewed in conjunction with the review performed under Section 18.1 of the Standard Review Plan. That review supports conclusions of the type specified in Section 18.1.

V. IMPLEMENTATION

The following is intended to provide guidance to all applicants and licensees regarding the staff's plans for using this SRP Section.

Except in those cases in which the applicant/licensee proposes an acceptable alternative method for complying with specified portions of the Commission's requirements, the method described herein will be used by the staff in its evaluation of conformance with Commission requirements.

Implementation schedules for conformance to parts of the methods discussed herein are contained in the referenced NUREGs and will be applied in the review of all ORs, OLs, and CPs in accordance with the following:

1. Acceptance criteria for Operating Reactors (ORs) and Operating Licenses (OLs) are implemented in accordance with Subsection II A, of this SRP section.
2. Acceptance criteria contained in Subsection II B of this SRP section are applied to all future CP application reviews.

VI. REFERENCES

1. General Design Criteria 19, "Control Room."
2. NUREG-0660, "NRC Action Plan Developed as a Result of the TMI-2 Accident," August 1980.
3. NUREG-0737, "Clarification of TMI Action Plan Requirements," November 1980
4. Supplement 1 to NUREG-0737, "Requirements for Emergency Response Capability (Generic Letter 82-33)," December 1982.
5. NUREG-0700, "Guidelines for Control Room Design Reviews," September 1981.
6. NSAC-39, "Verification and Validation for Safety Parameter Display Systems," December 1981.
7. SRP Section 18.1.
8. Appendix A to SRP Section 18.2.



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APPENDIX A to
SRP Section 18.2

HUMAN FACTORS REVIEW GUIDELINES FOR
THE SAFETY PARAMETER DISPLAY SYSTEM
(SPDS)

REVIEW RESPONSIBILITIES

Primary - Human Factors Engineering Branch (HFEB)

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REVIEWING THE SAFETY PARAMETER DISPLAY SYSTEM:

HUMAN FACTORS ENGINEERING GUIDELINES

1 INTRODUCTION

The accident at Three Mile Island Unit 2 (TMI-2) and subsequent investigations have demonstrated the need for improving how information is presented to people who operate reactors. This need becomes especially evident when a condition that could have safety significance occurs at a nuclear power plant. During such a condition, control room operators must monitor and process large amounts of data to make sure of the operating status and safety status of the plant and to intervene when intervention is needed to maintain the plant in a safe condition.

Supplement 1 to NUREG-0737 (Ref. 1) states some basic principles for designing a system that displays a minimum set of plant variables critical to safety (safety parameter display system (SPDS)), and describes how to coordinate and integrate the design of the SPDS with other emergency response facility initiatives as follows:

The design of the Safety Parameter Display System (SPDS), design of instrument displays based on Regulatory Guide 1.97 guidance, control room design review, development of function oriented emergency operating procedures, and operating staff training should be integrated with respect to the overall enhancement of operator ability to comprehend plant conditions and cope with emergencies. Assessment of information needs and display formats and locations should be performed by individual licensees. The SPDS could affect other control room improvements that licensees may consider. In some cases, a good SPDS may obviate the need for large-scale control room modifications. Installation of the SPDS should not be delayed by slower progress on other initiatives, and should not be contingent on completion of the control room design review. Nor should other initiatives, such as upgraded emergency operating procedures, be impacted by delays in SPDS procurement. While the NRC does not plan to impose additional requirements on licensees regarding SPDS, the NRC will work with the industry to assure the development of appropriate industry standards for SPDS systems.

The basic principles for the SPDS are stated in Supplement 1 of NUREG-0737 as follows:

- a. The SPDS should provide a concise display of critical plant variables to the control room operators to aid them in rapidly and reliably determining the safety status of the plant. Although the SPDS will be operated during normal operations as well as during abnormal conditions, the principal purpose and function of the SPDS is to aid the control

room personnel during abnormal and emergency conditions in determining the safety status of the plant and in assessing whether abnormal conditions warrant corrective action by operators to avoid a degraded core. This can be particularly important during anticipated transients and the initial phase of an accident.

- b. Each operating reactor shall be provided with a Safety Parameter Display System that is located convenient to the control room operators. This system will continuously display information from which the plant safety status can be readily and reliably assessed by control room personnel who are responsible for the avoidance of degraded and damaged core events.
- c. The control room instrumentation required (see General Design Criteria 13 and 19 of Appendix A to 10 CFR 50) provides the operators with the information necessary for safe reactor operation under normal, transient, and accident conditions. The SPDS is used in addition to the basic components and serves to aid and augment these components. Thus, requirements applicable to control room instrumentation are not needed for this augmentation (e.g., GDC 2, 3, 4 in Appendix A; 10 CFR Part 100; single-failure requirements). The SPDS need not meet requirements of the single-failure criteria and it need not be qualified to meet Class 1E requirements. The SPDS shall be suitably isolated from electrical or electronic interference with equipment and sensors that are in use for safety systems. The SPDS need not be seismically qualified, and additional seismically qualified indication is not required for the sole purpose of being a backup for SPDS. Procedures which describe the timely and correct safety status assessment when the SPDS is and is not available, will be developed by the licensee in parallel with the SPDS. Furthermore, operators should be trained to respond to accident conditions both with and without the SPDS available.
- d. There is a wide range of useful information that can be provided by various systems. This information is reflected in such staff documents as NUREG-0696, NUREG-0835, and Regulatory Guide 1.97. Prompt implementation of an SPDS can provide an important contribution to plant safety. The selection of specific information that should be provided for a particular plant shall be based on engineering judgment of individual plant licensees, taking into account the importance of prompt implementation.
- e. The SPDS display shall be designed to incorporate accepted human factors principles so that the displayed information can be readily perceived and comprehended by SPDS users.
- f. The minimum information to be provided shall be sufficient to provide information to plant operators about [the following critical safety functions]:
 - (i) Reactivity control
 - (ii) Reactor core cooling and heat removal from the primary system

- (iii) Reactor coolant system integrity
- (iv) Radioactivity control
- (v) Containment conditions.

The specific parameters to be displayed shall be determined by the licensee.

The documentation needed from applicants/licensees and for planned NRC review is stated in Supplement 1 to NUREG-0737 as follows:

- a. The licensee shall prepare a written safety analysis describing the basis on which the selected parameters are sufficient to assess the safety status of each identified function for a wide range of events, which include symptoms of severe accidents. Such analysis, along with the specific implementation plan for SPDS shall be reviewed as described below.
- b. The licensee's proposed implementation of an SPDS system shall be reviewed in accordance with the licensee's technical specifications to determine whether the changes involve an unreviewed safety question or change of technical specifications. If they do, they shall be processed in the normal fashion with prior NRC review. If the changes do not involve an unreviewed safety question or a change in the technical specifications, the licensee may implement such changes without prior approval by NRC or may request a pre-implementation review and approval. If the changes are to be implemented without prior NRC approval, the licensee's analysis shall be submitted to NRC promptly on completion of review by the licensee's offsite safety review committee. Based on the results of NRC review, the Director of IE or the Director of NRR may request or direct the licensee to cease implementation if a serious safety question is posed by the licensee's proposed system, or if the licensee's analysis is seriously inadequate.

This appendix proposes no new requirements; it presents guidelines to NRC staff and applicants/licensees on applying good principles of human factors engineering to the SPDS function and display. It is intended to provide guidance general enough that a reviewer may use it to evaluate the human factors engineering aspects of different SPDS installations.

2 ROLE OF SAFETY PARAMETER DISPLAY SYSTEMS

The SPDS helps the control room operating crew make quick and accurate assessments of the plant's safety status during abnormal and emergency conditions. By this assessment the reactor operator in the control room (control room operator) decides whether abnormal conditions demand corrective action to avoid a degraded reactor core. During normal operations, control room operators monitor the display in the course of performing their assigned monitoring

tasks. This serves to integrate the use of the display into normal operations. During emergencies, the SPDS should serve as an aid to the control room operating crew in evaluating the plant's current safety status and in executing function-oriented emergency procedures.

The SPDS is intended to provide information about the plant from a display system during normal operations as well as when conditions arise that could have safety significance. The system should continuously display information from which the control room operator responsible for avoiding degraded and damaged core events can readily and reliably assess the safety status of the plant. The SPDS is analogous to the way the basic attitude and flight performance instruments in an airplane provide status information to the pilot. The control room operating crew should be able to use the SPDS to detect conditions that could have safety significance and should also be able to use the information provided by the SPDS as an aid in taking corrective action to maintain or re-establish safe plant conditions.

Thus, the SPDS is a control room improvement to enhance the control room operator's ability to:

- ° comprehend plant status during stressful conditions,
- ° determine rapidly and reliably the safety status of the plant, and
- ° intervene in situations that demand human intervention.

The SPDS should provide control room operators with a readable, comprehensible and accurate display of critical plant variables, derived variables, or safety functions.

To use the system effectively, the control room operator must be trained in the use of the SPDS. Using the SPDS the control room operator should be able to interpret plant operating information, synthesize plant processes, and assess plant functions from the data provided on the display. The displayed data are read and processed by the control room operator to determine plant status. The design of the SPDS display should consider the control room operator's needs and should serve as a decision making aid to the control room operator.

3 SCOPE OF REVIEW GUIDELINES

The SPDS is a control room display device that is most effective when it has been designed to incorporate principles of good human factors engineering. This appendix presents only those SPDS review guidelines that are related to human factors engineering.

The scope of the staff's review will be limited to evaluating if the SPDS helps control room operators do their jobs well during conditions that have safety significance. The review will be bounded by the minimum set of critical plant variables, equipment display units, and data processing algorithms needed to achieve this goal. In this appendix, the staff will not provide review guidance

for such functions as the performance monitoring of plant systems or safety systems and the presentation of data to assist the control room operator with detailed diagnosis of abnormal operating conditions. The applicant/licensee should review these functions when the Detailed Control Room Design Review (DCRDR) is performed.

The review guidelines in this appendix can be applied generally to all types of SPDS displays; however, the trend in the nuclear industry is toward computer-driven cathode-ray tube (CRT) displays. Because most of the proposed SPDS designs in the technical briefings presented to the NRC staff have CRT displays, this document emphasizes review of CRT displays. Functional criteria for the SPDS do not rule out the use of other types of displays in SPDS designs. Review guidelines for specific SPDS designs that do not use CRT displays will be developed case by case, as they are needed.

NUREG-0700, "Guidelines for Control Room Design Reviews" (Ref. 3) provides general guidelines applicable to review of visual displays, process computers, and CRT displays from a human factors engineering standpoint. The SPDS as a display device has specialized functional requirements. In reviewing an SPDS, the specific device-oriented guidelines in this document should be used to complement the general guidelines in NUREG-0700. These specific guidelines are offered to help the reviewer evaluate the functional effectiveness of the SPDS. Information in NUREG-0700 is referenced where it is applicable.

Subsection 4, Use of SPDS Review Guidance, introduces major Subsections of this appendix. Subsection 4 also defines and comments on references which contain human factors engineering guidelines appropriate for use in NRC reviews of the SPDS.

In Subsection 5, Human Factors Guidelines for Reviewing an SPDS, the guidelines are tabulated. Generally, examples are provided for each guideline to illustrate acceptable human factors engineering practices. These guidelines and examples are provided to help a reviewer evaluate whether a given SPDS is designed sufficiently well from a human standpoint to serve its intended purpose and function. For SPDS designs not covered by these examples, the NRC reviewer should use the principles embodied in the guidelines to help evaluate the design.

In Subsection 6, Review Guidelines for SPDS Displays, further clarification of NRC guidance is provided for computer-driven CRT displays. Because computer-driven CRT displays offer considerable flexibility in how the data are presented, proposed systems will have a wide variety of display formats. The information in Subsection 6 will help a reviewer evaluate different displays objectively. Subsection 6 emphasizes important human factors engineering aspects about the use of CRT displays for the SPDS.

In Subsection 7, Verification and Validation of SPDS, the principles of a verification and validation program to ensure a high-quality SPDS are presented. In Subsection 8, NRC Staff Human Factors Engineering Review of

SPDS, the staff's review of the SPDS is discussed. In Subsection 9, Glossary of Terms, terms used in this appendix are defined, and in Subsection 10, References, documents used in preparing this appendix are listed.

The use of non-CRT types of displays is not precluded. These review guidelines should not eliminate consideration of other useful displays that are presently available or that may be developed as techniques for data presentation evolve.

4 USE OF SPDS REVIEW GUIDANCE

The human factors engineering guidelines provided in Subsections 5 and 6 of this appendix should serve both NRC staff who review systems as well as applicants/licensees who design those systems. The guidelines provided in this appendix should be used together with guidelines provided in NUREG-0700. NUREG-0700 provides human factors engineering information that may be used as guidelines for conducting a detailed control room design review (DCRDR). The DCRDR examines existing control rooms with the objective of improving the human factors of man-machine interfaces. SRP Section 18.1 (Ref. 4) provides information that will help a reviewer evaluate a DCRDR.

A reviewer should be familiar with the general human factors engineering guidelines in NUREG-0700, especially the following sections:

- ° Section 6.5, Visual Displays, which includes principles of display, meters, light indicators, and graphic recorders;
- ° Section 6.6, Labels and Location Aids, which includes labeling principles, label location, label content, and location aids;
- ° Section 6.7, Process Computers, which includes computer access, CRT displays, and printers; and
- ° Section 6.8, Panel Layout, which includes panel contents, recognition and identification enhancement, and layout arrangement factors.

A reviewer evaluating SPDS systems that use CRT displays should be familiar with CRT technology and its application to nuclear power systems. References 5, 6, and 7 provide general information on the design of CRT-based display systems.

An NRC contractor has developed a document detailing human engineering design data for CRT-based display systems. This document (Ref. 8) identifies relevant issues related to human performance in conjunction with the use of CRT-generated displays. Another recent report by an NRC contractor (Ref. 9) presents many ways of displaying data incorporating many variables within the plant's process.

5 HUMAN FACTORS GUIDELINES FOR REVIEWING AN SPDS DESIGN

The SPDS basic principles and review guidelines are presented here. These SPDS review guidelines address all SPDS display systems and emphasize guidelines applicable to CRT display systems.

The SPDS basic principles from NUREG-0737 Supplement 1, are reproduced here and then broken down into smaller components. Guidelines are provided for each component to illustrate good human factors engineering principles related to that component. In addition, examples offered for each guideline illustrate acceptable practices. These examples are not meant to be comprehensive, and NRC staff as well as designers and users of display systems could offer many other examples of good ways to implement the guideline. Also, where appropriate, the guidelines and examples are cross-referenced to NUREG-0700, Section 6.0, Control Room Human Engineering Guidelines.

In reviewing an SPDS for its incorporation of good principles of human factors engineering, the NRC reviewer could use the guidelines and examples presented in this SRP section as an aid in evaluating conformance to Section 4.1 of Supplement 1 to NUREG-0737. For designs not covered by these examples, the NRC reviewer should use those principles embodied in the guidelines as an aid in evaluating the SPDS. Additional clarification of some of the key guidelines and other appropriate human factors engineering principles are provided in Subsection 6 of this appendix, Review Guidelines for SPDS Displays, and in Section 6 of NUREG-0700, Control Room Human Engineering Guidelines. With regard to NUREG-0700, the NRC reviewer should focus on the guidelines associated with control room workspace, visual displays, labels and location aids, process computers, and panel layout in reviewing SPDS designs.

The nuclear industry has also published guidelines for an effective SPDS implementation program (Ref. 10). These guidelines appeared as a Nuclear Utility Task Action Committee (NUTAC) publication. Publications issued by NUTAC represent a consensus of utilities that participate in NUTAC. These publications are not intended to be interpreted as the industry standards. The publications are offered with the understanding that individual utilities are not obligated to use the suggestions.

The above-described NUTAC publication was not formally submitted by the industry for NRC review. The NRC reviewed the report informally and found it well structured and providing excellent guidance on program planning, system design, installation, and maintenance of the display. However, the NRC had the following comments and suggestions:

- ° The use of an operational control room as a test bed for the SPDS has the potential for misleading control room operators. This is a case where flexibility in display design may reduce safety. The control room should not be used as a test bed for developing the SPDS. In addition, it should not be possible to place the SPDS into a test mode from outside the control room. However, tests of the SPDS in the control room will be needed to confirm correct installation and to confirm that maintenance has been properly performed. For these occasions, the display format of the SPDS

should be clearly marked to reflect the test activities. A temporary sign should notify control room operators that test activities are taking place, and all members of the control room operating crew should be notified when test activities begin and when they end.

- ° The sole use of status lights--one for each critical safety function-- is not adequate for an SPDS. The variables associated with each critical safety function should also be available for display and operator assessment.
- ° The SPDS should be capable of continuously monitoring the status of critical safety functions.
- ° The SPDS should also contain trend data for the key variables displayed, because such data enhance:
 - Use of operating procedures,
 - Detection of abnormal operations,
 - Prediction capabilities of the control room operators.

The NRC reviewer should consider the above points when reviewing an SPDS design which references the subject NUTAC publication.

In the Subsections that follow (5.1 through 5.5) portions of NUREG-0737, Supplement 1, which apply human factors engineering principles to the SPDS are quoted. A subsection entitled, "Guideline," offers analysis of the component being discussed and gives one or more specific example(s) of how the applicant/licensee could apply the guideline.

5.1 NUREG-0737, Supplement 1, Section 4.1.a

"The SPDS should provide a concise display of critical plant variables to the control room operators to aid them in rapidly and reliably determining the safety status of the plant. Although the SPDS will be operated during normal operations as well as during abnormal conditions, the principal purpose and function of the SPDS is to aid the control room personnel during abnormal and emergency conditions in determining the safety status of the plant and in assessing whether abnormal conditions warrant corrective action by [control room]^a operators to avoid a degraded core. This can be particularly important during anticipated transients and the initial phase of an accident."

a. Bracketed words were added to clarify the wording in NUREG-0737, Supplement 1.

5.1.1 Concise Display

"The SPDS should provide a concise display^b. . . ."

5.1.1.1 Guideline

A concise display of critical plant variables will help the control room operator compare data from related plant functions and assess the safety status of the plant.

Examples

- (a) Critical plant variables for the SPDS are presented on the single primary display or on a group of displays at a single location.
- (b) Display formats contain patterns and enhancements that define the critical plant variables.

5.1.2 Critical Plant Variables

"The SPDS should provide a concise display of critical plant variables^c...."

5.1.2.1 Guideline

A predetermined minimum set of critical plant variables will help control room operators evaluate plant safety.

5.1.3 Rapid and Reliable Determination of Safety Status

"The SPDS should provide a concise display of critical plant variables to control room operators to aid them in rapidly and reliably determining the safety status of the plant."

5.1.3.1 Guideline^d

In order for the control room operator to rapidly and reliably determine the safety status of the plant, the displayed data should represent the current and correct status of critical plant variables.

Examples

- (a) The sampling rate for each critical plant variable is such that there is no meaningful loss of information in the data presented to the control room operator.

b. Underlining was added for emphasis.

c. The variables must be the ones determined by the applicant/licensee to be sufficient to provide the information needed by the control room operating crew to evaluate the safety status of the plant.

d. This guideline (and other guidelines designated "d") will be used in the human factors engineering evaluation of licensee's SPDS safety analysis reports for serious safety questions or seriously inadequate analysis.

- (b) The time delay from when the sensor signal is sampled to when it is displayed should be consistent with other control room displays and should be responsive to control room operators' needs in performing assigned tasks.

For each of the above examples:

- (a) Each critical plant variable is displayed with an accuracy sufficient for the control room operator to discriminate between conditions that impact the plant's safety status and normal operating conditions.
- (b) The display does not give false indications of plant status.

5.1.3.2 Guideline

In order to keep the control room operator current on the safety status of the plant, the display should be responsive to transient and accident sequences.

Examples

- (a) Operator comprehension of a change in the safety status of the plant from the SPDS display could be achieved in a matter of seconds.^e
- (b) The display system correctly portrays information about the plant's safety status for a wide range of events and includes symptoms of severe accidents.

5.1.3.3 Guideline^d

To prevent misleading the control room operator, displayed data should be validated on a "real time" basis where practical.^f

Examples

- (a) Redundant sensor readings are compared before displaying the critical plant variable. (For further guidance, see Section 6.7.2.7 of NUREG-0700.)

- e. The SPDS should provide timely information to the control room operating crew, which the crew can then use together with other available information to help it determine rapidly and reliably the plant's safety status and to assess whether corrective action is needed.
- f. This guideline ensures the display of reliable data and information to control room personnel.

- (b) Analytical redundancy among different critical plant variables is used and models and equations that have been documented and validated.^g
- (c) Validated data, unvalidated data,^h and invalid data are identified and coded where practical. (For further guidance, see Section 6.7.2.7 of NUREG-0700.)

5.1.3.4 Guideline

To instill the control room operator's confidence in the use of displayed data, members of the control room operating crew should be provided with the information and criteria they need to perform an operability evaluation of the SPDS. In addition, the crew must be able to easily recognize a failed SPDS.

Examples

- (a) The SPDS design incorporates an automatic or user-activated operability monitoring feature. (For further guidance, see Sections 6.7.2.6 and 6.7.2.7 of NUREG-0700.)
- (b) The design incorporates a display of calendar date and time of day such that the display is updated only when the system is operating properly so that a static time would indicate a system failure. The date and time would be located in a corner of the display so as not to distract the control room operator.

5.1.4 Aid to Control Room Personnel

"Although the SPDS will be operated during normal operations as well as during abnormal conditions, the principal purpose and function of the SPDS is to aid the control room personnel during abnormal and emergency conditions in determining the safety status of the plant and in assessing whether abnormal conditions warrant corrective actions by control room operators to avoid a degraded core."

- g. Operating conditions in which the equations used by the SPDS are not valid, such as the transition to two-phase liquid-vapor conditions in the primary coolant system of a pressurized water reactor, should be identified and documented. The design of the display hardware, computer hardware, and computer program for the SPDS should provide the capability for correcting identified problems.
- h. It is important that the control room operator know the validity of data, so the operator can correctly assess the safety status of the plant. Under some conditions, unvalidated data on the SPDS may be useful to trained control room operators in determining the safety status of the plant and in determining whether human intervention is needed. When the SPDS design allows presentation of unvalidated data, the SPDS users should be aware of this condition so that they will can exercise judgment on the use of the data.

5.1.4.1 Guideline

To aid the control room operating crew in evaluating the safety status of the plant during conditions that could have safety significance, the display should be capable of presenting magnitudes and trends of critical plant variables or derived variables.

Examples

- (a) The SPDS display format has the capability of indicating trends of each SPDS variable. (For further guidance, see Sections 6.7.2.1 and 6.7.2.8 of NUREG-0700.)
- (b) The display of time derivatives instead of trends may be acceptable under certain circumstances.ⁱ

For each of the above examples:

Trend data are displayed with sufficient resolution in time and magnitude to ensure that rapidly changing variables are accurately displayed. The frequency bandwidth of the signal measurement system, consisting of sensor, signal processing devices, and trend display devices, should be broad enough to transmit information of the measured variable or derived variable without extraneous background noise.

Further guidance may be found in Subsection 6.1, SPDS Data Display Formats.

5.1.4.2 Guideline

To help control room operators detect abnormal conditions which warrant corrective actions, the SPDS, where feasible, should include perceptual cues to alert personnel to the abnormal operating condition.

5.2 NUREG-0737, Supplement 1, Section 4.1.b

"Each operating reactor shall be provided with a Safety Parameter Display System that is located [so that it is] convenient to the control room operators. This system will continuously display information from which the plant safety status can be readily and reliably assessed by control room personnel who are responsible for the avoidance of degraded and damaged core events."

-
- i. Display of the time derivatives of variables is acceptable only when the derivatives unambiguously reflect the trends in the critical plant variables. The algorithm used for time derivations must be adequate to track transients or oscillations of critical plant variables that may exist during severe accident events for the plant. Trend data are generally the preferred method.

5.2.1 Convenient Location

"Each operating reactor shall be provided with a Safety Parameter Display System that is located [so that it is] convenient to the control room operators.

5.2.1.1 Guideline

To be convenient to the control room operating crew, the SPDS may be located on the control board. If the SPDS is part of the control board, it must be easily recognized and readable. (See Subsection 5.4.2.2 for additional guidance on SPDS location.)

Examples

- (a) The SPDS is readily distinguished from other displays on the control board. (For further guidance, see Sections 6.1 and 6.8 of NUREG-0700.)
- (b) The display meets the intent of the appropriate display readability guidelines stated in NUREG-0700. (For further guidance, see Section 6.7.2 of NUREG-0700.)

5.2.1.2 Guideline

The display should be located so that it is convenient to the control room operating crew and where control room operators who are responsible for avoiding degraded and damaged core events can observe the SPDS display. (See Subsection 5.4.2.2 for additional guidance on SPDS location.)

Examples

- (a) The display is readily accessible to the following personnel, but not necessarily simultaneously:
 - Shift Supervisor
 - Control Room Senior Reactor Operator
 - Shift Technical Advisor
 - One reactor operator
- (b) Members of the control room operating crew have physical access to the SPDS.

For each of the above examples:

- (a) Glare from normal or emergency lighting does not restrict the view of the SPDS from within the control room, and luminance levels and luminance contrast do not limit viewing the SPDS display. (For further guidance, see Sections 6.1.5.3 and 6.7.2.1 of NUREG-0700.)
- (b) The control room operating crew, not personnel outside the control room, control images displayed on the control room SPDS.

5.2.1.3 Guideline^d

To be convenient to the control room operating crew, the display system should not interfere with the crew's normal movement. The display system should not interfere with full visual access to other control room operating systems and with displays important for safe operation.

This guideline is self-evident; however, additional guidance may be found in Sections 6.1.1 and 6.1.2 of NUREG-0700.

5.2.2 Continuous Display

"This system will continuously display information. . . ."

5.2.2.1 Guideline

A continuous single-format primary display is not necessary. The primary display may be a continuous indication of individual plant variables or may be composed of a number of measured or derived variables. The main concern is that the SPDS users are made aware of important changes in critical safety-related variables when they occur and that the SPDS users can readily obtain information from the SPDS to help them determine the safety status of the plant.

Examples

- (a) A dedicated display, such as a CRT, continuously displays the minimum set of variables necessary to assess the safety status of the plant.
- (b) A hierarchical display system is used with control room operator-controlled means to access all levels of display formats needed to evaluate the safety status of the plant. (Further guidance may be found in Subsection 6.1, Display Formats.)
- (c) Perceptual (audible or visual) cues are provided by the system to alert the control room operator to return to the primary display format while viewing secondary information. (Further guidance may be found in Subsection 6.2, Display Techniques.)

5.3 NUREG-0737, Supplement 1, Section 4.1.c

"The control room instrumentation required (see General Design Criteria 13 and 19 of Appendix A to 10 CFR 50) provides the [control room] operators with the information necessary for safe reactor operation under normal, transient, and accident conditions. The SPDS is used in addition to the basic components and serves to aid and augment these components. Thus, requirements applicable to control room instrumentation are not needed for this augmentation (e.g., GDC 2, 3, 4 in Appendix A; 10 CFR Part 100; single-failure requirements). The SPDS need not meet requirements of the single-failure criteria and it need not be qualified to meet Class 1E requirements. The SPDS shall be suitably

isolated from electrical or electronic interference with equipment and sensors that are in use for [the] safety systems. The SPDS need not be seismically qualified, and additional seismically qualified indication is not required for the sole purpose of being a backup for [the] SPDS. Procedures which describe the timely and correct safety status assessment when the SPDS is and is not available, will be developed by the licensee in parallel with [development of] the SPDS. Furthermore, [control room] operators should be trained to respond to accident conditions both with and without the SPDS available."

5.3.1 Procedures and Training

"Procedures which describe the timely and correct safety status assessment when the SPDS is and is not available, will be developed by the licensee in parallel with [development of] the SPDS. Furthermore, [control room] operators should be trained to respond to accident conditions both with and without the SPDS available."

5.3.1.1 Guideline

As the SPDS is not a Class 1E qualified display, compensatory measures should be provided for control room operators when the SPDS is inoperable.

Example

Operating procedures and training are provided to the control room operating crew that will allow timely and correct safety status assessment when the SPDS is not operating.

5.3.1.2 Guideline

No additional operating staff other than the normal control room operating crew should be needed to operate the display during normal and abnormal plant operation and during display outages.

Examples

- (a) The control room operator's training program contains instruction and training in the use of the SPDS in conjunction with operating procedures for normal, abnormal, and emergency operating conditions.
- (b) An SPDS user's manual is available for reference in the control room.

5.4 NUREG-0737, Supplement 1, Section 4.1.e

"The SPDS display shall be designed to incorporate accepted human factors [engineering] principles so that the displayed information can be readily perceived and comprehended by SPDS users."

5.4.1 Incorporate Accepted Human Factors Engineering Principles

"The SPDS shall be designed to incorporate accepted human factors [engineering] principles. . ."

5.4.1.1 Guideline^d

The display format has to be designed to incorporate accepted human factors engineering principles.

Examples

- (a) The display format meets the intent of the applicable display guidelines in Subsection 6.0 of this appendix.
- (b) The SPDS display meets the intent of the display guidelines in NUREG-0700. (For further guidance, see Section 6.7.2 of NUREG-0700.)
- (c) The display meets the intent of other pertinent and compatible documented human factors engineering guidelines cited by the applicant/licensee.

Further guidance on human factors engineering principles may be found in Subsection 6, Review Guidelines for SPDS Displays.

5.4.2 Information Readily Perceived and Comprehended

"The SPDS display shall be designed to incorporate accepted human factors [engineering] principles so that the displayed information can be readily perceived and comprehended by SPDS users."

5.4.2.1 Guideline^d

Pattern and coding techniques are accepted human factors engineering design practices to communicate data and information to people from displays. Pattern and coding techniques should be used in the SPDS to help the [control room] operator detect and recognize unsafe plant operating conditions. (See also Subsection 6.3, Display Techniques.)

Examples

- (a) Color coding is used to indicate the approach to unsafe operation and to indicate unsafe operation. (For further guidance, see Section 6.7.2.7 of NUREG-0700.)
- (b) Limit marks are used for each critical plant variable displayed. The limit marks are representative of operational limits established by technical specifications, process limits, and safety system actuation setpoints, if applicable.

- (c) Patterns are used that noticeably distort when an unsafe condition is approached.

Further guidance may be found in Subsection 6.2, Display Techniques.

5.4.2.2 Guideline

To be readily perceived and comprehended, the SPDS display should be readable from the emergency station of the control room operator responsible for evaluating the safety status of the plant. (See Sections 5.2.11 and 5.2.12 for additional guidance on SPDS location.)

Example

The display design meets the intent of the appropriate display readability guidelines stated in NUREG-0700, such as viewing distance, viewing angle, flicker, noise, contrast, and screen location for standing and seated control room operators at the designated control room operating crew member's station. (For further guidance, see Section 6.7.2.1 of NUREG-0700.)

5.5 NUREG-0737, Supplement 1, Section 4.1.f

"The minimum information to be provided shall be sufficient to provide information to plant [control room] operators about [the following critical safety functions]:

- (i) Reactivity control
- (ii) Reactor core cooling and heat removal from the primary system
- (iii) Reactor coolant system integrity
- (iv) Radioactivity control
- (v) Containment conditions

The specific [plant] parameters [variables] to be displayed shall be determined by the licensee."

5.5.1 Sufficient Information

"The minimum information to be provided shall be sufficient to provide information to plant [control room] operators about:...."

5.5.1.1 Guideline

To monitor the plant process, the control room operator must be able to evaluate each of the above functions or their equivalents. Applicants/licensees should ensure that the selected SPDS variable(s) appropriately characterize(s) items (i)-(v) specified above. Supporting analyses should also cover these

items. The design of the display should have a single primary display format for each mode of plant operation.^j

Examples

- (a) The design has a display format for each mode of plant operation.
- (b) The design provides a primary display supported by a coordinated set of hierarchical subordinate displays for each mode of plant operation.
- (c) For each mode of operation, the displays contain the minimum set of indicators and data needed to assess the plant functions that are used to determine the plant's safety status.

5.5.1.2 Guideline

For each plant operating mode, display formats may either be automatically displayed or manually selected.

Examples

- (a) A manually operated switch or input from an alpha-numeric keyboard, touch panel, light pen, cursor, or equivalent arrangement is provided by the design to allow the control room operator to select the display format for the mode of plant operation.
- (b) Automatic display_k format change occurs with a change in the mode of plant operations.

j. Some typical modes of plant operation are power operation, startup, hot standby, and hot shutdown. Display formats composed of the same sets of variables or the same sets of hierarchical displays may be used in common for several modes of plant operation. A top level display format that is plant mode independent which is supported by mode-dependent subordinate displays may be desirable.

k Automatic change of the display format should be designed so that neither a gradual nor a rapid change in plant behavior from a condition that may have safety significance is automatically interpreted as a change in plant mode of operation. Provisions should be included for the control room operator to override automatic display format changes when necessary. In addition, there should be provisions in the display to indicate to the control room operator that a change in the mode of plant operation has occurred.

6 REVIEW GUIDELINES FOR SPDS DISPLAYS

The NRC has not explicitly described design specifications of SPDS displays. Those examples of displays given in this appendix are offered for information purposes only, to help reviewers interpret and use the review guidelines.

This section focuses on the use of computer-driven CRT displays. Several display formats are reviewed and important features of each that are pertinent to the SPDS functions are discussed. The use of SPDS displays other than CRTs is not precluded. These review guidelines are not intended to eliminate consideration of other display designs.

6.1 SPDS Data Display Formats

The mechanism for displaying the SPDS's safety information is not rigidly specified in Supplement 1 to NUREG-0737. The primary SPDS display format may be presented on a single display device or on a group of display devices concentrated at a single location specifically designated for the SPDS.

During plant operation, the primary SPDS display should accurately indicate the status of important plant functions. The SPDS should display a minimum set of variables from which the control room operating crew can assess the safety status of the plant. The minimum set of variables and the combinations of variables needed to characterize each plant function were not defined. The staff recognizes that the minimum set of variables is plant dependent and should be determined by the applicant/licensee.

The SPDS may provide a single primary display format, or it may use a system of primary and secondary display formats. When a single primary display is used, all information that control room operators need in order to assess the plant's safety status should be continuously visible to them. When the SPDS is concentrated in a single CRT display, the quantity of information sufficient to evaluate the plant's safety status may be too dense for rapid and reliable use.

A combination of primary and secondary displays may be used for the SPDS. The primary display format may provide information about a selected set of key variables, derived variables, or plant functions, or it may provide indicators to inform the control room operator about a change in the plant's safety status. With limited information displayed on the primary display format, the SPDS should prompt the control room operator to obtain more detailed data from the secondary display formats. The combined primary and secondary SPDS display formats should provide data on the complete set of variables used to assess the plant's safety status.

Use of primary and secondary display formats generally means that the control room operator must select a display format and present it on a display device. When a system of primary and secondary display formats is used (hierarchical display), the display formats should be ranked one above another in a systematic order of importance to facilitate the operator's access to information and to make it easy for the control room operator to

manipulate the displays. Computer-driven CRT display systems are well suited to the use of such display schemes.

The top level display format of a hierarchical SPDS display system could be composed of status indicators that provide information on the state of general plant functions. These indicators should provide the control room operator with enough information to detect a change in the plant's safety status and to select appropriate lower level display formats. A well-designed hierarchical display system allows the control room operator to readily select all levels of display formats.

In a hierarchical SPDS design, the top level display format may be replaced with secondary display formats when the control room operator needs more detailed information. If the primary display format is not continuously visible, provisions should be made to notify the control room operator about important changes in the status of plant functions that require attention. All lower level display formats should provide a simplified presentation of the status of general plant safety functions in addition to their detailed information, or they should signal the control room operator to return to a higher level display format when a change occurs in safety status. The hierarchical display system should be able to return quickly to the primary display format or to appropriate higher level display formats.

The information displayed on systems that display variables important to safety should be organized in formats that are easy for the control room operator to read and interpret. Acceptable formats may present information about the plant's safety status in combinations of alphanumeric, symbolic, or graphic form, and may present data about plant variables in analog or digital form. Display formats should be designed so that each specific element in a display corresponds directly and unambiguously with a single variable or function. Generally, each element of the display should have a label or other readily understood identifier that specifically associates that display element with the variable it represents.

Quantitative information about the magnitudes and time-dependent trends of the variables shown on the SPDS should be presented to help the control room operator assess the severity and dynamics of abnormal plant conditions. Magnitude and trend information need not always be present on the primary display format, provided the SPDS design allows the person using the display to readily acquire this information as needed. Magnitude and trend data may be provided on lower level display formats when a hierarchical display is used.

SPDS trend displays that show quantitative rate of change of a variable together with the direction of change may be used, provided the rate information accurately represents the trend of the variable. Trend rates presented to the control room operator should not fluctuate as a result of minor fluctuations in data or oscillatory behavior which may be superimposed on a well-defined trend of the variable. When a simple quantitative rate-of-change value is used, an indication should be provided to inform the control room operator when, as a result of minor fluctuations or oscillations, the rate

value does not accurately represent the trend of the variable.

Time-history data of the most safety-significant variables displayed or used in deriving safety functions should be available to the control room operating crew. This time history need not be presented on the SPDS if accurate data in a conveniently usable form are readily available in the control room from other data-recording instruments, such as chart recorders or computers that monitor, record, and display the processes of the plant (process computer). A time history of each safety-status variable should be provided; this should cover enough time and be accurate enough to depict the onset and development of conditions that vary from the preceding normal operating conditions. A presentation of time history data by the SPDS may appear on either the primary SPDS display formats or on secondary display formats.

6.2 Display Techniques

Because the main function of the SPDS is to assist the control room operating crew evaluate the safety status of the plant, the display should provide enhancements to improve the control room operator's perception, comprehension, and detection of operating conditions that may affect the plant's safety status. The display of these conditions should be distinctly different in appearance from the display of normal operating conditions. This distinction allows the control room operating crew to readily detect and identify operating conditions that may have safety significance, as soon as they occur.

Computer-driven CRT displays allow use of a wide variety of techniques to differentiate normal from abnormal conditions. Review guidance is provided for several techniques to ensure that, if used, each technique will provide an acceptable enhancement for the SPDS display. Much of what is contained in this section, however, may not pertain to any one particular display.

The display enhancement techniques discussed are:

- (1) Graphic representation of variables
- (2) Identification of displayed variables
- (3) Perceptual aids
 - (a) Color
 - (b) Symbols and mimics
 - (c) Graphic overlays
 - (d) Blinking
- (4) Display patterns
- (5) Status setpoints

Other display enhancement techniques may also be used.

6.2.1 Graphic Representation of Variables

SPDS displays may provide graphic representations of measured or derived plant variables. When a graphic representation is used, a change in the value of a displayed element should be readily interpreted as a corresponding change in the magnitude of the associated measured or derived variable. Generally a user most readily understands a linear relationship between the magnitude of the measured or derived value of the variable and the display element used to depict the value. In some cases, however, a nonlinear relationship between the variable and the display element is more appropriate. When a nonlinear relationship is used, it should be demonstrated that such a relationship is better understood by control room operators or that it will actually facilitate their interpretation of information. For example, a logarithmic relationship between reactor power level and the magnitude of the corresponding display element may be appropriate to display power during reactor startup, if accurate readings of reactor power level are needed over many decades.

Scaling of variables used for the SPDS display affects both the usability and the interpretability of the display, especially when pattern recognition is being used. Scales should be chosen to provide the range of data and level of accuracy that the control room operator needs in order to use the information. Displays of magnitude can, in some cases, be scaled to optimize recognition of changes from normal to abnormal plant conditions. In pattern recognition, the scale is chosen to produce under normal conditions an undistorted pattern that becomes distorted when an abnormal condition occurs. A reviewer should recognize that it may not be possible or desirable to apply such scaling to all displays. For example, if such scaling resulted in a display that is unacceptable to control room operators after they have been trained in its use, then it would not be acceptable scaling.

Scaling should also be chosen to allow tracking of variables over a wide range of abnormal conditions. Therefore, displays for normal conditions should not fill the entire display area. These displays may also provide a means of reading values should any variable go off scale during abnormal conditions. Under these circumstances, the SPDS should alert the control room operator when a variable is off scale.

It may be desirable to change the scaling factors used in a display if changes in relative magnitudes occur during plant operations. For example, normal operation at reduced power may result in a display that appears distorted relative to the display exhibited during operation at 100% power. Because reduced power operation does not necessarily represent a condition that may have safety significance, a change in scale would be appropriate to provide a display that remains undistorted. It is preferable that this type of change be made by a command by the control room operator rather than by automatic action of the display signal or data processing system. This ensures that an abnormal condition is not displayed inappropriately as the result of automatic scaling changes made by the SPDS. A system that is

designed to automatically change display scaling should alert the control room operator that the change is being made.

6.2.2 Identification of Displayed Variables

The control room operator must be able to readily interpret the information conveyed by the SPDS display. When a display changes, the control room operator must know what variables are changing and how they are changing in order to assess the nature of an abnormality and identify the system involved. Displays should include labels, symbols, or other ways of uniquely identifying each variable being displayed. It is unrealistic to rely on the control room operator to memorize the relationships between the display format or the display pattern and the specific variables being displayed.

6.2.3 Perceptual Aids

Perceptual aids can be used with all types of display mechanisms to aid the control room operator in evaluating the safety status of the plant. Among the perceptual aids suitable for use in SPDS displays are color, symbols and mimics, overlays, and blinking. Displays may use one or more of these perceptual aids, or may use none at all.

6.2.3.1 Color

Color may be used in SPDS displays to help identify and differentiate between elements of the display and to indicate a change in functional or operating status of a plant variable.

When color changes are used to indicate a change in functional or operating status, no more than three colors should be used, corresponding to two levels of change in severity of status. A neutral color should indicate normal status. The first color change could alert the control room operator that a variable is outside its normal range but does not represent a serious problem. A second more noticeable color change would occur when the variable is in a range that indicates a serious threat to safety. To be effective, the colors used in the SPDS display should be consistent with color codes used elsewhere in the control room.

Displays should avoid conflicts between the use of color coding to enhance selective identification of display elements and the use of color codes to enhance changes in operating status of displays, display elements, or displayed variables.

6.2.3.2 Symbols and Mimics

Graphic symbols and mimics may be used as distinctive ways to present information in a pictorial format. These should conform to the guidelines of NUREG-0700, Section 6.6.3.4, Symbols, and Section 6.6.6.4, Use of Mimics.

6.2.3.3 Graphic Overlays

Graphic overlays can effectively enhance displays by providing a reference to normal conditions, an indication of normal limits for individual variables, or an indication of abnormal operating ranges. An overlay of a normal pattern can enhance some graphic displays by providing immediate reference to normal operating conditions to facilitate pattern recognition or to detect deviation from normal conditions. Overlays are acceptable when they improve the control room operator's interpretation of the displayed information. Overlays should not distract the operator or interfere with observation of displayed information or interpretation of plant operating conditions.

6.2.3.4 Blinking

Blinking symbols or data on a CRT, blinking illuminated graphic displays, and blinking indicator lights and annunciator displays are effective ways to call an operator's attention to an abnormal condition. The use of blinking visual displays should conform to the guidelines of NUREG-0700, Section 6.3.3.2, Visual Alarm Recognition and Identification, and to Section 6.7.2.7, Graphic Coding and Highlighting.

6.2.4 Display Patterns

The incorporation of the display of process variables into a regular pattern can be an effective graphic representation of the plant's safety status. When a pattern is used to enhance the control room operator's assessment of the safety status of the plant, there should be a direct association between the display pattern and the status of the plant. The pattern for normal operating conditions should have distinctive characteristics that distinguish it from the patterns produced by other conditions. The change from a normal pattern to another configuration should be readily detectable.

One pattern change that is acceptable when properly designed and implemented is a change from a symmetric or regular geometric pattern during normal operating conditions to an asymmetric or irregular geometric pattern when an abnormal condition occurs. Another pattern change that may be acceptable is a change from a pattern displaying uniform magnitude or length for each variable during normal conditions to a pattern displaying unequal magnitudes or lengths for those variables that are in a different state.

A control room operator is more likely to notice changes from a normally undistorted pattern than to notice changes from an initially distorted pattern. Therefore, it is important that the display pattern for normal conditions be undistorted; then smaller differences in the pattern are required to detect a change. Production of an undistorted display pattern is largely determined by the choice of scaling for the variable. Displays relying on pattern recognition to identify an abnormal condition should be selected for variables that have small deviations about a steady-state value during normal operating conditions and that have distinctive variations from the steady state value during abnormal conditions.

Top level display formats based on shape coding, color coding, or alphanumeric coding of data and information to convey the status of plant safety to the control room operator are acceptable. However, top level display formats based only on shape coding or only on color coding or on a combination of these should be augmented with lower level display formats which are based on alphanumeric coding of data and information. Shape coding and color coding of data and information are acceptable display techniques in response to search and identification type of control room operator tasks. Alphanumeric coding of data and information is best for absolute identification of plant status, such as the safety status of the plant. Under these circumstances, a top level display format based on shape coding or color coding enhances the control room operator's perception via pattern recognition. Lower level display formats based on alphanumeric coding of plant variables and their magnitudes, and of trends or rates of variables allow a control room operator to more fully assess the safety status of the plant.

6.2.5 Status Setpoints

Setpoints are used to indicate a change in status of a variable to indicate the approach to unsafe operation. Technical considerations should establish setpoints for variables which are used to initiate changes in display presentation to alert control room operators to changes in operating status. Poorly chosen setpoints can result in frequent false alarms or failure to recognize a serious problem. Arbitrarily establishing setpoints as some nominal percentage of normal value or of maximum range generally is not appropriate. Setpoints used to indicate a change in status should be chosen specifically for their suitability to perform the desired function.

6.3 Application to Examples of Displays

Four convenient examples of displays of multivariate data were chosen for discussing the application of the guidelines developed in this section to specific displays. All of the examples were taken from a recent document prepared for NRC (Ref. 9) that presents numerous ways of displaying multivariate data in nuclear process control. Although this reference draws some conclusions on the applicability of various displays for process control, these conclusions do not necessarily apply to the SPDS functional needs.

The displays discussed in this appendix are to be considered only as examples of display concepts. The staff does not recommend that these specific displays should be adapted.

6.3.1 Bar Chart

The bar chart (Exhibit 6-1) synthesizes an array of analog meters, where each bar represents a specific variable. The length of each bar is generally proportional to the magnitude of the variable it represents. The control room operator can easily understand this type of display because analog meters are used in the control room to display the magnitude of operating variables.

Each bar on the display has a unique identification label that positively identifies the variable. Although a control room operator might memorize the positions on the bar display, the labels provide ready reference. It would not be realistic to expect a control room operator to memorize the position of each variable on the display.

The bar chart in Exhibit 6-1 would not, by itself, allow a quick assessment of the plant's safety status. Each bar has a different length, and, as demonstrated in Reference 7, the onset of conditions that may have safety significance may not be obvious to the control room operator.

Color coding the bars can be one effective way of signaling that a variable is outside its normal range. A bar color that does not attract attention is used while the variable is normal. When a variable exceeds the normal range, an attention-getting color is used on that bar. (See NUREG-0700 for a discussion on colors.) A contrast between each bar and the background may also be used in a similar way as a visual alert cue. A bar for a variable out of range would have much greater contrast with the background than that bar would have when the variable is within the normal range.

A blinking label or bar may be acceptable to call attention to an out-of-range variable. When a blinking display element is used as a visual alert cue, the blinking must not prevent the control room operator from using the display to obtain information. Blink rates should conform to NUREG-0700, paragraph 6.7.2.7.C.

A bar chart should provide a reference to the normal operating value of each variable displayed. It is also desirable to indicate the normal operating range of a variable on a bar chart when the operating range is a significant fraction of the total range. Such indications help the control room operator interpret the importance of a change.

6.3.2 Deviation Bar Chart

The deviation bar chart (Exhibit 6-2) is similar to the bar chart discussed above. However, each displayed bar represents the difference between the measured value and the normal value of a variable. Although the magnitude of a variable is generally positive, deviations from a normal value may be

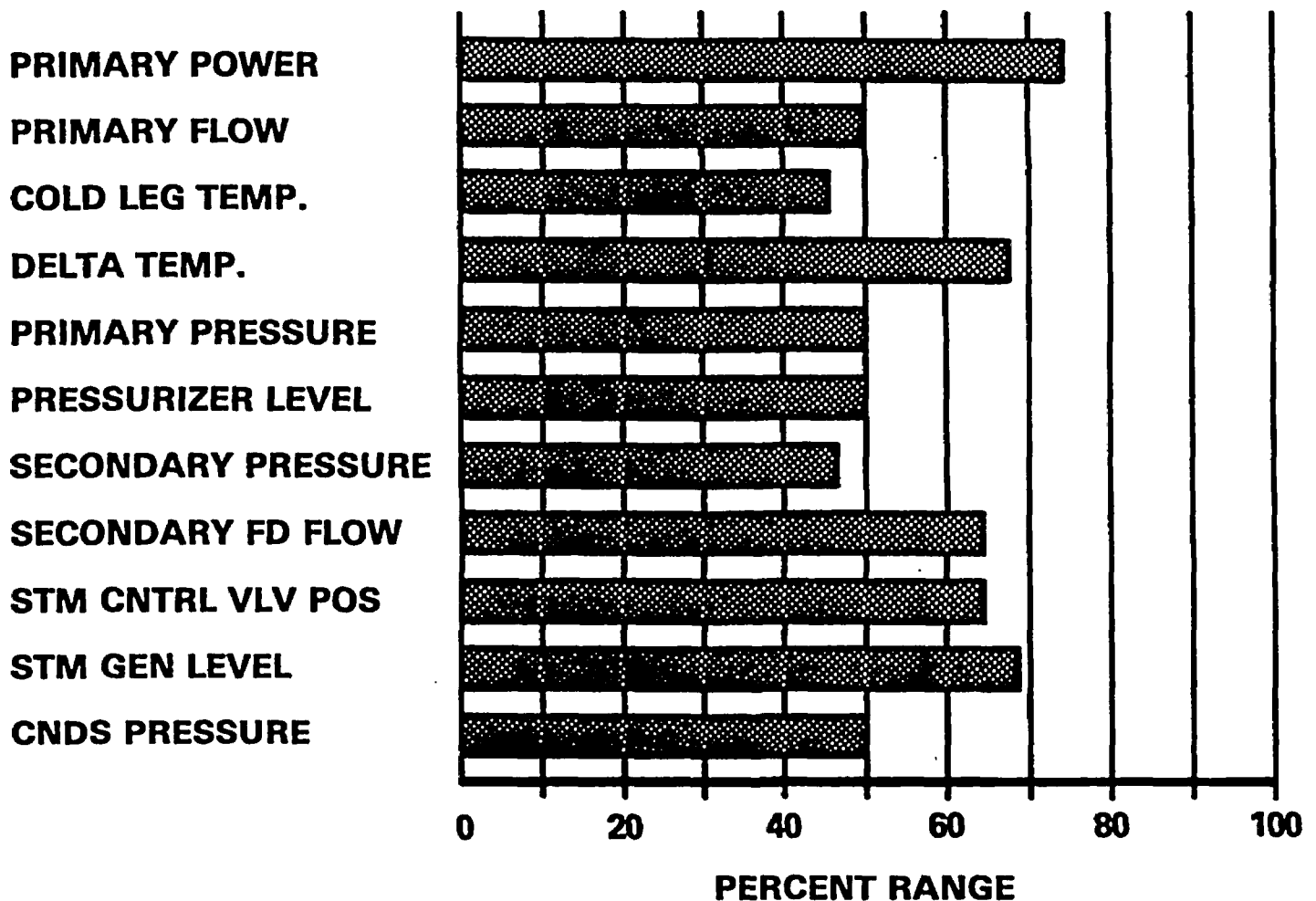


Exhibit 6-1 Simple Bar Chart Representing Normal Conditions

PRIMARY FLOW
 COLD LEG TEMP.
 DELTA TEMP.
 PRIMARY PRESSURE
 PRESSURIZER LEVEL
 SECONDARY PRESSURE
 SECONDARY FD FLOW
 STM CNTRL VLV POS
 STM GEN LEVEL
 CNDS PRESSURE

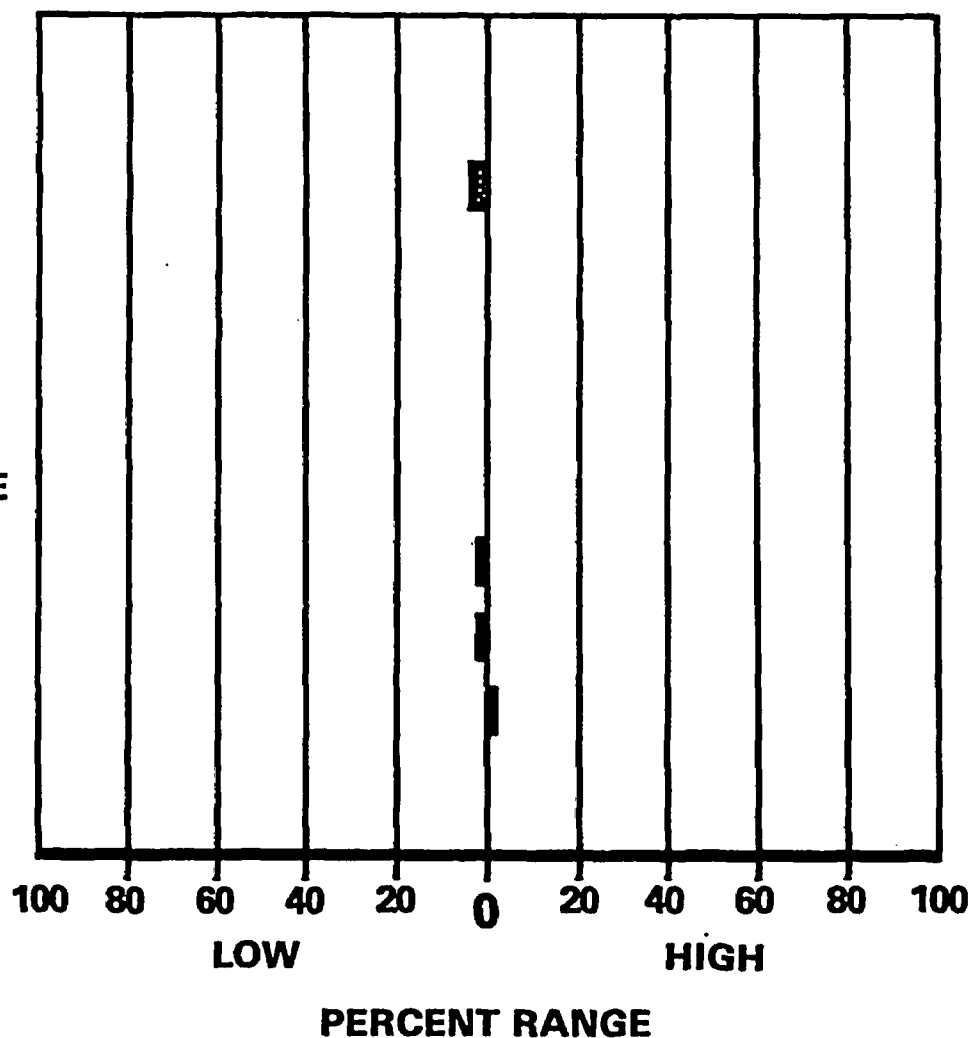


Exhibit 6-2 Deviation Bar Chart Representation of Normal Conditions

either positive or negative. Therefore, the zero reference should be in the center of the deviation bar chart. With this display, the control room operator can easily detect a significant deviation from a normal value.

There is a direct association between the deviation bar chart display and the safety status of the plant. Under normal conditions the bar chart deviations are small. In the event of a change, the magnitude and direction of a change in the variable from the normal condition is readily determined from the length and direction of the associated deviation bar.

The choice of scaling for each of the deviation bars is important to ensure that there is a distinct difference between normal and abnormal conditions. Deviation bars that can vary over the entire display range under normal conditions would be unacceptable. The range of normal conditions for positive or negative deviations should represent no more than 10% of the total range provided to display that variable's deviation. The normal deviation should also be considerably less for a variable that changes little during normal conditions but can vary a large amount when an abnormality occurs. An indication of the normal range for each deviation is desirable. When appropriately scaled, pattern recognition can help to detect an abnormal condition. Like the bar chart, a label should identify each bar. Thus a deviation in one bar can be readily associated with the corresponding variable.

Color coding or contrast may be used as a visual alert indicator on a deviation bar chart in the same way it is used with the conventional bar chart.

Some way to indicate the magnitude of each variable should be provided when the deviation bar display is used as a primary SPDS display format because this information is not included in the deviation bar chart itself. This could be done by a digital readout of the magnitude placed on the deviation bar display or by presenting information about magnitude on secondary display formats.

6.3.3 Circular Profile

The circular profile can be considered to be a variation of the bar chart. In the circular profile display, the lines that define each variable radiate from a common origin with equal angles between lines (Exhibit 6-3). The length of each line is proportional to the measurement of the corresponding variable. The endpoints of adjacent radial lines are generally connected to form the profile. The area within the profile may also be shaded for enhanced contrast.

The circular profile represents a display type where pattern recognition is the primary means of identifying an abnormal operating condition. An operator's attention is focused on the profile around the radial lines rather than on individual lines. Under normal conditions, this profile should be

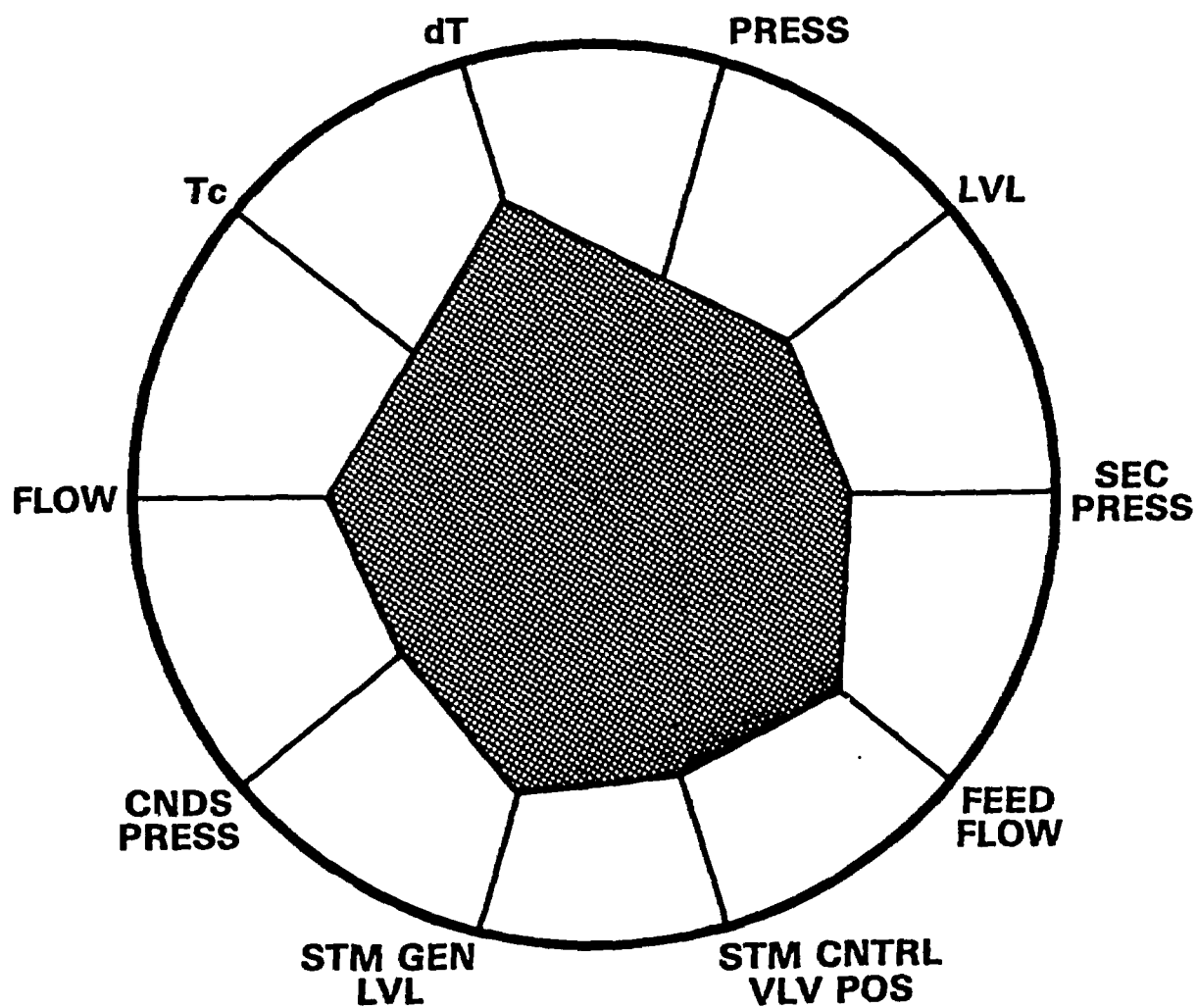


Exhibit 6-3 Circular Profile Representation of Normal Conditions

circular or regular. When an abnormal condition occurs, the profile would become noticeably distorted, indicating that an abnormal condition has developed. Scaling and variables selected are more important in producing a good symmetric circular profile display during normal operating conditions than they are for bar chart or deviation bar chart displays.

6.3.4 Chernoff Face

The Chernoff face is a graphic technique that maps multivariate data into facial features. Changes in magnitudes are translated into a change in the facial expression. Use of this type of display is dependent on pattern recognition to interpret data.

The Chernoff face is a good example of a display in which individual variables cannot be readily identified. This weakness stems from the assignment of variables to facial features. A frowning mouth is a composite of three variables. It may not be possible to identify which particular variable has changed when the mouth changes shape.

It also is difficult to relate a given change in the appearance of the face to a specific change in the safety status of the power plant. A control room operator can make no direct association between the facial features observed and the magnitude of plant variables. Many different linear and nonlinear mappings are used to relate the data being displayed to the different facial features. This complexity makes it difficult to evaluate changes in magnitude of the displayed variables. Use of this type of display would require control room operators to not only memorize the associations between variables and facial characteristics but also to memorize many different facial patterns in order to evaluate changes in variables.

Studies using Chernoff faces have shown that certain combinations of changes in the facial characteristics can result in a face that does not appear distorted (Ref. 11). Thus, there may not be a noticeable distinction between safe plant conditions and certain unsafe conditions.

These unfavorable characteristics make the Chernoff face unacceptable for use as the primary display format of an SPDS.

7 VERIFICATION AND VALIDATION OF SPDS

For the SPDS to fulfill its function, it is essential that it incorporate the basic principles discussed in Supplement 1 to NUREG-0737 so that it can provide reliable information from which the plant's safety status can be assessed. The SPDS user must have confidence in the validity of the information provided by the operational SPDS.

To ensure that a high quality SPDS is implemented, the applicant/licensee should conduct a verification and validation (V&V) program throughout the process of design, fabrication, testing, and installation of the SPDS. A V&V program should include the following:

- (1) A review of desired system capabilities to determine that the functional needs will be satisfied.
- (2) A design verification review performed after the system is initially designed to verify that the design will satisfy functional needs.
- (3) Validation tests performed after the system is assembled to confirm that the operating system satisfies functional needs.
- (4) Field verification tests performed after the system is installed to verify that the validated system was installed properly.
- (5) Documents that contain design modifications, resolutions to problems, and analyses of problems that still need to be corrected. These documents should serve to record and resolve all design problems identified by the V&V program. Qualified individuals who were not directly involved in the design, development, and installation of SPDS equipment or software should conduct the V&V program.

A V&V program performed by the applicant/licensee during design, installation, and implementation of an SPDS will facilitate the NRC staff review of the system. The staff would then evaluate the program for the results of the design V&V program. On the basis of an effective V&V program, the staff would reduce the scope and detail of the technical audit of the design.

The Nuclear Safety Analysis Center (NSAC) has prepared guidance for the nuclear industry on the SPDS V&V program for the SPDS. This guidance is documented in NSAC 39 (Ref. 12). The NRC staff will accept a V&V Program Plan which conforms to the guidance of NSAC 39. Other SPDS V&V programs which accomplish the desired goals would be equally acceptable to the staff.

8 NRC STAFF HUMAN FACTORS ENGINEERING REVIEW OF SPDS

The staff will use the guidance provided in this appendix in reviewing the design of the SPDS insofar as incorporating good principles of human factors engineering. This appendix presents a discussion of ways of incorporating basic principles of a good SPDS as set out in Supplement 1 of NUREG-0737 insofar as consideration of the human factor. For reviews of operating license applications (OL reviews) and for operating reactors for which the licensee has requested a preimplementation review of the SPDS design, the guidance offered in this appendix will assist the staff in evaluations under Supplement 1 of NUREG-0737. For such reviews, the NRC staff will initially evaluate the applicant/licensee's verification and validation (V&V) program plan and will audit the results of its design verification activities. Subsequently, the staff will audit the applicant/licensee's design validation program, test plans, and test results. During each audit, the staff plans to review safety analysis data and human factors engineering design data prepared by the applicant/licensee as well as to review its V&V activities.

These reviews will be conducted using the appropriate guidance provided in Subsections 5, 6, and 7 of this appendix. For preimplementation reviews, the NRC staff intends to conduct the review in two audit meetings with the applicant/licensee during the period of SPDS design and design validation tests. The staff will document its findings after each meeting. A third audit may be conducted following SPDS installations.

Prompt implementation of well-designed safety parameter display systems in operating reactors is a design goal of primary importance. The review process for operating reactors outlined in NUREG-0737, Supplement 1, is designed to avoid delays resulting from the time required for NRC staff review. The NRC staff will not review operating reactor SPDS designs pursuant to Supplement 1 of NUREG-0737 before implementation unless a licensee has specifically requested a preimplementation review. The licensee's safety analysis and SPDS implementation plan will be reviewed by the NRC staff only to determine if a serious safety question is posed by the proposed system or if the analysis is seriously inadequate. The NRC staff human factors engineering review to accomplish this will be directed at (1) confirming that means are provided to ensure that the data displayed are valid, and (2) confirming that the licensee has committed to a human factors engineering program to ensure that the displayed information can be readily perceived and comprehended so as not to mislead the control room operator. If, on the basis of this review, the staff identifies a serious safety question or seriously inadequate analysis, the Director of IE or the Director of NRR may request or direct the licensee to correct the deficiency or even to cease implementation. Those guidelines of Subsection 5 of this appendix that would be used in the evaluation of a licensee's SPDS safety analysis report for serious safety questions or seriously inadequate analysis have been footnoted.

Although it is unlikely that the SPDS design would raise a serious safety question or that the analysis would be seriously inadequate, the NRC staff review may identify some human factors engineering problem areas, which if corrected, could enhance effectiveness and improve control room operating crew acceptance. Problem areas that are identified by the NRC staff in its safety analysis review should be assessed for correction by the licensee during the detailed control room design review.

9 GLCSSARY OF TERMS

Many of the terms used in this document are specific to the fields of nuclear engineering and computer-software engineering. This glossary of terms is included to help the reader understand the terms as they are used in this document.

accuracy - A measure of the degree to which the actual output of a device approximates the output of an ideal device nominally performing the same function (IEEE Standard Dictionary of Electrical and Electronics Terms). In the control room of a nuclear power plant, the device is the entire measurement system from the sensor to the display of a nuclear power plant.

analytical redundancy - Intercomparison of measured variables, through the use of mathematical models based upon known physical relationships, between variables, to determine whether there are inconsistencies in the values of the measured variables (e.g., "reactor power," "reactor coolant temperature rise through the reactor core," and "reactor coolant flow rate" are interrelated variables based upon the physical principles of heat transfer. A measured value for coolant flow should be consistent with the analytically calculated value for coolant flow derived mathematically from the corresponding measured values of reactor power and coolant temperature rise.)

cathode-ray tube (CRT) - An electronic vacuum tube containing a luminescent display screen and a controlled beam of electrons that creates and refreshes images on the display screen.

control room operating crew - A group of individuals assigned to perform functions and tasks in a nuclear power plant control room to operate the plant. As such, the control room operating crew is a system within the power plant.

control room operator - An individual member of the control room operating crew including, but not necessarily limited to, a licensed reactor operator.

data - 1. An individual fact, statistic, or piece of information or a group or body of facts, information, statistics, or the like, either historical or derived by calculation or experimentation (The Random House College Dictionary, Revised Edition, 1980). 2. A general term used to denote facts, numbers, and symbols that refer to the state of the plant process or the status of systems and components that are part of the plant process.

derived variable - 1. A plant process variable derived from mathematical calculations that use the values of directly measured variables as inputs to the calculations or a variable determined by operational manipulation of the signals from directly measured variables. 2. A variable that is not measured directly but that can be derived analytically from the values of two or more measured variables (e.g. degrees subcooling can be derived from measured values of water temperature and pressure using the known physical properties of water as a function of temperature and pressure).

design criteria - Performance requirements and specifications for a system established as a goal prior to initiating detailed design of the system.

design validation - A process of system integration, testing, and evaluation activities carried out at the system/subsystem level to ensure that the developed operational product satisfies the system specifications and the user's functional requirements.

design verification - A process of iterative evaluation during the design process to determine whether the products of each step of the design effort are correct and fulfill design criteria.

display - A visual record that may be of either a permanent or temporary nature (Standard Dictionary of Computer and Information Processing, Revised Second Edition, 1978).

display format - The arrangement of characters, symbols, and visual representations on the display surface of a display unit.

display unit - A unit of hardware that provides a visual presentation of data and information on a display surface.

function - 1. The purpose for which something is designed or exists (The Random House College Dictionary, Revised Edition, 1980). 2. The performance that must be accomplished by a system to fulfill its assigned role or purpose.

hierarchical display - A display system having sets of display formats ranked one above another in a specified order of rank or importance.

information - 1. Knowledge communicated or received concerning a particular fact or circumstance. 2. Any data that can be coded for processing by a computer or similar data processing device (The Random House College Dictionary, Revised Edition, 1980). 3. The results obtained from data processed by pre-specified means or methods.

invalid data - Data that have been checked for accuracy and have failed to meet the specified criteria for validity.

measured variable - A plant process variable such as temperature, pressure, etc. that can be measured by a sensor instrument with the output signal from the sensor manipulated or converted to be displayed or read out on a display device as a magnitude of the variable, expressed in engineering units.

minimum set of critical plant variables - The fewest plant variables sufficient for the control room operators to evaluate the safety status of the plant.

perceptual aid - A display aid that assists the control room operator sense a significant change in the information provided by a display.

primary function - The principal or main purpose for which a system exists.

process computer - A computer that monitors, records and displays the processes of the plant.

process control - The collective functions performed in and by equipment in which a process variable is to be controlled.

process limit - A value of a process variable wherein a significant change in the process occurs; an example of a process limit is zero degrees of subcooling for water in conjunction with the inception of boiling which converts liquid water to a steam vapor.

process variable - A term or set of terms that characterizes a specific time-varying property of the state of a plant process quantitatively in engineering units (e.g., reactor core coolant inlet temperature, 545°F).

real time - Relating to the performance of computing during the specific time in which the related process, event, problem or communication is taking place, i.e., the computing must be fast enough, during the process of the happening of the event for the results of this computing to influence the related process or result (Computer Dictionary and Handbook, Second Edition, 1972).

time history - Data that display the magnitudes of a variable over a specified time interval.

trend data - Information that depicts whether the magnitude of a variable is changing or remaining constant.

unvalidated data - Data that have not been checked for accuracy. (Unvalidated data may be determined to be either valid or invalid if subjected to a data validation process.)

validated data - Data that have been subjected to the data validation process and meets the specified criteria for data validity.

validate - To substantiate or confirm (The Random House College Dictionary, Revised Edition, 1980).

validity - The degree to which an event, especially operations, are allowable, permissive, logical, complete, and comprehensible. Validity is a measure of the extent to which a standard has been met or a rule followed (Standard Dictionary of Computer and Information Processing, Revised Second Edition, 1978).

variable - A quantity or mathematical function that may assume any given value or set of values (The Random House College Dictionary, Revised Edition, 1980).

verification - A formal act or process to ascertain the truth, authenticity, or correctness of something (The Random House College Dictionary, Revised Edition, 1980).

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Enclosure 2

Resolution of CRGR Comments on
Draft NUREG-0835

CRGR reviewed draft NUREG-0835 at its Meeting Number 57 on February 15, 1984. Minutes of this meeting issued on March 16, 1984, expressed the CRGR concerns summarized below:

1. Considerable ambiguity and complicated language.
2. Use of NUREG document to generate/establish generic requirements is contrary to NRC policy.
3. Does not address when, how and by whom the SPDS will be used.
4. Exceeds design requirements originally intended in Supplement 1 to NUREG-0737.
 - a. SPDS operability is believed to be only required for plant conditions involving power, hot shutdown, or hot standby conditions.
 - b. A need for audible alarms has not been previously identified.
 - c. Time derivative data display is believed to have little practical value and should not be promoted as an acceptable substitute for clear trend information.
5. Document allows great flexibility on the part of individual reviewers and acceptance criteria to be defined based upon many available industry human factors documents. Guidance is referenced that has not been reviewed.
6. Acceptance guidelines are found in at least two sections (Sections 5 and 6) and it is difficult to determine minimum system capabilities that are considered necessary for system acceptance.

Since review of this document by CRGR, it has undergone a number of revisions to address CRGR concerns. In addition, both ELD and our technical editors have reviewed the document and their comments have been incorporated. The major change has been to make the proposed NUREG into an appendix to Section 18.2 of the Standard Review Plan. Our resolution of the specific CRGR comments that were summarized above is as follows:

1. The ambiguity and complicated language have been eliminated. Redundant sections have also been eliminated and we have clarified that the document is for staff use in reviewing SPDS designs. The final version of the document has been reviewed by both ELD and our technical editors.

2. We propose to publish this document as an appendix to Section 18.2 of the Standard Review Plan as a result of CRGR and EDO comments.
3. Supplement 1 to NUREG-0737 describes the SPDS as an operator aid in rapidly and reliably determining the safety status of the plant. It also explains that the SPDS will display information to control room personnel who are responsible for the avoidance of degraded and damaged core events. We do not believe that a specific user should be specified by the NRC. This is up to the utility and strongly depends on the operating philosophy of a utility.
- 4a. Although Supplement 1 to NUREG-0737 states that the SPDS will be operated during normal operations as well as abnormal conditions and will continuously display information, we have adopted CRGR's comment and have eliminated refueling and cold shutdown modes of operation as areas of NRC review.
- 4b. We have clarified the reference to audible alarms as follows:
"perceptual (audible or visual) cues are provided to alert the control room operator to return to the primary display format while viewing secondary information." The main concern is that SPDS users are made aware of important changes in the primary display when they are viewing secondary displays.
- 4c. We have stressed that trend data are preferred and that time derivatives are acceptable only when the derivatives unambiguously reflect trends in critical plant variables.
5. The document does allow flexibility because of the variety of SPDS designs. We have tried to allow sufficient latitude for the review of different systems as well as different displays. We have eliminated reference to reviewer use of guidance that has not been reviewed. Industry can utilize non-reviewed human factors guidelines which the staff will review on a case-by-case basis.
6. Section 5 presents Human Factors Guidelines for reviewing the SPDS system while Section 6 presents guidelines for reviewing computer-driven CRT displays that may be part of an SPDS. We have clarified the distinction between the two sections. Minimum system capabilities that are considered necessary for SPDS acceptance are defined in Supplement 1 to NUREG-0737 as being those sufficient to provide information to plant operators about the following critical safety functions:
 - (i) Reactivity control
 - (ii) Reactor core cooling and heat removal from the primary system

- (iii) Reactor coolant system integrity
- (iv) Radioactivity control
- (v) Containment conditions

The human factors engineering review is directed at (1) confirming that means are provided to ensure that the data displayed are valid, and (2) confirming that the licensee has committed to a human factors engineering program to ensure that the displayed information can be readily perceived and comprehended so as not to mislead the control room operator.

The parameter selection review is directed at confirming that the plant specific parameters selected as the bases for a critical safety function are sufficient to assess the safety status of that function for a wide range of events. Justification for the choice of parameters can be provided by reference to analyses supporting approved generic emergency procedure guidelines. The types of scenarios to be considered include those which result from events identified in SRP Section 15.0. DHFS plans to document the process used for parameter selection review.

NRC FORM 335 (7-77)		U.S. NUCLEAR REGULATORY COMMISSION BIBLIOGRAPHIC DATA SHEET		1. REPORT NUMBER (Assigned by DDC) NUREG-0800 (formerly NUREG-75/087)	
4. TITLE AND SUBTITLE (Add Volume No., if appropriate) STANDARD REVIEW PLAN FOR THE REVIEW OF SAFETY ANALYSIS PORTS FOR NUCLEAR POWER PLANTS," LWR Edition, July 1981				2. (Leave blank)	
7. AUTHOR(S)				3. RECIPIENT'S ACCESSION NO.	
9. PERFORMING ORGANIZATION NAME AND MAILING ADDRESS (Include Zip Code) Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, D. C. 20555				5. DATE REPORT COMPLETED MONTH YEAR July 1981	
12. SPONSORING ORGANIZATION NAME AND MAILING ADDRESS (Include Zip Code) Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, D. C. 20555				DATE REPORT ISSUED MONTH YEAR July 1981	
13. TYPE OF REPORT STANDARD REVIEW PLAN				6. (Leave blank)	
15. SUPPLEMENTARY NOTES				8. (Leave blank)	
16. ABSTRACT (200 words or less) The Standard Review Plan (SRP) provides guidance to staff reviewers in the Office of Nuclear Reactor Regulation who perform safety reviews of applications to construct or operate nuclear power plants. The SRP assures the quality and uniformity of the staff reviews and presents a well-defined base from which to evaluate proposed changes in the scope and requirements of the staff reviews. The SRP makes information about the regulatory licensing process widely available and improves communication and understanding of the staff review process by interested members of the public and the nuclear power industry. NUREG-0800 consists of 18 Chapters, approximately 220 individual SRP sections. Each individual SRP section addresses the responsibilities for performing the review, the matters that are reviewed, the Commission's regulations and acceptance criteria necessary for the review, how the review is accomplished, the conclusions that are appropriate and the implementations requirements.				10. PROJECT/TASK/WORK UNIT NO.	
17. KEY WORDS AND DOCUMENT ANALYSIS				11. CONTRACT NO.	
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IDENTIFIERS/OPEN-ENDED TERMS					
18. AVAILABILITY STATEMENT Unlimited		19. SECURITY CLASS (This report) Unclassified		21. NO. OF PAGES	
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