

September 6, 2005

Mr. Mark B. Bezilla
Vice President-Nuclear, Davis-Besse
FirstEnergy Nuclear Operating Company
Davis-Besse Nuclear Power Station
5501 North State Route 2
Oak Harbor, OH 43449-9760

SUBJECT: DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1 - ISSUANCE OF
AMENDMENT RE: REACTOR VESSEL INTERNALS VENT VALVES
(TAC NO. MC6955)

Dear Mr. Bezilla:

The Commission has issued the enclosed Amendment No. 268 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit 1. The amendment revises the Technical Specifications in response to your application dated May 11, 2005.

This amendment modifies Surveillance Requirement 4.4.10.1.b to permit a one-time extension of the surveillance interval for the reactor vessel internal vent valves from September 22, 2005, to March 25, 2006.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

William A. Macon, Jr., Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosures: 1. Amendment No. 268 to NPF-3
2. Safety Evaluation

cc w/ends: See next page

September 6, 2005

Mr. Mark B. Bezilla
Vice President-Nuclear, Davis-Besse
FirstEnergy Nuclear Operating Company
Davis-Besse Nuclear Power Station
5501 North State Route 2
Oak Harbor, OH 43449-9760

SUBJECT: DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1 - ISSUANCE OF
AMENDMENT RE: REACTOR VESSEL INTERNALS VENT VALVES
(TAC NO. MC6955)

Dear Mr. Bezilla:

The Commission has issued the enclosed Amendment No. 268 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit 1. The amendment revises the Technical Specifications in response to your application dated May 11, 2005.

This amendment modifies Surveillance Requirement 4.4.10.1.b to permit a one-time extension of the surveillance interval for the reactor vessel internal vent valves from September 22, 2005, to March 25, 2006.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,
/RA/

William A. Macon, Jr., Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosures: 1. Amendment No. 268 to NPF-3
2. Safety Evaluation

cc w/encs: See next page

DISTRIBUTION:

PUBLIC	RidsNrrDlpmLpdiii(A) (HNieh)	RidsOgcRp
PDIII/2 R/F	RidsNrrDlpmLpdiii2 (GSuh)	RidsAcrcAcnwMail
RidsNrrPMJHopkins	RidsNrrLADClarke	TBoyce, O-12H2
GGrant, RIII	RidsRgn3MailCenter	RidsNrrDlpmDpr GHill (2)

Package Accession Number: ML051930375

Amendment Accession Number: ML051930340

Nrr-058

TS Accession Number: ML052500242

OFFICE	PM:PD3-2	LA:PD3-2	SRXB-B	IROB-A	OGC	SC:PD3-2
NAME	WMacon	THarris	JNakoski	TBoyce	Uttal NLO	GSuh
DATE	8/4/05	07/14/05	07/19/05	8/5/05	8/12/05	9/06/05

OFFICIAL RECORD COPY

Davis-Besse Nuclear Power Station, Unit 1

cc:

Mary E. O'Reilly
FirstEnergy Corporation
76 South Main St.
Akron, OH 44308

Manager - Regulatory Affairs
FirstEnergy Nuclear Operating Company
Davis-Besse Nuclear Power Station
5501 North State - Route 2
Oak Harbor, OH 43449-9760

Director, Ohio Department of Commerce
Division of Industrial Compliance
Bureau of Operations & Maintenance
6606 Tussing Road
P.O. Box 4009
Reynoldsburg, OH 43068-9009

Regional Administrator
U.S. Nuclear Regulatory Commission
801 Warrenville Road
Lisle, IL 60523-4351

Michael A. Schoppman
Framatome ANP
24 Calabash Court
Rockville, MD 20850

Resident Inspector
U.S. Nuclear Regulatory Commission
5503 North State Route 2
Oak Harbor, OH 43449-9760

Barry Allen, Plant Manager
FirstEnergy Nuclear Operating Company
Davis-Besse Nuclear Power Station
5501 North State - Route 2
Oak Harbor, OH 43449-9760

Dennis Clum
Radiological Assistance Section Supervisor
Bureau of Radiation Protection
Ohio Department of Health
P.O. Box 118
Columbus, OH 43266-0118

Carol O'Claire, Chief, Radiological Branch
Ohio Emergency Management Agency
2855 West Dublin Granville Road
Columbus, OH 43235-2206

Zack A. Clayton
DERR
Ohio Environmental Protection Agency
P.O. Box 1049
Columbus, OH 43266-0149

State of Ohio
Public Utilities Commission
180 East Broad Street
Columbus, OH 43266-0573

Attorney General
Office of Attorney General
30 East Broad Street
Columbus, OH 43216

President, Board of County
Commissioners of Ottawa County
Port Clinton, OH 43252

President, Board of County
Commissioners of Lucas County
One Government Center, Suite 800
Toledo, OH 43604-6506

The Honorable Dennis J. Kucinich
United States House of Representatives
Washington, D.C. 20515

The Honorable Dennis J. Kucinich
United States House of Representatives
14400 Detroit Avenue
Lakewood, OH 44107

Mr. Joseph J. Hagan
Chief Operating Officer
FirstEnergy Nuclear Operating Company
Davis-Besse Nuclear Power Station
5501 North State Route 2
Oak Harbor, OH 43449-9760

FIRSTENERGY NUCLEAR OPERATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 268
License No. NPF-3

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the FirstEnergy Nuclear Operating Company (the licensee) dated May 11, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 268, are hereby incorporated in the license. FirstEnergy Nuclear Operating Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 14 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Gene Y. Suh, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 6, 2005

ATTACHMENT TO LICENSE AMENDMENT NO. 268

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove

Insert

3/4 4-31

3/4 4-31

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 268 TO FACILITY OPERATING LICENSE NO. NPF-3
FIRSTENERGY NUCLEAR OPERATING COMPANY
DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1
DOCKET NO. 50-346

1.0 INTRODUCTION

By application to the U.S. Nuclear Regulatory Commission (NRC, Commission) dated May 11, 2005 (Accession No. ML051370132), FirstEnergy Nuclear Operating Company (the licensee) requested changes to the Technical Specifications (TSs) for the Davis-Besse Nuclear Power Station, Unit 1 (DBNPS).

The proposed changes would revise TS 3/4.4.10, "Reactor Coolant System - Structural Integrity, American Society of Mechanical Engineers [ASME] Code Class 1, 2, and 3 Components," to allow a one-time extension of the surveillance interval that is applicable to Surveillance Requirement (SR) 4.4.10.1.b. SR 4.4.10.1.b currently requires that the reactor vessel internals vent valves be tested and inspected at least once per 24 months during shutdown.

SR 4.4.10.1.b was last performed on March 25, 2003, during the 13th refueling outage. The surveillance interval, including the 25% extension afforded by the provisions of TS 4.0.2, will expire on September 22, 2005, which is prior to the scheduled start of the 14th refueling outage in March 2006. Since the SR can only be performed in Operational Mode 6 (Refueling), a one-time extension to the surveillance interval is necessary to avoid an unwarranted plant shutdown. The proposed one-time extension would allow the next performance of SR 4.4.10.1.b to be delayed until March 25, 2006, which is 36 months from the last performance.

The proposed changes superseded a previous license amendment request submitted by letter dated April 29, 2004 (Accession No. ML041250176), which was noticed in the *Federal Register* (69 FR 29766) on May 25, 2004. This previous request proposed to relocate the surveillance requirement to the DBNPS Technical Requirements Manual.

2.0 REGULATORY EVALUATION

Title 10 of the *Code of Federal Regulations* (10 CFR), Part 50.55a, "Codes and standards," contains requirements for inservice testing of ASME Code Class 1, 2, and 3 components. DBNPS is subject to the requirements of the ASME Code for Operation and Maintenance of Nuclear Power Plants, 1995 Edition through 1996 Addenda (ASME OM Code). The reactor vessel internals vent valves are not classified as ASME Code Class 1, 2, or 3 components; however, they have been added to the DBNPS inservice testing program and are tested in accordance with the TSs at the frequency required for check valves per the ASME OM Code.

3.0 TECHNICAL EVALUATION

The four reactor vessel internals vent valves at DBNPS are vertically mounted 14-inch swing check valves located on the core support shield. The valves promote steam relief from the core following certain loss-of-coolant accidents which enhances the effectiveness of emergency core cooling systems.

The valve's components are designed to minimize the possibility of loss of parts and the operating fasteners include a positive locking device. The valve materials were selected based on corrosion resistance, surface hardness, anti-galling characteristics, and compatibility with mating materials. Additionally, reactor coolant system (RCS) chemistry is controlled to minimize corrosion of RCS components. The licensee states that since the performance of the last surveillance, there has been no chemistry condition that would have an adverse impact on RCS components.

The valve's hinge assembly consists of a shaft, two valve body journal receptacles, two valve disc journal receptacles, and four flanged shaft journals. The licensee states that the hinge assembly provides eight loose rotational clearances and two end clearances. The licensee also states that at least three clearances of a particular combination must bind or seize to adversely affect the valve disc-free motion.

The licensee reviewed the maintenance and surveillance history of the valves at DBNPS. The review showed that there have been no surveillance test failures and there has been no maintenance required or performed on the reactor vessel internals vent valves. Additionally, Framatome-ANP conducted a survey of the other operating Babcock & Wilcox plants regarding their experience with reactor vessel internals vent valve surveillances and did not find a single instance of valve failure. These reviews indicate a high degree of reliability for the reactor vessel internals vent valves since no degradation in valve operability has been observed.

Previously, by license Amendment No. 95, dated August 20, 1986 (Accession No. ML021190297), the NRC staff evaluated and approved a one-time extension to the reactor vessel internals vent valve surveillance interval for DBNPS to approximately 42 months. There were no valve failures when the surveillances were performed in 1988 after the 42 month surveillance interval. This request is for a surveillance interval of 36 months, which is conservatively bounded by the previously evaluated one-time extension.

The NRC staff has reviewed the information provided by the licensee and, based on the above, the NRC staff concludes that a one-time extension of the surveillance interval for the reactor vessel internals vent valves to March 25, 2006, is acceptable.

The licensee also provided a discussion of risk considerations in support of their request. While the NRC staff supports licensees evaluating risk and providing that information, the subject application was not risk-informed; therefore, the NRC staff did not perform a risk review. The licensee's risk considerations were neither evaluated by the NRC staff nor relied upon in making the determination of the acceptability of the licensee's application (i.e., it was reviewed and approved solely based on the deterministic information provided).

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Ohio State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (70 FR 38719). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The NRC staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Hopkins

Date: September 6, 2005