

September 26, 2005

Mr. Christopher M. Crane, President
and Chief Nuclear Officer
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: QUAD CITIES AND DRESDEN NUCLEAR POWER STATIONS - ISSUANCE OF
AMENDMENTS FOR ADDITION OF OSCILLATION POWER RANGE MONITORS
(TAC NOs. MC2198, MC2199, MC2200, AND MC2201)

Dear Mr. Crane:

The Commission has issued the enclosed Amendment No. 216 to Facility Operating License No. DPR-19 and Amendment No. 208 to Facility Operating License No. DPR-25 for Dresden Nuclear Power Station, Units 2 and 3, and Amendment No. 227 to Facility Operating License No. DPR-29 and Amendment No. 222 to Facility Operating License No. DPR-30 for Quad Cities Nuclear Power Station, Units 1 and 2, respectively. The amendments are in response to your application dated February 27, 2004, as supplemented by letters dated October 11, 2004, and January 3, August 11, and September 12, 2005.

The amendments add Technical Specifications containing operability requirements for the Oscillation Power Range Monitor (OPRM) channels, associated surveillance requirements, Core Operating Limits Report OPRM requirements, and a reference for the approved methodology for determining setpoints for the OPRM trip function.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Maitri Banerjee, Senior Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-237, 50-249, 50-254, and 50-265

Enclosures: 1. Amendment No. to DPR-19
2. Amendment No. to DPR-25
3. Amendment No. to DPR-29
4. Amendment No. to DPR-30
5. Safety Evaluation

cc w/encls: See next page

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EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 216
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Exelon Generation Company, LLC (the licensee) dated February 27, 2004, as supplemented by letters dated October 11, 2004, and January 3, August 11, and September 12, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-19 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 216, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented before restart from the refueling outage which begins in October 2005.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by George Dick for/

Gene Y. Suh, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 26, 2005

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 208
License No. DPR-25

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Exelon Generation Company, LLC (the licensee) dated February 27, 2004, as supplemented by letters dated October 11, 2004, and January 3, August 11, and September 12, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Facility Operating License No. DPR-25 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 208, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented prior to December 31, 2005.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by George Dick for/

Gene Y. Suh, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 26, 2005

ATTACHMENT TO LICENSE AMENDMENT NOS. 216 AND 208

FACILITY OPERATING LICENSE NOS. DPR-19 AND DPR-25

DOCKET NOS. 50-237 AND 50-249

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain a line in the margin indicating the area of change.

REMOVE

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5.6-3

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3.3.1.3-1

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EXELON GENERATION COMPANY, LLC

AND

MIDAMERICAN ENERGY COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 227
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated February 27, 2004, as supplemented by letters dated October 11, 2004, and January 3, August 11, and September 12, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-29 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 227 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented prior to December 31, 2005.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by George Dick for/

Gene Y. Suh, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 26, 2005

EXELON GENERATION COMPANY, LLC

AND

MIDAMERICAN ENERGY COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 222
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated February 27, 2004, as supplemented by letters dated October 11, 2004, and January 3, August 11, and September 12, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-30 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.222, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented prior to December 31, 2005.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by George Dick for/

Gene Y. Suh, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 26, 2005

ATTACHMENT TO LICENSE AMENDMENT NOS. 227 AND 222

FACILITY OPERATING LICENSE NOS. DPR-29 AND DPR-30

DOCKET NOS. 50-254 AND 50-265

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain a line in the margin indicating the area of change.

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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 216 TO FACILITY OPERATING LICENSE NO. DPR-19,
AMENDMENT NO. 208 TO FACILITY OPERATING LICENSE NO. DPR-25,
AMENDMENT NO. 227 TO FACILITY OPERATING LICENSE NO. DPR-29
AND AMENDMENT NO. 222 TO FACILITY OPERATING LICENSE NO. DPR-30
EXELON GENERATION COMPANY, LLC
AND
MIDAMERICAN ENERGY COMPANY
DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3, AND
QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2
DOCKET NOS. 50-237, 50-249, 50-254 AND 50-265

1.0 INTRODUCTION

By application dated February 27, 2004, as supplemented by letters dated October 11, 2004, and January 3, August 11, and September 12, 2005, the Exelon Generation Company (the licensee) requested changes to the Technical Specifications (TSs) for Dresden Nuclear Power Station, (DNPS) Units 2 and 3 and Quad Cities Nuclear Power Station, (QCNPS) Units 1 and 2. The supplements dated October 11, 2004, and January 3, August 11, and September 12, 2005, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on December 7, 2004 (69 FR 70718).

The proposed changes incorporate a new TS Section 3.3.1.3, "Oscillation Power Range Monitor (OPRM) Instrumentation," into the TSs and modify TS Section 5.6.5, "Core Operating Limits Report (COLR)," to add appropriate reference for the licensing basis methodology on long-term reactor stability solution. The proposed TS Section 3.3.1.3 is added to give the OPRM limiting conditions for operation, applicability, action statement, completion time for actions, and system surveillance requirements. TS 5.6.5 is revised to add NEDO-32465-A, "BWR Owners' Group Reactor Stability Detect and Suppress Solutions Licensing Basis Methodology for Reload Applications" (August 1996), as an U.S. Nuclear Regulatory Commission (NRC) approved analytical method to determine the OPRM setpoints.

2.0 REGULATORY EVALUATION

According to Title 10 of the *Code of Federal Regulation* (10 CFR) Part 50, Appendix A, General Design Criterion (GDC) 10, "Reactor Design," the reactor core and associated coolant, control, and protection systems must be designed with appropriate margins to assure that specified acceptable fuel design limits are not exceeded during any condition of normal operation, including the effects of anticipated operational occurrences. GDC 12, "Suppression of Reactor Power Oscillations," requires that the reactor core and associated coolant, control, and protection systems be designed to assure that power oscillations which can result in conditions exceeding specified acceptable fuel design limits are not possible or can be reliably and readily detected and suppressed. The OPRM system provides compliance with GDC 10 and GDC 12 by detecting the onset of oscillations and initiating a reactor scram to suppress the oscillations. This assures that the fuel design safety limit will not be violated for anticipated oscillations.

The OPRM instrumentation satisfies Criterion 3 of 10 CFR 50.36(c)(2)(ii) for Limiting conditions for operation for a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

The staff evaluation of the acceptability of the OPRM instrumentation is also based on 10 CFR 50.55a(h), "Protection and Safety Systems (IEEE Std. 279)."

3.0 TECHNICAL EVALUATION

3.1 Background

Generic Letter (GL) 94-02, "Long-Term Solutions and Upgrade of Interim Operating Recommendations for Thermal-Hydraulic Instabilities in Boiling Water Reactors," requested boiling water reactor (BWR) licensees to take appropriate measures to prevent thermal-hydraulic instabilities. In response to the NRC's concerns, General Electric Nuclear Energy issued the following licensing topical reports:

- A. NEDO-31960 and NEDO-31960, Supplement 1, "BWR Owners Group Long-Term Stability Solutions Licensing Methodology" (approved by the NRC in November 1995).
- B. NEDO-32465-A, "BWR Owners' Group Reactor Stability Detect and Suppress Solutions Licensing Basis Methodology for Reload Applications" (approved by the NRC in August 1996).

Asea Brown Boveri Combustion Engineering's (ABB-CE's) topical report CENPD-400-P, Rev.1, "Generic Topical Report for the ABB Option III Oscillation Power Range Monitor," was approved by the NRC in August 1995. The report specifically addresses BWR plants with the ABB Option III OPRM.

In response to GL 94-02, in September 1994, the licensee committed to implement the long-term solution designated as Option III in NEDO-31960-A (including Supplement 1) by installing the ABB-CE Option III OPRM system at DNPS and QCNPS. GL 94-02 also discussed the use of interim corrective actions (ICAs) to provide operator-controlled actions for avoiding regions of potential instability and to insert a manual reactor scram if oscillations are detected. The ABB system utilizes the OPRM detect-and-suppress function to implement Option III. The system uses existing local power range monitor (LPRM) signals for indicating the neutron flux

oscillations. The OPRM system initiates a trip whenever it detects an instability condition in the predefined region of the power-to-flow map.

The OPRM instrumentation modules, selected annunciators, and sequence of events recorder points were installed at DNPS and QCNPS from 1999 to 2000 and the scram signals from this instrumentation will be activated upon implementation of this TS amendment. The OPRM trip functions were not activated at the time of installation in order to allow evaluation of the performance of the OPRM algorithms without the risk of spurious scrams. In the interim, the licensee has continued to implement the ICAs to detect and suppress power oscillations until implementation of the OPRM system is incorporated into the plant TSs. The OPRMs will provide automatic "detect and suppress" action to replace the administrative controls under the ICAs currently in effect through operator training and manual actions.

3.2 Conformance With the Generic Topical Report

On August 16, 1995, the NRC staff completed the safety evaluation of the ABB-CE topical report CENPD-400-P, Revision 1. As stated in the staff's safety evaluation, a licensee referencing the topical report for implementation of the OPRM system must properly address each area of the design and implementation of the system in its TS submittal. The plant-specific actions are as follows:

- (1) Confirm the applicability of CENPD-400-P, including clarifications and reconciled difference between the plant-specific design and the topical report design descriptions.
- (2) Confirm the applicability of the BWR owners group (BWROG) topical reports on the OPRM and associated instability functions, setpoints, and margins.
- (3) Provide a plant-specific technical specification (TS) for the OPRM functions consistent with CENPD-400-P, Appendix A.
- (4) Confirm that the plant-specific environmental conditions (temperature, humidity, radiation, electromagnetic and seismic) are enveloped by the OPRM equipment environmental qualification profiles.
- (5) Confirm that administrative controls are provided for manually bypassing OPRM channels or protective functions and for controlling access to the OPRM functions.
- (6) Confirm that any changes to the plant operators' main control room panel have received human factors reviews per plant-specific procedures.

In Attachment 1 of their February 27, 2004 submittal, the licensee addressed the plant-specific conformance of the OPRM system design and implementation to the conditions specified above.

The staff compared the applicable design features with the corresponding design features in CENPD-400-P. The DNPS and QCNPS units are General Electric BWR/3s, a BWR design addressed in CENPD-400-P. The licensee stated that the BWROG topical reports NEDO-32465-A and NEDO-31960-A are applicable to DNPS and QCNPS.

In Attachments 2 and 5 of their February 27, 2004 submittal, the licensee provided a plant-specific technical specification for the OPRM functions consistent with CENPD-400-P, Appendix A, for DNPS and QCNPS, respectively.

The OPRM system and components are mounted in main control room cabinets at DNPS and QCNPS. The cabinets are located in a mild environmental zone. The OPRM components, including the replacement power supply, are qualified to perform their Class 1E safety function. The generic OPRM environmental qualification profiles envelop the DNPS and QCNPS temperature and radiation environmental conditions. With respect to relative humidity (RH), the licensee stated that the OPRM is designed to operate continuously in a humidity range of 30 percent to 95 percent relative humidity, noncondensing. The normal RH range for the DNPS and QCNPS control rooms is between 40 percent and 70 percent RH. However, the DNPS and QCNPS control rooms rely on air conditions to control RH. The RH may temporarily drop below 30 percent. The primary concern at low-humidity conditions is the possibility of damage from electrostatic discharge. The potential for electrostatic discharge is minimized because the modules are inside panels in metal enclosures. The OPRM equipment has been installed and has operated satisfactorily for several years. Thus the OPRM equipment should continue to operate properly if relative humidity is below 30 percent RH. The staff finds this justification acceptable.

Electromagnetic interference (EMI) testing of the OPRM equipment was done by ABB to ensure that the equipment would not be adversely affected by the plant EMI environment (susceptibility), and to ensure the OPRM modules would not be detrimental to the existing plant EMI environment (emissions). The licensee stated that post-maintenance testing of the system at DNPS and QCNPS has energized all portions of the OPRM circuits and has not resulted in any adverse effects on other systems.

The OPRM system is subjected to a minimum of five operating basis earthquakes in each axis followed by at least one safe-shutdown earthquake in each axis. The licensee stated that it has verified and documented that the DNPS and QCNPS control room response spectra are bounded by the test seismic response spectra in the generic design.

Based on the above discussion, the staff finds that the DNPS and QCNPS plant-specific environment conditions (temperature, humidity, radiation, electromagnetic, and seismic) are enveloped by the generic OPRM equipment environmental qualification profiles.

The OPRM has two modes of operation: operational and testing. In the operational mode, the system performs normal trip and alarm functions. The testing mode is used for testing, calibration, setpoint adjustment, and downloading of event data. In the testing mode, the OPRM's trip output is bypassed and the OPRM module is considered inoperable. In their October 11, 2004, response to the staff's request for information on system access control, the

licensee stated that the administrative procedures for access to the OPRM internal functions are as follows:

Procedures will be written in a manner that limits access to one OPRM channel at a time. The system engineer will require a password to access the menu that allows changing of setpoints and parameters; this password is not distributed to non-engineering personnel. All procedures will contain a requirement to obtain and document an Operations Senior Reactor Operator's permission before accessing OPRM functions.

The staff accepts this administrative control of access to setpoint adjustments, calibration, and test points.

The licensee stated that the changes to the main control room panels at DNPS and QCNPS for the OPRM system were evaluated by a human factors engineer in accordance with human factors engineering procedures for conformance to human engineering design principles.

Based on the above discussion, the staff concluded that the DNPS and QCNPS plant-specific OPRM systems conform with the Generic Topical Report CENPD-400-P-A.

3.3 Evaluation of Proposed Technical Specification Changes

- (1) Add TS Section 3.3.1.3, "Oscillation Power Range Monitor (OPRM) Instrumentation," to TS Section 3.3, "Instrumentation."

TS Section 3.3.1.3 requires the OPRM system to be operable when reactor power is 25 percent rated thermal power. The required minimum number of operable OPRM channels will be four channels.

The proposed changes are consistent with the generic TS in the NRC-approved topical report CENPD-400-P, Rev.1, "Generic Topical Report for the ABB Option III Oscillation Power Range Monitor," with the following exceptions.

- C The action note regarding average power range monitor (APRM) indication is not part of the CENPD-400-P TS changes.
- C The model TS in CENPD-400-P generic TS includes a surveillance requirement (SR) that requires calibration of the local power range monitors (LPRMs) every 1000 megawatt days per metric ton uranium (MWD/MTU). The proposed changes in this amendment request include an SR for calibration of the LPRM every 2000 effective full power hours, similar to SR 3.3.1.1.9 in the current DNPS and QCNPS TSs. The surveillance frequency is updated to include plant-specific information.
- C A clarification has been added to SR 3.3.1.3.3 to state that the setpoints for the trip function are specified in the core operating limits report (COLR).

The licensee provided the basis for these exceptions. The staff finds that these exceptions are for clarification purposes. The exceptions require no deviations from the generic technical specifications and are therefore acceptable.

The NRC staff also reviewed the proposed new TS 3.3.1.3 and found it acceptable because:

- (a) The OPRM systems provide an increase in the reliability of the protection of the margin of safety for the MCPR Safety Limit for any expected thermal-hydraulic instability transient and an operator burden is eased with the elimination of the current operating restriction; and
 - (b) The OPRM and associated instability functions, setpoints, and margins, closely follow the BWROG positions stated in NRC approved licensing Topical Reports NEDO-32465-A and NEDO-31960-A.
- (2) Modify TS Section 5.6.5, "Core Operating Limits Report (COLR)".

The licensee proposed to add: 1) a new TS 5.6.5.a.5, which is the OPRM setpoints for the trip function for SR 3.3.1.3.3; and 2) a new TS 5.6.5.b.15 for Dresden and TS 5.6.5.b.17 for Quad Cities, which is NEDO-32465-A, "BWR Owners' Group Reactor Stability Detect and Suppress Solutions Licensing Basis Methodology for Reload Applications," August 1996.

The staff has reviewed the proposed changes and found them acceptable because the proposed TS changes are administrative in nature and use NEDO-32465-A which is the NRC approved methodology for determining the setpoints for the OPRM trip function.

3.4 Summary

Based on the staff's review of submittals dated February 27, and October 11, 2004, and January 3, August 11, and September 12, 2005, the staff finds that the licensee's application for the OPRM implementation and the associated proposed TS changes are consistent with the staff safety evaluation approving CENPD-400-P and the appropriate guidance for design of digital instrumentation and control system modifications. The staff has concluded in the review of CENPD-400-P that the ABB-CE digital OPRM system functions and design meet the requirements of IEEE Std. 279-1971, 10 CFR 50.55(a)(h), and 10 CFR 50, Appendix B, for digital reactor protection system design. The staff also concludes that the proposed TS changes involving the implementation of the BWROG long-term stability solution Option III, using ABB/CE OPRM instrumentation for DNPS and QCNPS are acceptable because the proposed TS changes are in accordance with the approved methodology specified in NEDO-32465-A. Furthermore, the OPRM system functions meet the requirements of GDC 12 and acceptably address the related requirements of GDC 10 for ensuring reactor safety in the event of power instabilities. The proposed TS is in conformance with 10 CFR 50.36(c)(2)(ii). Therefore, the proposed changes are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (69 FR 70718). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: H. Li
T. Huang

Date: September 26, 2005

Dresden and Quad Cities Nuclear Power Stations

cc:

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Dresden and Quad Cities Nuclear Power Stations

- 2 -

cc:

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