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**10 CFR 50.59**

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102-05194-SAB/TNW/CJJ  
December 21, 2004

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Mail Station P1-37  
Washington, DC 20555-0001

Dear Sirs:

**Subject: Palo Verde Nuclear Generating Station (PVNGS)  
Units 1, 2, & 3  
Docket Nos. STN 50-528/529/530  
10 CFR 50.59 Report (January – December 2003)**

Pursuant to 10 CFR 50.59(d)(2), Arizona Public Service Company is submitting the enclosed report. This report contains a brief description of each change and a brief summary of the evaluation required by 10 CFR 50.59(d)(1) for each change. This report contains all evaluations written during 2003, regardless of the implementation status of the evaluated action.

There were no NRC Commitment changes during 2003.

No commitments are being made to the NRC by this letter. Should you have any questions, please contact Thomas N. Weber at (623) 393-5764.

Sincerely,

SAB/TNW/CJJ/kg

Enclosure

cc: B. S. Mallett (all w/enclosure)  
M. B. Fields  
N. L. Salgado

IE47

**ENCLOSURE**

**PALO VERDE NUCLEAR GENERATING STATION**

**ACRONYM/ABBREVIATION LIST**

**AND**

**10 CFR 50.59 REPORT**

**JANUARY – DECEMBER 2003**

## ACRONYM/ABBREVIATION LIST

AC	-	Alternating Current
AFW	-	Auxiliary Feedwater System
CENTS	-	Combustion Engineering Nuclear Transient Simulation
DMWO	-	Design Modification Work Order
GTGs	-	Gas Turbine Generators
MSIS	-	Main Steam Isolation Signal
OSG	-	Original Steam Generator
PROC	-	Procedure
PWSCC	-	Primary Water Stress Corrosion Cracking
RSG	-	Replacement Steam Generator
SG	-	Steam Generator
SGRP	-	Steam Generator Replacement Project
TMOD	-	Temporary Modification
TS	-	Technical Specifications
TSP	-	Tri-Sodium Phosphate

## 10 CFR 50.59 Annual Report (January - December 2003)

Log	Doc Type	Doc Number	Description	Summary
E-03-00001	LDCR	03-B003	This revision to TS Bases 3.5.6 revised text describing surveillance test methods used to demonstrate the operability of Tri-Sodium Phosphate (TSP) bins installed for post-accident sump pH control to reflect changes in the supporting analysis.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-03-00002	LDCR	03-F004	This revision to the UFSAR was the result of Revision 2 to Calculation 13-MC-SI-0016, which sizes the Tri-Sodium Phosphate (TSP) bins for post-accident sump pH control.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-03-00003	LDCR	01-F053	This revision to the UFSAR allows periodic load testing of GTGs to be performed while a Unit is in any plant mode. This is a change from information that was originally provided to the NRC describing the use of the GTGs as an Alternate AC source in compliance with 10 CFR 50.63, Station Blackout Rule. This change was initiated to allow APS to better manage plant risks in accordance with 10 CFR 50.65(a)(4), Maintenance Rule	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-03-00004	PROC	31MT-9ZC07	This revision to procedure 31MT-9ZC07, Steam Generator Replacement Project Main Steam Pipe Spool Lifts allows the steam generator main steam pipe spools inside containment to be moved concurrently with movement of fuel between the reactor pressure vessel and the spent fuel pool. This 50.59 evaluated the use of a fault tree analysis to demonstrate the revised administrative controls during a specific heavy load movement is within the PVNGS original commitment to compliance with NUREG-0612.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-03-00005	DMWO	2513813	This modification to the pressurizer replaces a portion of each heater sleeve with new heater sleeve and weld material to mitigate the affects of PWSCC. This modification also replaces damaged heaters (like-for-like), as required.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-03-00006	PROC	40TI-9ZZ04	This procedure 40TI-9ZZ04, SGRP, AFW Waterhammer Flow Test is new. The objectives of the procedure are to verify that the revamped auxiliary feedwater piping system and the replacement steam generator (RSG) feed ring are not damaged by full flow injection of auxiliary feedwater.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).

Log	Doc Type	Doc Number	Description	Summary
E-03-00009	DMWO	221240	This modification to the Unit 2 SGs replaced OSG steam generator level transmitters and calibrated the new ones to the RSG requirements. In addition, this includes a revision to the UFSAR to update response times for MSIS on high SG level in Mode 3 only.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-03-00010	LDCR	03-F054	This revision to the UFSAR reflects the new CENTS analysis for 3876 MWt and for the power uprate analysis completed for Unit 2.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-03-00011	T-Mod	2659140	This T-Mod provides an alarm in the Control Room to alert operators of declining pressure in the annulus space between the inner and outer o-ring seals of the Unit 2 reactor vessel.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).