



SOUTHERN CALIFORNIA
EDISON[®]

An EDISON INTERNATIONAL[®] Company

Dwight E. Nunn
Vice President

December 10, 2004

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

**SUBJECT: SAN ONOFRE NUCLEAR GENERATING STATION, UNITS 2 AND 3
DOCKET NOs. 50-361 and 50-362
PROPOSED CHANGE NUMBER (PCN) 550
APPLICATION FOR TECHNICAL SPECIFICATION IMPROVEMENT TO
ELIMINATE REQUIREMENTS FOR HYDROGEN MONITORS USING
THE CONSOLIDATED LINE ITEM IMPROVEMENT PROCESS**

Dear Sir or Madam:

Pursuant to 10 CFR 50.90, Southern California Edison (SCE) hereby requests an amendment to the Technical Specifications (TS) for the San Onofre Nuclear Generating Station, Units 2 and 3.

The proposed amendment will delete the TS requirements related to hydrogen monitors. The proposed TS change supports implementation of the revisions to 10 CFR 50.44, "Standards for Combustible Gas Control System in Light-Water-Cooled Power Reactors," that became effective on October 16, 2003. The changes are consistent with Revision 1 of NRC-approved Industry/Technical Specification Task Force (TSTF) Standard Technical Specification Change Traveler, TSTF-447, "Elimination of Hydrogen Recombiners and Change to Hydrogen and Oxygen Monitors." The availability of this TS improvement was announced in the *Federal Register* on September 25, 2003 as part of the consolidated line item improvement process (CLIP).

Attachment 1 provides a description of the proposed change, the requested confirmation of applicability, and plant-specific verifications and commitments. Attachments 2 and 4 provide the existing TS pages marked-up to show the proposed change. Attachments 3 and 5 provide revised, clean TS pages. Implementation of TSTF-447 also involves various changes to the TS Bases. The TS Bases changes will be submitted with a future update in accordance with TS 5.4.4, "Technical Specifications (TS) Bases Control."

SCE requests approval of the proposed License Amendment by January 3, 2006, with the amendment being implemented within 60 days of approval of this amendment request.

P.O. Box 128
San Clemente, CA 92674-0128
949-368-1480
Fax 949-368-1490

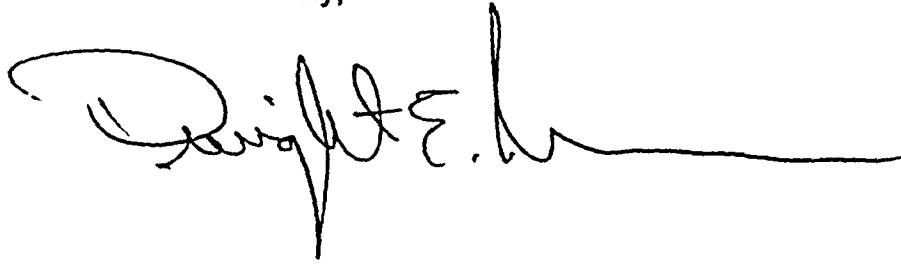
A001

December 10, 2004

In accordance with 10 CFR 50.91, a copy of this application, with attachments, is being provided to the designated California State Official.

If you should have any questions regarding this submittal, please contact Mr. Jack Rainsberry at 949-368-7420.

Sincerely,

A handwritten signature in black ink, appearing to read "Dwight E. H.", followed by a long horizontal line.

Enclosures:

1. Notarized affidavits
2. Licensee's evaluation of the proposed changes

Attachments:

1. Description and Assessment
2. Proposed Technical Specification Changes, Unit 2
3. Revised Technical Specification Pages, Unit 2
4. Proposed Technical Specification Changes, Unit 3
5. Revised Technical Specification Pages, Unit 3

cc: B. S. Mallett, Regional Administrator, NRC Region IV,
B. M. Pham, NRC Project Manager, San Onofre Units 2 and 3
C. C. Osterholtz, NRC Senior Resident Inspector, San Onofre Units 2 and 3
S. Y. Hsu, Department of Health Services, Radiologic Health Branch

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION


| | | |
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| Application of SOUTHERN CALIFORNIA |) | |
| EDISON COMPANY, ET AL. for a Class 103 |) | Docket No. 50-361 |
| License to Acquire, Possess, and Use |) | |
| A Utilization Facility as Part of |) | Amendment Application No. 228 |
| Unit No. 2 of the San Onofre Nuclear |) | |
| Generating Station |) | |

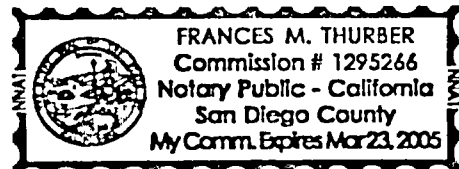
SOUTHERN CALIFORNIA EDISON COMPANY, et al. pursuant to 10CFR50.90, hereby submit Amendment Application No. 228. This amendment application consists of Proposed Change Number (PCN) 550 to Facility Operating License NPF-10. PCN-550 is a request to delete the requirements related to hydrogen monitors in Technical Specification (TS) 3.3.11, "Post Accident Monitoring Instrumentation," for San Onofre Nuclear Generating Station Unit 2. The availability of this TS improvement was announced by the Nuclear Regulatory Commission in the *Federal Register* on September 25, 2003 as part of the consolidated line item improvement process (CLIP).

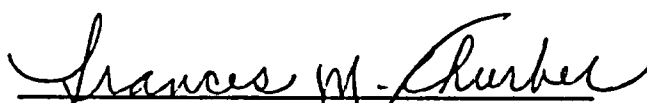
State of California
County of San Diego

Subscribed and sworn to (or affirmed) before me this 10th day of

Dec., 2004

By: 
Dwight E. Nunn
Vice President




Notary Public

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

| | | |
|--|---|-------------------------------|
| Application of SOUTHERN CALIFORNIA |) | |
| EDISON COMPANY, ET AL. for a Class 103 |) | Docket No. 50-362 |
| License to Acquire, Possess, and Use |) | |
| A Utilization Facility as Part of |) | Amendment Application No. 212 |
| Unit No. 3 of the San Onofre Nuclear |) | |
| Generating Station |) | |

SOUTHERN CALIFORNIA EDISON COMPANY, et al. pursuant to 10CFR50.90, hereby submit Amendment Application No. 212. This amendment application consists of Proposed Change Number (PCN) 550 to Facility Operating License NPF-15. PCN-550 is a request to delete the requirements related to hydrogen monitors in Technical Specification (TS) 3.3.11, "Post Accident Monitoring Instrumentation," for San Onofre Nuclear Generating Station Unit 3. The availability of this TS improvement was announced by the Nuclear Regulatory Commission in the *Federal Register* on September 25, 2003 as part of the consolidated line item improvement process (CLIP).

State of California
County of San Diego

Subscribed and sworn to (or affirmed) before me this 10th day of
Dec., 2004

By: 

Dwight E. Nunn
Vice President


Notary Public



Enclosure 2
ATTACHMENT 1
DESCRIPTION AND ASSESSMENT

1.0 INTRODUCTION

The proposed License amendment deletes Technical Specification references to the hydrogen monitors in Technical Specification (TS) 3.3.11, "Post Accident Monitoring Instrumentation (PAMI)." The proposed TS changes support implementation of the revisions to 10 CFR 50.44, "Standards for Combustible Gas Control System in Light-Water-Cooled Power Reactors," that became effective on October 16, 2003. The deletion of references to hydrogen monitors resulted in numbering and formatting changes to other TS, which were otherwise unaffected by this proposed amendment.

The changes are consistent with Revision 1 of NRC-approved Industry/Technical Specification Task Force (TSTF) Standard Technical Specification Change Traveler, TSTF-447, "Elimination of Hydrogen Recombiners and Change to Hydrogen and Oxygen Monitors." The availability of this TS improvement was announced in the *Federal Register* on September 25, 2003 as part of the consolidated line item improvement process (CLIIP).

2.0 DESCRIPTION OF PROPOSED AMENDMENT

Consistent with the NRC-approved Revision 1 of TSTF-447, the proposed TS changes include:

| | | |
|------------------------|--|---------|
| TS 3.3.11, Condition D | Inoperable Hydrogen Monitors | Deleted |
| Table 3.3.11-1 | Item 10, Containment Hydrogen Monitors | Deleted |

Other TS changes included in this application are limited to renumbering and formatting changes that resulted directly from the deletion of the above requirements related to hydrogen monitors.

As described in NRC-approved Revision 1 of TSTF-447, the changes to TS requirements result in changes to various TS Bases sections. The TS Bases changes will be submitted with a future update in accordance with TS 5.4.4, "Technical Specifications (TS) Bases Control."

3.0 BACKGROUND

The background for this application is adequately addressed by the NRC Notice of Availability published on September 25, 2003 (68 FR 55416), TSTF-447, the documentation associated with the 10 CFR 50.44 rulemaking, and other related documents.

4.0 REGULATORY REQUIREMENTS AND GUIDANCE

The applicable regulatory requirements and guidance associated with this application are adequately addressed by the NRC Notice of Availability published on September 25, 2003 (68 FR 55416), TSTF-447, the documentation associated with the 10 CFR 50.44 rulemaking, and other related documents.

5.0 TECHNICAL ANALYSIS

Southern California Edison (SCE) has reviewed the safety evaluation (SE) published on September 25, 2003 (68 FR 55416), as part of the CLIIP Notice of Availability. This verification included a review of the NRC staff's SE, as well as the supporting information provided to support TSTF-447. SCE has concluded that the justifications presented in the TSTF proposal and the SE prepared by the NRC staff are applicable to San Onofre Nuclear Generating Station, Units 2 and 3 (SONGS 2 & 3), and justify this amendment for the incorporation of the changes to the SONGS 2 & 3 TS.

6.0 REGULATORY ANALYSIS

A description of this proposed change and its relationship to applicable regulatory requirements and guidance was provided in the NRC Notice of Availability published on September 25, 2003 (68 FR 55416), TSTF-447, the documentation associated with the 10 CFR 50.44 rulemaking, and other related documents.

6.1 Verification and Commitments

As discussed in the model SE published in the *Federal Register* on September 25, 2003 (68 FR 55416), for this TS improvement, SCE is making the following verifications and regulatory commitments:

1. SCE has verified that a hydrogen monitoring system capable of diagnosing beyond design-basis accidents is installed at SONGS 2 & 3 and is making a regulatory commitment to maintain that capability. The hydrogen monitors will be included in the Licensee Controlled Specifications. This regulatory commitment will be implemented within 60 days of approval of this amendment request.
2. SONGS 2 & 3 do not have inerted containments.

7.0 NO SIGNIFICANT HAZARDS CONSIDERATION

Southern California Edison (SCE) has reviewed the proposed no significant hazards consideration determination published on September 25, 2003 (68 FR 55416) as part of the CLIIP. SCE has concluded that the proposed determination presented in the notice is applicable to San Onofre Nuclear Generating Station Units 2 and 3, and the determination is hereby incorporated by reference to satisfy the requirements of 10 CFR 50.91(a).

8.0 ENVIRONMENTAL EVALUATION

Southern California Edison (SCE) has reviewed the environmental evaluation included in the model SE published on September 25, 2003 (68 FR 55416) as part of the CLIIP. SCE has concluded that the staff's findings presented in that evaluation are applicable to San Onofre Nuclear Generating Station Units 2 and 3, and the evaluation is hereby incorporated by reference for this application.

9.0 PRECEDENT

This application is being made in accordance with the CLIP. With the exception noted below, Southern California Edison (SCE) is not proposing variations or deviations from the TS changes described in TSTF-447 or the NRC staff's model SE published on September 25, 2003 (68 FR 55416).

EXCEPTION:

SCE is proposing not to renumber the function list in Table 3.3.11-1. SCE is making this exception in order to reduce the possibility of errors arising where references to the functions are made by number.

10.0 REFERENCES

Federal Register Notice: Notice of Availability of Model Application Concerning Technical Specification Improvement To Eliminate Hydrogen Recombiner Requirement, and Relax the Hydrogen and Oxygen Monitor Requirements for Light Water Reactors Using the Consolidated Line Item Improvement Process, published September 25, 2003 (68 FR 55416).

Enclosure 2
ATTACHMENT 2
PROPOSED TECHNICAL SPECIFICATION CHANGES, UNIT 2 (MARK-UP)

ACTIONS (continued)

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|--|--|---------------------|
| <p>C. NOTE Not applicable to hydrogen monitor channels.</p> <p>One or more Functions with two required channels inoperable.</p> | C.1 Restore one channel to OPERABLE status. | 7 days |
| D. Two hydrogen monitor channels inoperable. | D.1 Restore one hydrogen monitor channel to OPERABLE status. | 72 hours |
| ED. Required channel of Functions 18, 21, 24, or 25 inoperable. | ED.1 Restore required channel to OPERABLE status. | 7 days |
| FE. Required Action and associated Completion Time of Condition C, D or ED not met. | FE.1 Enter the Condition referenced in Table 3.3.11-1 for the channel. | Immediately |
| GF. As required by Required Action FE.1 and referenced in Table 3.3.11-1. | GF.1 Be in MODE 3. <u>AND</u> GF.2 Be in MODE 4. | 6 hours 12 hours |
| HG. As required by Required Action FE.1 and referenced in Table 3.3.11-1. | HG.1 Prepare and submit a Special Report to the NRC in accordance with Specification 5.7.2. | 30 days |

SURVEILLANCE REQUIREMENTS

-----NOTE-----
These SRs apply to each PAMI Function in Table 3.3:11-1, with exceptions noted.

| SURVEILLANCE | | FREQUENCY |
|--------------|---|-----------|
| SR 3.3.11.1 | Perform CHANNEL CHECK for Function 9. | 12 hours |
| SR 3.3.11.2 | Perform CHANNEL CHECK for each required instrumentation channel, except Function 9, that is normally energized. | 31 days |
| SR 3.3.11.3 | Perform CHANNEL FUNCTIONAL TEST for function 9. | 31 days |
| SR 3.3.11.4 | Perform CHANNEL CALIBRATION, for functions 2,3,14,15,16,17, and 20. | 18 months |
| SR 3.3.11.5 | Perform CHANNEL CALIBRATION for functions 1,4,5,6,7,8,9,10,11,12,13,18, 19,21,22,23,24,25,26, and 27. | 24 months |

Table 3.3.11-1 (page 1 of 1)
Post Accident Monitoring Instrumentation

| FUNCTION | REQUIRED CHANNELS | CONDITIONS REFERENCED FROM REQUIRED ACTION F.1 |
|--|-----------------------------------|---|
| 1. Excore Neutron Flux | 2 | GF |
| 2. Reactor Coolant System Hot Leg Temperature | 2 (1 per steam generator) | GF |
| 3. Reactor Coolant System Cold Leg Temperature | 2 (1 per steam generator) | GF |
| 4. Reactor Coolant System Pressure (wide range) | 2 | GF |
| 5. Reactor Vessel Water Level | 2(d) | HG |
| 6. Containment Water Level (wide range) | 2 | GF |
| 7. Containment Pressure (wide range) | 2 | GF |
| 8. Containment Isolation Valve Position | 2 per penetration flow path(a)(b) | GF |
| 9. Containment Area Radiation (high range) | 2 | HG |
| 10. Deleted Containment Hydrogen Monitors | 2 | G |
| 11. Pressurizer Level | 2 | GF |
| 12. Steam Generator Water Level (wide range) | 2 per steam generator | GF |
| 13. Condensate Storage Tank Level | 2 | GF |
| 14. Core Exit Temperature - Quadrant 1 | 2(c) | GF |
| 15. Core Exit Temperature - Quadrant 2 | 2(c) | GF |
| 16. Core Exit Temperature - Quadrant 3 | 2(c) | GF |
| 17. Core Exit Temperature - Quadrant 4 | 2(c) | GF |
| 18. Auxiliary Feedwater Flow | 1 per steam generator | GF |
| 19. Containment Pressure (narrow range) | 2 | GF |
| 20. Reactor Coolant System Subcooling Margin Monitor | 2 | GF |
| 21. Pressurizer Safety Valve Position Indication | 1 per valve | GF |
| 22. Containment Temperature | 2 | GF |
| 23. Containment Water Level (narrow range) | 2 | GF |
| 24. HPSI Flow Cold Leg | 1 per cold leg | GF |
| 25. HPSI Flow Hot Leg | 1 per hot leg | GF |
| 26. Steam Line Pressure | 2 per steam generator | GF |
| 27. Refueling Water Storage Tank Level | 2 | GF |

(a) Not required for isolation valves whose associated penetration is isolated by at least one closed and de-activated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

(b) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

(c) A channel consists of two or more core exit thermocouples.

(d) A channel consists of eight sensors in a probe. A channel is OPERABLE if four or more sensors, one sensor in the upper head and three sensors in the lower head are OPERABLE.

5.7 Reporting Requirements (continued)

5.7.2 Special Reports

Special Reports may be required covering inspection, test, and maintenance activities. These special reports are determined on an individual basis for each unit and their preparation and submittal are designated in the Technical Specifications.

Special Reports shall be submitted to the U. S. Nuclear Regulatory Commission, Attention: Document Control Desk, Washington, D. C. 20555, with a copy to the Regional Administrator of the Regional Office of the NRC, in accordance with 10 CFR 50.4 within the time period specified for each report.

The following Special Reports shall be submitted:

- a. When a pre-planned alternate method of monitoring post-accident instrumentation functions is required by Condition B or Condition HG of LCO 3.3.11, a report shall be submitted within 30 days from the time the action is required. The report shall outline the action taken, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the function to OPERABLE status.
- b. Any abnormal degradation of the containment structure detected during the tests required by the Pre-Stressed Concrete Containment Tendon Surveillance Program shall be reported to the NRC within 30 days. The report shall include a description of the tendon condition, the condition of the concrete (especially at tendon anchorages), the inspection procedures, the tolerances on cracking, and the corrective action taken.
- c. Following each inservice inspection of steam generator (SG) tubes, in accordance with the SG Tube Surveillance Program, the number of tubes plugged and tubes sleeved in each SG shall be reported to the NRC within 15 days. The complete results of the SG tube inservice inspection shall be submitted to the NRC within 12 months following the completion of the inspection. The report shall include:
 1. Number and extent of tubes and sleeves inspected, and
 2. Location and percent of wall-thickness penetration for each indication of an imperfection, and
 3. Identification of tubes plugged and tubes sleeved.

(continued)

Enclosure 2
ATTACHMENT 3
PROPOSED TECHNICAL SPECIFICATION PAGES, UNIT 2

ACTIONS (continued)

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|--|--|-----------------|
| C. One or more Functions with two required channels inoperable. | C.1 Restore one channel to OPERABLE status. | 7 days |
| D. Required channel of Functions 18, 21, 24, or 25 inoperable. | D.1 Restore required channel to OPERABLE status. | 7 days |
| E. Required Action and associated Completion Time of Condition C or D not met. | E.1 Enter the Condition referenced in Table 3.3.11-1 for the channel. | Immediately |
| F. As required by Required Action E.1 and referenced in Table 3.3.11-1. | F.1 Be in MODE 3. | 6 hours |
| | <u>AND</u> F.2 Be in MODE 4. | 12 hours |
| G. As required by Required Action E.1 and referenced in Table 3.3.11-1. | G.1 Prepare and submit a Special Report to the NRC in accordance with Specification 5.7.2. | 30 days |

SURVEILLANCE REQUIREMENTS

-----NOTE-----
These SRs apply to each PAMI Function in Table 3.3.11-1, with exceptions noted.

| SURVEILLANCE | | FREQUENCY |
|--------------|---|-----------|
| SR 3.3.11.1 | Perform CHANNEL CHECK for Function 9. | 12 hours |
| SR 3.3.11.2 | Perform CHANNEL CHECK for each required instrumentation channel, except Function 9, that is normally energized. | 31 days |
| SR 3.3.11.3 | Perform CHANNEL FUNCTIONAL TEST for function 9. | 31 days |
| SR 3.3.11.4 | Perform CHANNEL CALIBRATION, for functions 2,3,14,15,16,17, and 20. | 18 months |
| SR 3.3.11.5 | Perform CHANNEL CALIBRATION for functions 1,4,5,6,7,8,9,11,12,13,18, 19,21,22,23,24,25,26, and 27. | 24 months |

Table 3.3.11-1 (page 1 of 1)
Post Accident Monitoring Instrumentation

| FUNCTION | REQUIRED CHANNELS | CONDITIONS REFERENCED FROM REQUIRED ACTION F.1 |
|--|---|---|
| 1. Excore Neutron Flux | 2 | F |
| 2. Reactor Coolant System Hot Leg Temperature | 2 (1 per steam generator) | F |
| 3. Reactor Coolant System Cold Leg Temperature | 2 (1 per steam generator) | F |
| 4. Reactor Coolant System Pressure (wide range) | 2 | F |
| 5. Reactor Vessel Water Level | 2 ^(d) | G |
| 6. Containment Water Level (wide range) | 2 | F |
| 7. Containment Pressure (wide range) | 2 | F |
| 8. Containment Isolation Valve Position | 2 per penetration flow path ^(a) ^(b) | F |
| 9. Containment Area Radiation (high range) | 2 | G |
| 10. Deleted | | |
| 11. Pressurizer Level | 2 | F |
| 12. Steam Generator Water Level (wide range) | 2 per steam generator | F |
| 13. Condensate Storage Tank Level | 2 | F |
| 14. Core Exit Temperature – Quadrant 1 | 2 ^(c) | F |
| 15. Core Exit Temperature – Quadrant 2 | 2 ^(c) | F |
| 16. Core Exit Temperature – Quadrant 3 | 2 ^(c) | F |
| 17. Core Exit Temperature – Quadrant 4 | 2 ^(c) | F |
| 18. Auxiliary Feedwater Flow | 1 per steam generator | F |
| 19. Containment Pressure (narrow range) | 2 | F |
| 20. Reactor Coolant System Subcooling Margin Monitor | 2 | F |
| 21. Pressurizer Safety Valve Position Indication | 1 per valve | F |
| 22. Containment Temperature | 2 | F |
| 23. Containment Water Level (narrow range) | 2 | F |
| 24. HPSI Flow Cold Leg | 1 per cold leg | F |
| 25. HPSI Flow Hot Leg | 1 per hot leg | F |
| 26. Steam Line Pressure | 2 per steam generator | F |
| 27. Refueling Water Storage Tank Level | 2 | F |

(a) Not required for isolation valves whose associated penetration is isolated by at least one closed and de-activated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

(b) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

(c) A channel consists of two or more core exit thermocouples.

(d) A channel consists of eight sensors in a probe. A channel is OPERABLE if four or more sensors, one

5.7 Reporting Requirements (continued)

5.7.2 Special Reports

Special Reports may be required covering inspection, test, and maintenance activities. These special reports are determined on an individual basis for each unit and their preparation and submittal are designated in the Technical Specifications.

Special Reports shall be submitted to the U. S. Nuclear Regulatory Commission, Attention: Document Control Desk, Washington, D. C. 20555, with a copy to the Regional Administrator of the Regional Office of the NRC, in accordance with 10 CFR 50.4 within the time period specified for each report.

The following Special Reports shall be submitted:

- a. When a pre-planned alternate method of monitoring post-accident instrumentation functions is required by Condition B or Condition G of LCO 3.3.11, a report shall be submitted within 30 days from the time the action is required. The report shall outline the action taken, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the function to OPERABLE status.
- b. Any abnormal degradation of the containment structure detected during the tests required by the Pre-Stressed Concrete Containment Tendon Surveillance Program shall be reported to the NRC within 30 days. The report shall include a description of the tendon condition, the condition of the concrete (especially at tendon anchorages), the inspection procedures, the tolerances on cracking, and the corrective action taken.
- c. Following each inservice inspection of steam generator (SG) tubes, in accordance with the SG Tube Surveillance Program, the number of tubes plugged and tubes sleeved in each SG shall be reported to the NRC within 15 days. The complete results of the SG tube inservice inspection shall be submitted to the NRC within 12 months following the completion of the inspection. The report shall include:
 1. Number and extent of tubes and sleeves inspected, and
 2. Location and percent of wall-thickness penetration for each indication of an imperfection, and
 3. Identification of tubes plugged and tubes sleeved.

(continued)

Enclosure 2
ATTACHMENT 4
PROPOSED TECHNICAL SPECIFICATION CHANGES, UNIT 3 (MARK-UP)

ACTIONS (continued)

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|---|---|--------------------------------|
| <p>C. NOTE Not applicable to hydrogen monitor channels.</p> <p>One or more Functions with two required channels inoperable.</p> | C.1 Restore one channel to OPERABLE status. | 7 days |
| D. Two hydrogen monitor channels inoperable. | D.1 Restore one hydrogen monitor channel to OPERABLE status. | 72 hours |
| ED. Required channel of Functions 18, 21, 24, or 25 inoperable. | ED.1 Restore required channel to OPERABLE status. | 7 days |
| FE. Required Action and associated Completion Time of Condition C, D or ED not met. | FE.1 Enter the Condition referenced in Table 3.3.11-1 for the channel. | Immediately |
| GE. As required by Required Action FE.1 and referenced in Table 3.3.11-1. | <p>GE.1 Be in MODE 3.</p> <p>AND</p> <p>GE.2 Be in MODE 4.</p> | <p>6 hours</p> <p>12 hours</p> |
| HG. As required by Required Action FE.1 and referenced in Table 3.3.11-1. | HG.1 Prepare and submit a Special Report to the NRC in accordance with Specification 5.7.2. | 30 days |

SURVEILLANCE REQUIREMENTS

-----NOTE-----
These SRs apply to each PAMI Function in Table 3.3.11-1, with exceptions noted.

| SURVEILLANCE | | FREQUENCY |
|--------------|---|-----------|
| SR 3.3.11.1 | Perform CHANNEL CHECK for Function 9. | 12 hours |
| SR 3.3.11.2 | Perform CHANNEL CHECK for each required instrumentation channel, except Function 9, that is normally energized. | 31 days |
| SR 3.3.11.3 | Perform CHANNEL FUNCTIONAL TEST for function 9. | 31 days |
| SR 3.3.11.4 | Perform CHANNEL CALIBRATION, for functions 2,3,14,15,16,17, and 20. | 18 months |
| SR 3.3.11.5 | Perform CHANNEL CALIBRATION for functions 1,4,5,6,7,8,9,10,11,12,13,18, 19,21,22,23,24,25,26, and 27. | 24 months |

Table 3.3.11-1 (page 1 of 1)
Post Accident Monitoring Instrumentation

| FUNCTION | REQUIRED CHANNELS | CONDITIONS REFERENCED FROM REQUIRED ACTION F.1 |
|--|-----------------------------------|---|
| 1. Excore Neutron Flux | 2 | GF |
| 2. Reactor Coolant System Hot Leg Temperature | 2 (1 per steam generator) | GF |
| 3. Reactor Coolant System Cold Leg Temperature | 2 (1 per steam generator) | GF |
| 4. Reactor Coolant System Pressure (wide range) | 2 | GF |
| 5. Reactor Vessel Water Level | 2(d) | HG |
| 6. Containment Water Level (wide range) | 2 | GF |
| 7. Containment Pressure (wide range) | 2 | GF |
| 8. Containment Isolation Valve Position | 2 per penetration flow path(a)(b) | GF |
| 9. Containment Area Radiation (high range) | 2 | HG |
| 10. Deleted Containment Hydrogen Monitors | 2 | G |
| 11. Pressurizer Level | 2 | GF |
| 12. Steam Generator Water Level (wide range) | 2 per steam generator | GF |
| 13. Condensate Storage Tank Level | 2 | GF |
| 14. Core Exit Temperature - Quadrant 1 | 2(c) | GF |
| 15. Core Exit Temperature - Quadrant 2 | 2(c) | GF |
| 16. Core Exit Temperature - Quadrant 3 | 2(c) | GF |
| 17. Core Exit Temperature - Quadrant 4 | 2(c) | GF |
| 18. Auxiliary Feedwater Flow | 1 per steam generator | GF |
| 19. Containment Pressure (narrow range) | 2 | GF |
| 20. Reactor Coolant System Subcooling Margin Monitor | 2 | GF |
| 21. Pressurizer Safety Valve Position Indication | 1 per valve | GF |
| 22. Containment Temperature | 2 | GF |
| 23. Containment Water Level (narrow range) | 2 | GF |
| 24. HPSI Flow Cold Leg | 1 per cold leg | GF |
| 25. HPSI Flow Hot Leg | 1 per hot leg | GF |
| 26. Steam Line Pressure | 2 per steam generator | GF |
| 27. Refueling Water Storage Tank Level | 2 | GF |

(a) Not required for isolation valves whose associated penetration is isolated by at least one closed and de-activated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

(b) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

(c) A channel consists of two or more core exit thermocouples.

5.7 Reporting Requirements (continued)

5.7.2 Special Reports

Special Reports may be required covering inspection, test, and maintenance activities. These special reports are determined on an individual basis for each unit and their preparation and submittal are designated in the Technical Specifications.

Special Reports shall be submitted to the U. S. Nuclear Regulatory Commission, Attention: Document Control Desk, Washington, D. C. 20555, with a copy to the Regional Administrator of the Regional Office of the NRC, in accordance with 10 CFR 50.4 within the time period specified for each report.

The following Special Reports shall be submitted:

- a. When a pre-planned alternate method of monitoring post-accident instrumentation functions is required by Condition B or Condition HG of LCO 3.3.11, a report shall be submitted within 30 days from the time the action is required. The report shall outline the action taken, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the function to OPERABLE status.
- b. Any abnormal degradation of the containment structure detected during the tests required by the Pre-Stressed Concrete Containment Tendon Surveillance Program shall be reported to the NRC within 30 days. The report shall include a description of the tendon condition, the condition of the concrete (especially at tendon anchorages), the inspection procedures, the tolerances on cracking, and the corrective action taken.
- c. Following each inservice inspection of steam generator (SG) tubes, in accordance with the SG Tube Surveillance Program, the number of tubes plugged and tubes sleeved in each SG shall be reported to the NRC within 15 days. The complete results of the SG tube inservice inspection shall be submitted to the NRC within 12 months following the completion of the inspection. The report shall include:
 1. Number and extent of tubes and sleeves inspected, and
 2. Location and percent of wall-thickness penetration for each indication of an imperfection, and
 3. Identification of tubes plugged and tubes sleeved.

(continued)

Enclosure 2
ATTACHMENT 5
PROPOSED TECHNICAL SPECIFICATION PAGES, UNIT 3

ACTIONS (continued)

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|--|--|-----------------|
| C. One or more Functions with two required channels inoperable. | C.1 Restore one channel to OPERABLE status. | 7 days |
| D. Required channel of Functions 18, 21, 24, or 25 inoperable. | D.1 Restore required channel to OPERABLE status. | 7 days |
| E. Required Action and associated Completion Time of Condition C or D not met. | E.1 Enter the Condition referenced in Table 3.3.11-1 for the channel. | Immediately |
| F. As required by Required Action E.1 and referenced in Table 3.3.11-1. | F.1 Be in MODE 3. | 6 hours |
| | <u>AND</u> F.2 Be in MODE 4. | 12 hours |
| G. As required by Required Action E.1 and referenced in Table 3.3.11-1. | G.1 Prepare and submit a Special Report to the NRC in accordance with Specification 5.7.2. | 30 days |

SURVEILLANCE REQUIREMENTS

-----NOTE-----
These SRs apply to each PAMI Function in Table 3.3.11-1, with exceptions noted.

| SURVEILLANCE | | FREQUENCY |
|--------------|---|-----------|
| SR 3.3.11.1 | Perform CHANNEL CHECK for Function 9. | 12 hours |
| SR 3.3.11.2 | Perform CHANNEL CHECK for each required instrumentation channel, except Function 9, that is normally energized. | 31 days |
| SR 3.3.11.3 | Perform CHANNEL FUNCTIONAL TEST for function 9. | 31 days |
| SR 3.3.11.4 | Perform CHANNEL CALIBRATION, for functions 2,3,14,15,16,17, and 20. | 18 months |
| SR 3.3.11.5 | Perform CHANNEL CALIBRATION for functions 1,4,5,6,7,8,9,11,12,13,18, 19,21,22,23,24,25,26, and 27. | 24 months |

Table 3.3.11-1 (page 1 of 1)
Post Accident Monitoring Instrumentation

| FUNCTION | REQUIRED CHANNELS | CONDITIONS REFERENCED FROM REQUIRED ACTION F.1 |
|--|------------------------------------|---|
| 1. Excore Neutron Flux | 2 | F |
| 2. Reactor Coolant System Hot Leg Temperature | 2 (1 per steam generator) | F |
| 3. Reactor Coolant System Cold Leg Temperature | 2 (1 per steam generator) | F |
| 4. Reactor Coolant System Pressure (wide range) | 2 | F |
| 5. Reactor Vessel Water Level | 2(d) | G |
| 6. Containment Water Level (wide range) | 2 | F |
| 7. Containment Pressure (wide range) | 2 | F |
| 8. Containment Isolation Valve Position | 2 per penetration flow path (a)(b) | F |
| 9. Containment Area Radiation (high range) | 2 | G |
| 10. Deleted | | |
| 11. Pressurizer Level | 2 | F |
| 12. Steam Generator Water Level (wide range) | 2 per steam generator | F |
| 13. Condensate Storage Tank Level | 2 | F |
| 14. Core Exit Temperature - Quadrant 1 | 2(c) | F |
| 15. Core Exit Temperature - Quadrant 2 | 2(c) | F |
| 16. Core Exit Temperature - Quadrant 3 | 2(c) | F |
| 17. Core Exit Temperature - Quadrant 4 | 2(c) | F |
| 18. Auxiliary Feedwater Flow | 1 per steam generator | F |
| 19. Containment Pressure (narrow range) | 2 | F |
| 20. Reactor Coolant System Subcooling Margin Monitor | 2 | F |
| 21. Pressurizer Safety Valve Position Indication | 1 per valve | F |
| 22. Containment Temperature | 2 | F |
| 23. Containment Water Level (narrow range) | 2 | F |
| 24. HPSI Flow Cold Leg | 1 per cold leg | F |
| 25. HPSI Flow Hot Leg | 1 per hot leg | F |
| 26. Steam Line Pressure | 2 per steam generator | F |
| 27. Refueling Water Storage Tank Level | 2 | F |

(a) Not required for isolation valves whose associated penetration is isolated by at least one closed and de-activated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

(b) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

(c) A channel consists of two or more core exit thermocouples.

5.7 Reporting Requirements (continued)

5.7.2 Special Reports

Special Reports may be required covering inspection, test, and maintenance activities. These special reports are determined on an individual basis for each unit and their preparation and submittal are designated in the Technical Specifications.

Special Reports shall be submitted to the U. S. Nuclear Regulatory Commission, Attention: Document Control Desk, Washington, D. C. 20555, with a copy to the Regional Administrator of the Regional Office of the NRC, in accordance with 10 CFR 50.4 within the time period specified for each report.

The following Special Reports shall be submitted:

- a. When a pre-planned alternate method of monitoring post-accident instrumentation functions is required by Condition B or Condition G of LCO 3.3.11, a report shall be submitted within 30 days from the time the action is required. The report shall outline the action taken, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the function to OPERABLE status.
- b. Any abnormal degradation of the containment structure detected during the tests required by the Pre-Stressed Concrete Containment Tendon Surveillance Program shall be reported to the NRC within 30 days. The report shall include a description of the tendon condition, the condition of the concrete (especially at tendon anchorages), the inspection procedures, the tolerances on cracking, and the corrective action taken.
- c. Following each inservice inspection of steam generator (SG) tubes, in accordance with the SG Tube Surveillance Program, the number of tubes plugged and tubes sleeved in each SG shall be reported to the NRC within 15 days. The complete results of the SG tube inservice inspection shall be submitted to the NRC within 12 months following the completion of the inspection. The report shall include:
 1. Number and extent of tubes and sleeves inspected, and
 2. Location and percent of wall-thickness penetration for each indication of an imperfection, and
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(continued)
