



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET, SW, SUITE 23T85
ATLANTA, GEORGIA 30303-8931**

December 8, 2004

Tennessee Valley Authority
ATTN: Mr. K. W. Singer
Chief Nuclear Officer and
Executive Vice President
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

**SUBJECT: NOTIFICATION OF SEQUOYAH SAFETY SYSTEM DESIGN AND
PERFORMANCE CAPABILITY INSPECTION - NRC INSPECTION REPORT
05000327/20050006, 05000328/2005006**

Dear Mr. Singer:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a safety system design and performance capability inspection at your Sequoyah Nuclear Plant during the weeks of February 14, 2005, and February 28, 2005. A team of five inspectors and one trainee will perform this inspection. The inspection team will be led by Mr. Frank Jape, a Senior Project Manager from the NRC Region II Office. This biennial inspection will be conducted in accordance with baseline inspection program, Attachment 71111.21, "Safety System Design and Performance Capability."

The inspection will evaluate selected portions of the component cooling system. Procedures which direct the actions for operation of this system will also be evaluated.

During a telephone conversation on December 6, 2004, Mr. Jape of my staff, and R. Profitt of your staff, confirmed arrangements for an information gathering site visit and the two-week onsite inspection. The schedule is as follows:

- Information gathering visit: Week of January 18, 2005
- Onsite inspection weeks: February 14-18, 2005 and
February 28-March 4, 2005

The purpose of the information gathering visit is to obtain information and documentation outlined in the Enclosure needed to support the inspection. Mr. R. Bernhard, a Region II Senior Reactor Analyst, may accompany Mr. Jape during the information gathering visit to review probabilistic risk assessment data and identify risk significant components which will be examined during the inspection. Please contact Mr. Jape prior to preparing copies of the materials listed in the Enclosure. The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for inspection preparation.

During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space; specific documents requested to be made available to the team in their office space; arrangements for site access; and the availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection.

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Jape at (404) 562-4541 or me at (404) 562-4605.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

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Charles R. Ogle, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos.: 50-327, 50-328
License Nos.: DPR-77, DPR-79

Enclosure: Information Request for the Safety System Design and
Performance Capability Inspection

cc w/encl:

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cc w/encl cont'd - (See page 3)

TVA

3

(cc w/encl cont'd)

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**INFORMATION REQUEST FOR THE SAFETY SYSTEM DESIGN AND
PERFORMANCE CAPABILITY INSPECTION
COMPONENT COOLING SYSTEM, (CCS)**

Note: Electronic media is preferred if readily available. (The preferred file format is searchable “.pdf” files on CDROM. The CDROM should be indexed and hyperlinked to facilitate ease of use. Please provide 6 copies of each CDROM submitted. Information in “lists” should contain enough information to be easily understood by someone who has a knowledge of an ice condenser pressurized reactor technology.

1. Design basis documents for the system and components. Include calculations used to determine setpoints for the components.
2. Piping, electrical, and instrumentation drawings (P&IDs) for the systems. (Two paper copies are preferred for this request.)
3. Procedures used to implement the operation of the system. Include alarm response procedures, normal, and abnormal procedures as appropriate.
4. List of temporary modifications and operator work-arounds (for the past 2 years) involving any components required for operation of the system.
5. A list of major modifications completed in the past 5 years to the CCS.
6. System descriptions and operator training modules for the system.
7. List of operating experience program evaluations of industry, vendor, or NRC generic issues related to the CCS.
8. Probabilistic risk assessment (PRA) event tree for the CCS. A list of PRA identified system dependencies and success criteria for the system.
9. A brief description of the mitigation strategy for handling a loss of CCS event, including operator actions and equipment used.
10. System health reports and/or other performance monitoring information used to assess the performance of the CCS system.
11. A list of condition reports and non-routine work requests initiated since 2002 related to the CCS system.
12. Quality Assurance audits, self-assessments, and third party assessments performed on the CCS system.

Enclosure

13. Maintenance Rule performance criteria for the systems. A list of maintenance rule failures of equipment and components.
14. Plant Technical Specifications, Bases, and Technical Requirements Manual.
15. A current copy of the Updated Final Safety Analysis Report.
16. Procedures that provide implementation guidance for the following programs:
Corrective Action Program, Maintenance Rule Program, Design Control Program, and
Operating Experience Program.