UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
611 RYAN PLAZA DRIVE, SUITE 400
ARLINGTON, TEXAS 76011-4005

April 22, 2004

Mr. James Shetler, Assistant General Manager
Energy Supply
Sacramento Municipal Utility District
6201 'S' Street
P.O. Box 15830
Sacramento, California 95852

SUBJECT: NRC INSPECTION REPORT 050-00312/04-001; 072-00011/04-001

Dear Mr. Shetler:

An NRC inspection was conducted March 22-25, 2004, at your Rancho Seco Nuclear Generating Station. On March 25, 2004, at the conclusion of the inspection, an exit briefing was conducted with Mr. Steve Redeker, Plant Manager, and other members of your staff. The enclosed report presents the scope and results of that inspection.

The inspection was an examination of activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas, the inspection included reviews of safety reviews, design changes, and modifications; operation of your Independent Spent Fuel Storage Installation; maintenance and surveillances and the status of decommissioning activities. No violations of NRC regulations were identified during the inspection.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Should you have any questions concerning this inspection, please contact Emilio M. Garcia at (530) 756-3910 or the undersigned at (817) 860-8191.

Sincerely,

/RA/

D. Blair Spitzberg, Ph.D., Chief
Fuel Cycle and Decommissioning Branch

Docket Nos.: 050-00312; 072-00011
License Nos.: DPR-54; SNM-2510

Enclosure:
NRC Inspection Report
050-00312/04-001;072-00011/04-001

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ENCLOSURE

U.S. NUCLEAR REGULATORY COMMISSION
REGION IV

Docket Nos.:	050-00312; 072-00011
License Nos.:	DPR-54; SNM-2510
Report Nos.:	050-00312/04-001; 072-00011/04-001
Licensee:	Sacramento Municipal Utility District
Facility:	Rancho Seco Nuclear Generating Station
Location:	14440 Twin Cities Road Herald, California
Dates:	March 22-25, 2004
Inspectors:	Emilio M. Garcia, Health Physicist
Approved By:	D. Blair Spitzberg, Ph.D., Chief Fuel Cycle and Decommissioning Branch
Attachments:	Supplemental Information Partial List of Documents Reviewed
ADAMS Entry :	IR 05000312-04-001; 0720011-04-01 on 03/22-25/04; Sacramento Municipal Utility District; Rancho Seco Nuclear Generating Station. Decommissioning Report; No Violations.

EXECUTIVE SUMMARY

Rancho Seco Nuclear Generating Station
NRC Inspection Report 050-00312/04-001; 072-00011/04-001

All spent fuel had been removed from the spent fuel pool to the Independent Spent Fuel Storage Installation (ISFSI). During the inspection the licensee removed the pressurizer from the containment building, and began preparations for shipping it to a disposal site. The reactor vessel head had been segmented into five parts and shipped to a low-level waste disposal site. The licensee was continuing its dismantling activities in the reactor, auxiliary, and spent fuel buildings and outdoor areas.

Safety Reviews, Design Changes, Modifications and Review of 10 CFR 72.48 Evaluations

- Safety evaluations of changes to the facility and procedures described in the Safety Analysis Report had been performed as required by the licensee's procedures and the applicable regulations. Qualified reviewers and Commitment Management Review Group members and alternates were trained in performing the evaluations (Section 1).

Operation of an ISFSI

- The inspector observed that the doors to the horizontal storage modules remain sealed, and the ISFSI was free of debris. Radiation dosimeters were in their designated locations (Section 2).

- The licensee had conducted daily surveillances to monitor horizontal storage modules roof temperatures and to visually verify that the horizontal storage modules air vents were not blocked (Section 2).

- Direct radiation monitors placed around the ISFSI demonstrated that the annual dose equivalent to any real individual located outside the ISFSI controlled area did not exceed regulatory limits (Section 2).

- Effluent release reports for the ISFSI had been submitted late for years 2001, 2002, and 2003. The late filings had no safety significance, however, it was noted that licensee identified corrective actions had not prevented recurrence. Consequently, an inspection followup item was opened to track the licensee's ongoing corrective actions (Section 2).

Maintenance and Surveillance

- There was minimal maintenance and surveillance required due to the relocation of the spent fuel to the ISFSI and the disposal of the spent fuel pool water. Maintenance activities observed indicated that the licensee had an effective program (Section 3).

Decommissioning Performance and Status Review

- The licensee continued to conduct dismantlement activities in the reactor, auxiliary, spent fuel buildings and other areas of the site in a generally safe manner (Section 4).

- The licensee removed the pressurizer from the containment building, and began preparations for shipping it to a disposal site (Section 4).

- Work was proceeding on cutting and removing the spent fuel pool liner plates (Section 4).

Report Details

Summary of Facility Status

The Rancho Seco facility was undergoing active decommissioning with dismantlement work in progress in the auxiliary, reactor and spent fuel buildings and outdoor areas. Most major components in the auxiliary building have been removed, packaged and shipped for disposal. In the reactor building, most of the major piping, the four reactor coolant pumps, the core flood tanks, and the reactor vessel head had been removed, packaged and disposed. During this inspection, the licensee removed the pressurizer from the containment building, and began preparations for shipping it to a disposal site.

The licensee had removed all 493 spent fuel assemblies from the spent fuel pool. Twenty-one canisters had been loaded with spent fuel and transferred to the Independent Spent Fuel Storage Installation (ISFSI). In the fuel handling building, the spent fuel pool water had been processed and released offsite. The cutting and removal of the pool liner plates was in progress.

1 Safety Reviews, Design Changes, Modifications and Review of 10 CFR 72.48 Evaluations (IP 37801 and IP 60857)

1.1 Inspection Scope

The inspector reviewed selected 10 CFR 50.59 and 10 CFR 72.48 safety screens and the qualifications of the reviewers that had been conducted since the previous inspection in this area.

1.2 Observations and Findings

This area was last inspected in September 2003, see Inspection Report 50-133/03-04. Section 2.2 of the report described the safety screening used by the licensee after the relocation of all the spent fuel to the ISFSI and the issuance of license Amendments 129 and 130. The inspector reviewed the Commitment Management Review Group (CMRG) minutes for the period of September 2003 through March 24, 2004. These minutes indicated that during this time, four safety reviews had been approved by the CMRG. The inspector reviewed the records of the evaluations conducted during that period. The packages were complete and signed by a qualified reviewer and the plant manager for the CMRG. The inspector noted that the minutes of the CMRG indicated that these evaluations had been reviewed, discussed and unanimously approved by the CMRG. Coincidently all of these reviews had been conducted by one reviewer. The inspector noted that this individual was listed on the list of qualified reviewers and that training records indicated that he had successfully completed training as a 10 CFR 50.59 and 10 CFR 72.48 reviewer, and received refresher training within the last 12 months. Records maintained by the licensee indicated that CMRG members and alternates had been trained. CMRG training included being a qualified reviewer.

1.3 Conclusion

Safety evaluations of changes to the facility and procedures described in the Safety Analysis Report had been performed as required by the licensee's procedures and the applicable regulations. Qualified reviewers and Commitment Management Review Group members and alternates were trained in performing the evaluations

2 Operation of an Independent Spent Fuel Storage Installation (ISFSI) (60855)

2.1 Inspection Scope

The inspector toured the ISFSI, observed ongoing maintenance activities, and reviewed compliance with selected ISFSI license technical specifications.

2.2 Observations and Findings

a. Tours

On March 22, 2004, the inspector toured the ISFSI and observed that the welds on the horizontal storage modules (HSM) doors appeared unchanged and all the doors were closed. Appropriate radiological postings were noted. The bird screens between the HSM and the HSM air vents were clear of debris. The inner fence reached within a few inches of the gravel, and the eight inner radiation monitors were in their storage canisters. There was one fire extinguisher inside the ISFSI and the attached tag indicated that it was within is inspection interval. There were no vehicles parked inside the ISFSI. The location of the read-out for the plant integrated computer system (PICS) was staffed and the PICS was operational.

On March 24 the inspector toured the 100-meter fence and noted that seven direct radiation monitors were in their storage location and the locations identification coincided with the designations on the licensee's map in Annual Radiological Environmental Operating Report for January-December 2002.

b. Daily Surveillance

Rancho Seco ISFSI Technical Specification 5.5.3, HSM Thermal Monitoring Program, required, in part, that each HSM roof temperature be monitored and if the temperature rises by more than 80 degrees Fahrenheit or is 225 degrees Fahrenheit or greater, that appropriate corrective action be taken to avoid exceeding the concrete and cladding temperature limits. This technical specification also required that the licensee conduct daily visual inspections of the air vents to ensure that HSM air vents are not blocked.

Rancho Seco used Operation Surveillance Procedure, SP-10, to perform daily surveillances to monitor the HSM roof temperature and to visually inspect the HSM air vents. On March 25, 2004, the inspector interviewed an ISFSI technician on duty and reviewed the previous day's SP-10 data sheets. The individual was familiar with the requirements of SP-10 and reasons for conducting the surveillance. The inspector selected 23 days and reviewed the associated SP-10 data sheets for completion of the

Rancho Seco ISFSI Technical Specification 5.5.3 requirements. The 24-hour temperature raise and maximum temperature had been evaluated for each of the selected days and the limits had not been exceeded and the records indicated that the individual performing the surveillance had visually examined the HSM air vents and found them to not be damaged or plugged.

c. Radiological Environmental Monitoring Program

The Rancho Seco ISFSI Technical Specification 5.5.2 states, in part, the radiological environmental monitoring program ensures the annual dose equivalent to any real individual located outside the ISFSI controlled area does not exceed regulatory limits in 10 CFR 72.104(a), and that dosimetry will be used to monitor direct radiation around the ISFSI. 10 CFR 72.104(a) stated, in part, that during normal operations and anticipated occurrences, the annual dose equivalent to any real individual who is located beyond the controlled area boundary must not exceed 25 millirem (or 0.025 rem) to the whole body.

The inspector interviewed the radiological health supervisor and reviewed portions of the Annual Radiological Environmental Operating Report for January-December 2002. The report for 2003 was not available at the time of the inspection. The licensee had established a number of direct radiation monitoring locations around the site including the ISFSI. The radiological health supervisor identified seven locations as applicable to evaluating the direct radiation due to the ISFSI. Through review of data for the period January-December 2002, the inspector determined that the annual dose from the ISFSI was below the applicable regulatory limits.

d. Radiological Effluent Release Reports

The Rancho Seco ISFSI Technical Specification 5.5.2.d and 10 CFR 72.44(d)(3) require that an annual report be submitted to the Commission specifying the quantity of each of the principal radionuclides released to the environment in liquid and in gaseous effluents during the previous calendar year of operation. 10 CFR 72.44(d)(3) specifies that the report must be submitted within 60 days after the end of the 12-month monitoring period. Since the Rancho Seco ISFSI is by design a sealed system, no gaseous or liquid effluents were produced. In fact Technical Specification 5.5.2.b states that the "operation of the Rancho Seco ISFSI will not create any radioactive materials or result in any credible liquid or gaseous effluent release."

The licensee submitted the required reports for calender years 2001, 2002 and 2003 late. On April 23, 2003, the licensee identified that the report was overdue and opened a Potential Deviation from Quality (PDQ) report. This PDQ, stated that operation of the ISFSI began on February 11, 2002, and that the licensee needed to provide a report of the information requested in 10 CFR 72.44(d)(3). In fact, the first spent fuel was loaded on April 19, 2001. On June 26, 2003, the licensee submitted the report for calender year 2002. There was no report for 2001. Part of the corrective action from the PDQ was to change the controlling procedure to include the ISFSI effluent release report with the Part 50 Radiological Environmental Monitoring Program. The Part 50 Radiological Environmental Monitoring Program is due 90 days after the end of each year. 10 CFR 72.44(d)(3) specifies that the report must be submitted within 60 days after the end of the 12-month monitoring period.

On March 24, 2004, the inspector identified that the year 2001 report had not been sent and that the report for the year 2003 was late.

The licensee issued a second PDQ to address the problems not resolved by the initial PDQ. Although there is no safety significance to the reports required by the 10 CFR 72.44(d)(3), nonetheless, the licensee was required to submit the reports on a timely basis. The corrective actions that will be taken by the licensee to assure that regulatory required reports are submitted on a timely basis will be reviewed in a future inspection followup item (IFI) (072-00011/2004-01-01).

2.3 Conclusion

The inspector observed that the doors to the horizontal storage modules remain sealed, and the ISFSI was free of debris. Radiation dosimeters were in their designated locations. The licensee had conducted daily surveillances to monitor horizontal storage modules roof temperatures and to visually verify that the horizontal storage modules air vents were not blocked. Direct radiation monitors placed around the ISFSI demonstrated that the annual dose equivalent to any real individual located outside the ISFSI controlled area did not exceed regulatory limits. Effluent release reports for the ISFSI had been submitted late for years 2001, 2002, and 2003. The late filings had no safety significance, however, it was noted that licensee identified corrective actions had not prevented recurrence. Consequently, an inspection followup item was opened to track the licensee's ongoing corrective actions.

3 Maintenance and Surveillance (IP 62801)

3.1 Inspection Scope

The inspector evaluated the licensee's maintenance program by observing two maintenance department activities being performed.

3.2 Observations and Findings

With the relocation of the spent fuel to the ISFSI and the treatment and release of the spent fuel pool water, the structures, systems and components (SSC) important to the safe storage of spent fuel, and source of radioactive material that could result in offsite effluents had been substantially reduced on the Part 50 portion of the site. There was very little applicable maintenance and surveillance. On March 22, 2004, the inspector observed workers performing two maintenance department activities, installation of a gate alarm and trouble shooting and servicing an air conditioner unit for the ISFSI Communications Building. The first activity was to provide easier access to the ISFSI while maintaining required access control. The second activity was to ensure that temperature limits were not exceeded in the ISFSI Communications Building that provides telemetric data of ISFSI roof temperature sensors. Although, both of these activities were not in the Part 50 portion of the site, they did involve personnel that would perform Part 50 maintenance and activities of SSCs important to the safety of the spent fuel stored in the ISFSI.

The inspector interviewed the workers performing the maintenance, reviewed the work packages that they were using and observed the work being performed. The inspector observed that the design engineer for the gate alarm package visited the work area during the installation and examined the work that had been completed prior to his arrival. One of the individuals working on the air conditioner was the mechanical maintenance supervisor. The inspector observed appropriate tag-outs, clearances and approvals had been obtained, and noted that the individuals performing the maintenance were knowledgeable of their job tasks, and had appropriate work orders and schematic drawings.

There was no backlog of required maintenance work.

3.3 Conclusion

There was minimal maintenance and surveillance required due to the relocation of the spent fuel to the ISFSI and the disposal of the spent fuel pool water. Maintenance activities observed indicated that the licensee had an effective program.

4 Decommissioning Performance and Status Review (IP 71801)

4.1 Inspection Scope

The licensee's dismantlement activities were reviewed. Tours of the site were conducted to observe work activities underway, including observation of housekeeping, safety practices, fire loading and radiological controls. The inspector observed the removal of the pressurizer from the containment building

4.2 Observations and Findings

Tours of the reactor, auxiliary, and spent fuel buildings, and other areas of the plant were conducted to observe dismantling and decommissioning activities in progress. The work observed was being conducted in a safe and orderly manner. Radiological controls, including postings and barriers, were in place as needed. The inspector noted good housekeeping, radiological and fire protection practices in all areas. Major activities observed are noted below.

a. Removal of the Pressurizer from the Containment/Reator Building

During this inspection, the major activity was the removal of the pressurizer from the containment/reactor building. Prior to removal, the structural steel around the pressurizer and on its path out of the building had been removed. The licensee had contracted with Maxim Crane Works to remove the pressurizer from the reactor building and load it onto a special rail truck. The inspector interviewed selected individuals working for this contractor and found them to be knowledgeable of their radiation safety responsibilities and practices, including the requirements of the radiation work permit for the job. The pressurizer had been painted with a polymeric coating to encapsulate and minimize removable contamination. No personnel contamination incidents occurred. The lifting of the pressurizer required the used of both hooks of the reactor building

polar crane. In order to maintain control of the approximate 150-ton load, the location of the center of gravity and maintaining an amount of tilt was necessary. Since the design of the crane did not permit the operation of both hooks simultaneously, the lift required the crane operator to shift control from one hook to the other while remaining within the limits of tilt. Close proximity to remaining structural steel and the concrete biological shield further limited the permissible range of motion as the pressurizer was lifted or lowered first with one hook then the other. Figures 1-8 show portions of the pressurizer movement.

b. Reactor Vessel Head Segmentation

During the previous inspection, the major activity had been the cutting of the reactor vessel head into segments for ease of disposal. The segmentation that had began in early November 2003, and was completed by January 22, 2004. All the pieces had been removed from the reactor building by the end of February and shortly after had been shipped for disposal at a low-level radioactive waste site.

c. Reactor Building

The major activity was the removal of the pressurizer from the building. The fuel transfer tubes were being pulled into the reactor building and segmented for packaging and eventual disposal. Removal of miscellaneous structural steel and piping from the South D ring was continuing.

d. Auxiliary Building

Work had been completed on sectioning and removing the A Coolant Waste Receiver Tank, the last remaining major component in the auxiliary building. Electrical equipment, cabling, heating, ventilation, and air conditioning ducting was being removed from various elevations in the building. The Bulk Waste Building had been dismantled to permit the removal of the pressurizer from the reactor building.

e. Fuel Handling Building

Work was proceeding on milling and removing the spent fuel pool liner plates. The liner plates had been removed from most of the walls. The rack lifting trolley had been removed from the fuel bridge lead abated, segmented and packaged for disposal.

f. Industrial Accident

On March 4, 2004, an electrician working in a non-contaminated area fell approximately 10-feet. The individual suffered a slight concussion and some bruised ribs. Although the licensee's industrial safety procedures required fall protection equipment for potential falls of 6-feet or greater, no fall protection equipment was used. The California Occupational Health and Safety Administration had informed the licensee that they intended to conduct a site visit.

4.3 Conclusion

The licensee continued to conduct its dismantlement activities in the reactor, auxiliary, spent fuel buildings and other areas of the site in a generally safe manner. The reactor vessel head had been segmented and disposed. The pressurizer was removed from the reactor building and was being prepared to ship for disposal. All major components had been removed and disposed from the auxiliary building. The liner plates had been removed from most of the walls in the spent fuel pool.

5 Exit Meeting Summary

The inspector presented the inspection results to members of licensee management and staff at the exit meeting on March 25, 2004. No information reviewed by the inspector was identified as proprietary.

Figure 1
Rancho Seco Pressurizer
Lifted out of South Biological Shield “D” Ring

Figure 2
Rancho Seco Pressurizer
Approaching North Biological Shield “D” Ring

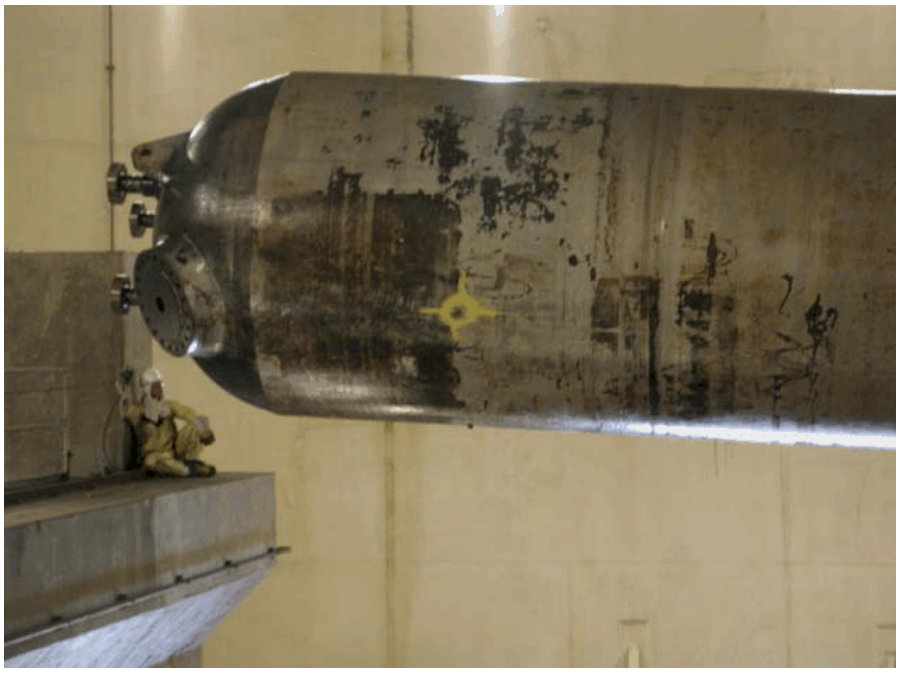

Figure 3
Rancho Seco Pressurizer
Moving toward Reactor Building Equipment Hatch

Figure 4
Rancho Seco Pressurizer
Tilted by Workers toward Equipment Hatch Bay

Figure 5
Rancho Seco Pressurizer
Rigging Shackle Details

Figure 6
Rancho Seco Pressurizer
Crane Operator Using Remote Controls

Figure 7
Rancho Seco Pressurizer
Pulled Outside Reactor Building Equipment Hatch

Figure 8
Rancho Seco Pressurizer
Lowered by Two Cranes at Shipping Stagging Area

ATTACHMENT 1

PARTIAL LIST OF PERSONS CONTACTED

Sacramento Municipal Utility District

M. Bua, Radiation Protection/Chemistry Superintendent
J. Delezenski, Quality Assurance/Licensing/Administration/Training Superintendent
M. Dobleman, Cost Scheduling Specialist
D. Gardner, Decommissioning Project Manager
R. Jones, Sr. Nuclear Engineer
R. Mannheimer, Sr., Quality Control Engineer
S. Nicolls, Radiological Health Supervisor
L. Porteous, Mechanical Maintenance Supervisor
S. Redeker, Manager, Plant Closure and Decommissioning
G. Roberts, Maintenance Superintendent
S. Speyer, Maintenance
K. Zucco, Maintenance
A. Zwierzynski, ISFSI Technician

J. A. Jones

R. Gately, Electrician

INSPECTION PROCEDURES USED

IP 37801	Safety Reviews, Design Changes, and Modifications
IP 60855	Operation of an ISFSI
IP 60857	Review of 10 CFR 72.48 Evaluations
IP 62801	Maintenance and Surveillance
IP 71801	Decommissioning Performance and Status Review

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

072-00011/2004-01-01	IFI	Corrective actions to assure that reports required by regulations are submitted on a timely basis.

Closed

None

Discussed

None

LIST OF ACRONYMS

CMRG	Commitment Management Review Group
CY	Calender Year
HSM	Horizontal Storage Modules
IFI	Inspection Followup Item
ISFSI	Independent Spent Fuel Storage Installation
PDQ	Potential Deviation from Quality
PICS	Plant Integrated Computer System
SSC	Structures, Systems and Components

ATTACHMENT 2

PARTIAL LIST OF DOCUMENTS REVIEWED

Agendas

- Agenda CMRG Meeting Held on September 24, 2003, at 8:00 a.m.
- Agenda for Special CMRG Meeting Held on October 16, 2003, at 8:00 a.m.
- Agenda CMRG Meeting Held on January 14, 2004, at 8:00 a.m.

Correspondences and Memorandums

- MPC&D 03-077, June 26, 2003, from Manager, Plant Closure & Decommissioning to U. S. Nuclear Regulatory Commission, Subject: Rancho Seco ISFSI Effluent Release Report.
- MPC&D 03-083, July 14, 2003, from Manager, Plant Closure & Decommissioning to CMRG Members and Alternates, Subject: CMRG Membership.
- MPC&D 03-121, October 23, 2003, from Manager, Plant Closure & Decommissioning to Qualified Reviewers, Subject: Qualified Reviewers List.
- MPC&D 04-027, March 18, 2004, from Manager, Plant Closure & Decommissioning to U. S. Nuclear Regulatory Commission, Subject: 2003 Annual Radioactive Effluent Release Report.
- MPC&D 04-032, March 24, 2004, from Manager, Plant Closure & Decommissioning to U. S. Nuclear Regulatory Commission, Subject: Rancho Seco ISFSI Effluent Release Report.
- NQA 03-081, September 24, 2003, from Richard Mannheimer to Steve Redeker (Plant Manager), Subject: Minutes for CMRG Meeting Held on September 24, 2003.
- NQA 03-095, October 16, 2003, from Richard Mannheimer to Steve Redeker, Subject: Minutes for CMRG as SSRC Meeting Held on October 16, 2003.
- NQA 04-009, January 14, 2004, from Richard Mannheimer to Plant Manager, Subject: Minutes for CMRG as SSRC Meeting Held on January 14, 2004.
- Watts Happening Newsletter, March 15, 2004 issue.
- Watts Happening Newsletter, March 22, 2004 issue.

Procedures and Data Sheets

- Rancho Seco used Operation Surveillance Procedure, SP-10, ISFSI & Instrument Checks & System Verification Daily Surveillance, Revision 1, effective August 26, 2002.

- Procedure, SP-10, data sheets for January 1, 2004.
- Procedure, SP-10, data sheets for January 10, 2004.
- Procedure, SP-10, data sheets for February 6, 2004.
- Procedure, SP-10, data sheets for February 14, 2004.
- Procedure, SP-10, data sheets for February 27-29, 2004.
- Procedure, SP-10, data sheets for March 1-15, 2004.
- Procedure, SP-10, data sheets for March 24, 2004.

Potential Deviation from Quality (PDQ) Reports

- PDQ 03-0016, Issue Identified on April 23, 2003, Annual Effluent report for the Part 72 ISFSI license was not submitted by the required due date. Date due would be 60 days after operation of the ISFSI commenced (Feb 11, 2002).
- PDQ 04-0008, Issue Identified on March 24, 2004, ISFSI Annual Effluent report for 2001 not submitted to NRC. ISFSI Annual Effluent report for 2003 not submitted to NRC by required due date (Feb 29, 2004).

Reports

- Rancho Seco Training Information Management System Report of training received by Steve Nicolls as of March 25, 2004.

10 CFR 50.59 and 10 CFR 72.48 Screenings and Evaluations

- Change to ISFSI FSAR Section 7.5.2, approved September 24, 2003.
- CAP-0002, Revision 16 (ODCM), implementation of DCP R03-0010, “Liquid Effluent System Reconfiguration,” approved October 16, 2003.
- Emergency Plan (Change 5), approved January 14, 2004.
- Emergency Plan Implementing Procedure - 01, approved January 14, 2004.

Work Orders and Drawings

- 18009060, Service air conditioning unit A-245B, ISFSI Communications Building (Z-45) air conditioning unit.
- Drawing E32.08-25, Sheets 2, 3, 4 and 5.
- Drawing E-350, Sheets 22, 26, and 31
- Personnel Gate Position Switch Installation Field Notes.