

March 11, 2004

Mr. J. A. Scalice
Chief Nuclear Officer and
Executive Vice President
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

SUBJECT: SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2, AND WATTS BAR NUCLEAR PLANT, UNIT 1 - APPROVAL TO USE MORE RECENT EDITION OF THE AMERICAN SOCIETY OF MECHANICAL ENGINEERS BOILER AND PRESSURE VESSEL CODE FOR ALTERNATIVE QUALIFICATION OF VT-2 EXAMINATION PERSONNEL (TAC NOS. MC2272, MC2273, AND MC2274)

Dear Mr. Scalice:

By letter dated March 2, 2004, the Tennessee Valley Authority (TVA or the licensee) requested the U.S. Nuclear Regulatory Commission's (NRC's) approval to use a more recent edition of the American Society of Mechanical Engineers Boiler and Pressure Vessel (ASME) Code for the alternative qualification of VT-2 (visual) examination personnel for the Sequoyah Nuclear Plant (SQN), Units 1 and 2, and for the Watts Bar Nuclear Plant (WBN), Unit 1. The licensee requested approval to use paragraph IWA-2316 of the 1998 ASME Section XI Code related to the alternative qualification of VT-2 examination personnel for the remainder of the current 10-year inservice inspection interval for each unit. SQN Units 1 and 2 are currently in the second inservice inspection interval, which is scheduled to end on December 15, 2005. WBN Unit 1 is currently in its first inservice inspection interval, which ends December 31, 2006.

The licensee stated that paragraph IWA-2316 provides an alternative to the requirements in the ASME Section XI and that the embedded vision test requirements specified in IWA-2321, IWA-2322, and Table IWA-2322-1, are the only related requirements from the 1998 Edition which are associated with IWA-2316. The 1998 Edition up to and including the 2000 Addenda of the ASME Section XI Code were incorporated in Title 10, *Code of Federal Regulations* (10 CFR), Section 50.55a(b)(2)(xviii), with one limitation placed on the use of IWA-2316. The limitation states that it may only be used to qualify personnel that observe for leakage during system leakage and hydrostatic tests conducted in accordance with the IWA-5211(a) and (b), 1998 Edition through the latest edition and addenda incorporated by reference in paragraph (b)(2).

As stated in 10 CFR 50.55a(g)(4)(iv), the inservice examination of components and system pressure tests may meet the requirements set forth in subsequent editions and addenda that are incorporated by reference in Section 50.55a(b), subject to the limitations and modifications listed therein and NRC approval. Portions of the editions or addenda may be used provided that all related requirements of the respective editions and addenda are met. TVA's request to use the portions of the 1998 ASME Section XI Code associated with alternative qualification of VT-2 examination personnel for observation of leakage during system leakage and hydrostatic

tests is approved pursuant to 10 CFR 50.55a(g)(4)(iv), for the remainder of the current inservice inspection interval for each unit, subject to the following limitation: paragraph IWA-2316 of the 1998 ASME Code may only be used to qualify alternate VT-2 examiners who observe system leakage tests or fluid hydrostatic tests conducted in accordance with the IWA-5211(a) and (b), 1998 Edition through the latest edition and addenda incorporated by reference in paragraph (b)(2).

Sincerely,

/RA/

William F. Burton, Acting Chief, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-327, 50-328, 50-390

cc: See next page

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Mr. J. A. Scalice
Tennessee Valley Authority

**SEQUOYAH NUCLEAR PLANT
WATTS BAR NUCLEAR PLANT**

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Mr. J. A. Scalice
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**SEQUOYAH NUCLEAR PLANT
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