

February 12, 2004

Mr. Gregory A. Maret
Vice President - Decommissioning
Yankee Atomic Electric Company
49 Yankee Road
Rowe, Massachusetts 01367

SUBJECT: NRC INSPECTION REPORT NO. 50-029/2003-002

Dear Mr. Maret:

On January 21, 2004, the NRC completed an inspection at your nuclear reactor facility in Rowe, Massachusetts, which covered an inspection period that began on June 28, 2003. The findings of the inspection were discussed with you and members of your staff on January 21, 2004. The enclosed report presents the results of that inspection.

Your radiation protection, quality assurance and self-assessment, and maintenance and operations department surveillance programs were inspected during this inspection period. In addition, your activities involving the survey and unconditional release of your turbine building and other onsite structures were reviewed. An independent survey was completed by our consultant, the Oak Ridge Institute for Science and Education, (ORISE), to verify adequacy of your release program. Response to the NRC Security Order issued October 16, 2002, to all decommissioning reactor facilities operating a Dry Independent Spent Fuel Storage Installation, was also reviewed. The inspection consisted of selective examinations of procedures and representative records, interviews with personnel, and observations by the inspector. The programs were considered to be appropriately implemented and no safety concerns were identified.

In accordance with Section 2.790 of the NRC's "Rules and Practices," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and its enclosure will be placed in the NRC Public Document Room (PDR) and will be accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html>. No reply to this letter is required.

Sincerely,

/RA by Marie Miller Acting for/

Ronald R. Bellamy, Chief
Decommissioning and Laboratory Branch
Division of Nuclear Materials Safety

Docket No. 05000029
License No. DPR-03

Enclosure: NRC Region I Inspection Report No. 50-029/2003-002

cc w/encl:

R. Kacich, YAEC President

J. Kay, Principal Licensing Engineer

G. van Noordennen, Manager, Regulatory Affairs

B. Woods, Site Manager

R. Walker, Department of Public Health, Commonwealth of Massachusetts

K. Smith, Yankee Rowe Community Advisory Board

Citizens Awareness Network

Commonwealth of Massachusetts, SLO Designee

State of Vermont, SLO Designee

Mr. G. Maret

3

Distribution w/encl:
J. Jolicoeur, OEDO
J. Hickman, NMSS
R. Temps, SFPO
D. Screnci, PAO
N. Sheehan, PAO
IPAS

DOCUMENT NAME: C:\ORPCheckout\FileNET\ML040430204.wpd

After declaring this document "An Official Agency Record" it **will/will not** be released to the Public.

To receive a copy of this document, indicate in the box: "C" = Copy without attachment/enclosure "E" = Copy with attachment/enclosure "N" = No copy

OFFICE	RI:DNMS		RI:DNMS		RI:DNMS				
NAME	Wray JRW		Bellamy MTM1		Pangburn FMC				
DATE	2/11/04		2/11/04		2/11/04				

OFFICIAL RECORD COPY

U.S. NUCLEAR REGULATORY COMMISSION

REGION I

Docket No. 05000029

License No. DPR-03

Report No. 50-029/2003-002

Licensee: Yankee Atomic Electric Company (YAEC)
580 Main Street
Bolton, Massachusetts 01740-1398

Facility Name: Yankee Nuclear Power Station

Location: Rowe, Massachusetts

Dates: June 28, 2003 to January 21, 2004

Inspector: J. Wray, CHP, Health Physicist, DNMS
M. Miller, Senior Health Physicist, DNMS

Approved by: Ronald R. Bellamy, PhD, Chief, Decommissioning Branch, Region I

EXECUTIVE SUMMARY

Yankee Facility NRC Inspection Report No. 50-029/2003-002

Inspections were conducted to determine whether the decommissioning activities carried out at the Yankee Rowe facility were conducted safely and in accordance with NRC requirements. Areas reviewed included the radiation protection program, quality assurance and self-assessment program, security, maintenance and operations department surveillance programs, spent fuel pool clean up activities, the survey and unconditional release of your turbine building, preparations for cold weather operations, and Independent Spent Fuel Storage Installation (ISFSI) operations.

Operations and Decommissioning Status

An effective maintenance and surveillance program relative to safe storage, maintenance and control of spent fuel was maintained.

The licensee established an adequate program to maintain the operability of systems and equipment, identified as important to safety, during the cold weather season.

The licensee maintained an adequate program for demolition and the unconditional release of buildings and structures in the non-radiological controlled area (RCA) side of the plant. Surveys conformed to the monitoring requirements of IE Circular 81-07. Buildings and structures demolished inside the RCA were shipped to a licensed radioactive waste disposal site.

The licensee maintained an adequate program to survey and monitor the ISFSI and stored spent fuel.

The licensee and their contractor DEMCO maintained effective corrective action programs and performed audits and assessments to help self-identify and correct issues and problems.

YAEC effectively implemented their security program for compliance with the May 23, 2002 Order for Interim Compensatory Measures (ICMs) for safeguards and security for Yankee Rowe while spent fuel was stored in the Spent Fuel Pool (SFP). Following removal of all spent fuel from the SFP, the licensee adequately complied with the October 16, 2002, Order applicable to operational, stand alone ISFSIs.

Plant Support and Radiological Controls

The licensee has provided good controls to limit exposures of workers to external sources of radiation. The relocation of the RCA control point was completed in a safe manner and in accordance with regulatory requirements. Radiation protection and chemistry laboratory equipment were adequately recalibrated and returned to service following the relocation.

The licensee completed processing and drain down of the SFP water in a safe and compliant manner. Discharge of the SFP water to the Deerfield River was conducted in accordance with National Pollutant Discharge Elimination System (NPDES) and Off Site Dose Calculation

Manual (ODCM) requirements.

Results of independent measurements performed by ORISE confirm that the radiological conditions of the turbine building, service building annex, and the turbine building office area rubble satisfy the criteria for release of these structures for unrestricted use.

TABLE OF CONTENTS

EXECUTIVE SUMMARY	ii
TABLE OF CONTENTS	iv
REPORT DETAILS	1
<u>Summary of Facility Activities</u>	1
I. Operations and Decommissioning Status	1
O1 Conduct of Operations	1
O1.1 <u>Maintenance and Surveillance Program</u>	1
O1.2 <u>Station Freeze Protection Program</u>	2
O1.3 <u>Status of Structures and Building Demolition</u>	2
O1.4 <u>Independent Spent Fuel Storage Installation Operations</u>	3
O1.5 <u>Quality Assurance Audits and Self Assessments</u>	4
S1 Conduct of Security and Safeguards	5
S1.1 <u>Interim Compensatory Measures at Decommissioning Nuclear Power Plants</u> ..	5
II. Plant Support and Radiological Controls	6
R1 Radiological Protection and Chemistry Controls	6
R1.1 <u>External Exposure Controls</u>	6
R1.2 <u>Spent Fuel Pool Draindown</u>	7
R1.3 <u>Independent Confirmatory Survey of Turbine Building</u>	8
III. MANAGEMENT MEETINGS	9
X1 Exit Meeting Summary	9
X2 Other Meetings	9
PARTIAL LIST OF PERSONS CONTACTED	10
LIST OF ACRONYMS	11
INSPECTION PROCEDURES USED	12
ITEMS OPENED, CLOSED, AND DISCUSSED	12

REPORT DETAILS

Summary of Facility Activities

Decommissioning activities at the Yankee Rowe Nuclear Power Station continued under the approval granted through a letter from the NRC to Mr. James Kay (October 28, 1996).

I. Operations and Decommissioning Status

O1 Conduct of Operations

O1.1 Maintenance and Surveillance Program

a. Inspection Scope (62801)

The inspector reviewed the licensee's maintenance and surveillance program including planned and completed maintenance and surveillance activities of structures, systems and components important to the safe storage of spent fuel. The inspector reviewed the preventive and corrective maintenance program and observed the performance of selected preventive maintenance activities.

b. Observations and Findings

Structures, systems, and components were in good material condition. Informational tags on equipment were appropriate and housekeeping was adequate. Appropriate security and fire protection measures were in place.

The inspector reviewed the preventive maintenance program and performance goals, including work prioritization, specific implementing procedures, and the work order tracking system procedure. Work is planned and coordinated with the appropriate departments in a timely manner, and work orders and specific procedures are provided in advance. The licensee effectively managed the preventive maintenance program, including the backlog and emergent work. There was minimal backlog of maintenance requests during this inspection period. The inspector reviewed the maintenance department surveillance schedule and observed selected scheduled surveillances on plant equipment and systems. Work orders and procedures were at the job site. Follow-up work and testing were appropriately completed. The inspector reviewed the completed surveillance of the liquid waste monitor channel and calibration of the auxiliary service water effluent radiation monitor. No safety concerns were identified.

c. Conclusions

An effective maintenance and surveillance program relative to safe storage, maintenance and control of spent fuel was maintained.

O1.2 Station Freeze Protection Program

a. Inspection Scope (71714)

The inspector evaluated the licensee's preparations to maintain the operability of systems and equipment, identified as important to safety, during the cold weather season.

b. Observations

The inspector reviewed preventive maintenance and operations procedures, checklists and the schedule for weekly tests and checks of the equipment. The inspector observed a licensee representative conduct a required surveillance and discussed the process to resolve as-found deficient conditions. The inspector verified that procedures OP-2115, "Warm Or Cold Weather Operation", and OP-5751, "Freeze Protection Inspection", were completed. Heat-trace equipment was energized where required and area space heaters were deployed where required. No safety concerns were identified.

c. Conclusions

The licensee established an adequate program to maintain the operability of systems and equipment, identified as important to safety, during the cold weather season.

O1.3 Status of Structures and Building Demolition

a. Inspection Scope (71801)

The inspector reviewed activities associated with building demolition and dismantlement of onsite structures. Information was gathered through a review of procedures and documents, tours of the site, and through interviews with cognizant personnel.

b. Observations and Findings

During this inspection period, the turbine building, the turbine building office area, the stores warehouse, and the service building annex were razed. These buildings were not part of the RCA. The inspector reviewed pre-demolition survey plans and design change packages for these structures. The licensee performed radiological surveys to release the buildings for unrestricted use and disposal. The inspector observed scanning of floors and walls of the service building annex and discussed calibration of the gas flow proportional detectors used for the beta/gamma surveys. Surveys for release for unrestricted use were completed in accordance with the monitoring requirements specified in IE Circular 81-07, "Control of Radioactively Contaminated Material", May 8, 1981. Some radioactive contamination was detected and the inspector verified that remedial actions were appropriate. The licensee disposed of the material as radioactive waste when detectable radioactivity was identified. The inspector noted that historical use data was used in the characterization of survey plans. No violations were identified.

The inspector observed management review meetings for the turbine building demolition and reviewed DEMCO Task Package DEM-TP-029 (TB Demolition) and DEM-WP-06 (Pedestal Implosion Work Package). The reviews were thorough and detailed. The inspector verified issues were tracked and adequately resolved prior to commencement of demolition work. The inspector noted precautions were included for handling asbestos materials and for safeguarding explosives used for imploding the reinforced concrete turbine pedestal. The inspector observed proper packaging of waste material into containers. No safety concerns were identified.

The licensee continued remediation and demolition activities in the RCA. The inspector reviewed documentation of the demolition of the Fuel Transfer Enclosure and the Yard Crane. They were dismantled in accordance with procedures and transported for disposal as radioactive waste. The inspector discussed with licensee representatives the preliminary plans for dismantlement of the reactor sphere and observed preparations for the demolition of the remote shutdown building. No safety concerns were identified.

c. Conclusion

The licensee maintained an adequate program for demolition and the unconditional release of buildings and structures in the non-RCA side of the plant. Surveys conformed to the monitoring requirements of IE Circular 81-07. Buildings and structures demolished inside the RCA were shipped to a licensed radioactive waste disposal site.

O1.4 Independent Spent Fuel Storage Installation Operations

a. Inspection Scope (60855)

The inspector toured the ISFSI, reviewed records of Technical Specification (TS) required surveillances, and discussed controls of ISFSI activities with licensee operations and security personnel.

b. Observations and Findings

The inspector verified that the ISFSI pad and the Vertical Concrete Casks (VCCs) in storage on the pad were in good material condition and that all radiological postings were visible and in good repair. The inspector observed remote and roving surveillance of the security fence and confirmed adequate security barriers were present to control and prevent unauthorized intruders. Operations personnel performed routine surveillances on the spent fuel in accordance with TS requirements. The inspector verified that appropriate data for the VCC Thermal Monitoring Program (TS 5.4.1) were collected and logged in the control room. Roles and responsibilities for the maintenance of the ISFSI were discussed with day and night shift control room Certified Fuel Handlers and security personnel. No safety concerns were identified.

c. Conclusions

The licensee maintained an adequate program to survey and monitor the ISFSI and stored spent fuel.

O1.5 Quality Assurance Audits and Self Assessments

a. Inspection Scope (40801)

A review was performed to evaluate the effectiveness of licensee controls in identifying, resolving, and preventing issues that degrade safety or the quality of decommissioning. The inspector evaluated the licensee's self-assessment, auditing, corrective actions, and root cause evaluations through a review of licensee documents and interviews with licensee personnel. Qualifications of personnel performing audits of licensee activities were examined.

b. Observations

The inspector reviewed selected Quality Assurance (QA) Audits and Quality Surveillance Reports (QSRs) of licensee activities :

Audit No. Y-03-A-11-01	Chemistry/Radiation Protection Program, Process Control Program, and Radwaste Program Audit
Audit No. Y-03-A-06-01	Operations/Maintenance/Spent Fuel Pool Makeup Monitoring Program/Compliance of Operations to Tech Specs and Fire Protection
QSR-03-108-YR	Assessment of the License Termination Plan (LTP) Project
QSR-03-109-YR	Readiness Review DEMCO Demolition
QSR-03-110-YR	Final Acceptance/Turnover of Records Generated by NAC
QSR-03-111-YR	Follow-up Assessment of DEMCO Readiness

Audits performed by the licensee were thorough and detailed, with adequate management attention to effect timely resolution of issues. The inspector determined that an appropriate threshold for initiating a condition report (CR) exist.

During this inspection period, the licensee contracted demolition activities to DEMCO, which established an onsite Quality Assurance Department. The inspector reviewed qualifications of the DEMCO Onsite QA Manager and the DEMCO QA inspector. The individuals responsible for implementing the onsite DEMCO QA program as detailed in a letter dated July 3, 2003, from the President of DEMCO are experienced in the nuclear industry and meet the requirements to adequately perform audits. The inspector reviewed selected field surveillances completed by DEMCO and no safety concerns were identified.

The inspector discussed with the licensee their program for self-identifying problems and reviewed selected CRs. Approximately 430 CRs were initiated during this inspection period. The threshold for identifying problems and initiating a CR appears to be at an appropriate level. The inspector discussed with licensee management the apparent trend that most CRs were being written by contract supervisors and licensee personnel and not front line laborers, as was the case with the previous contractor for fuel transfer to the ISFSI. Licensee management stated that they also identified a similar trend and are actively working toward lower level site personnel identifying issues through meeting with the craft. No safety concerns were identified.

c. Conclusions

The licensee and their contractor DEMCO maintained effective corrective action programs and performed audits and assessments to help self-identify and correct issues and problems.

S1 Conduct of Security and Safeguards

S1.1 Interim Compensatory Measures at Decommissioning Nuclear Power Plants

a. Inspection Scope (Temporary Instruction (TI) 2561/004)

The inspector reviewed YAEC's implementation of ICMs for safeguards and security measures at their Yankee Rowe facility. Review areas included staffing and armament, protective strategies, and access control procedures and equipment. Information was gathered through a review of procedures and documents, tours of the site, and through interviews with cognizant personnel.

b. Observations and Findings

On May 23, 2002, the licensee was issued an Order for implementation of interim safeguards and security compensatory measures for their Yankee Rowe facility. This Order was applicable to nuclear power plants with spent fuel in their SFP. Spent fuel was contained in the SFP until the last spent fuel cask was removed on May 27, 2003, and transferred to the ISFSI on May 31, 2003. The inspector reviewed the licensee's responses to the May 23, 2002, Order contained in letters dated June 12, 2002, August 8, 2002, and November 18, 2002, which all contain Safeguards Information. The inspector examined physical features of the site, including installed equipment for access control, and reviewed plans and procedures for compliance with the ICMs. The inspector discussed security force staffing, training, armament, equipment, and protective strategies with cognizant security staff. Activities for general site access, spent fuel pool access, and onsite and offsite communications were directly observed by the inspector. During an earlier inspection (Report No. 50-029/2003-001, August 7, 2003, ADAMS accession No. ML032190487) the inspector verified that the licensee's defueled Emergency Preparedness program had been adequately revised to address the security interface requirements associated with the ICMs. No concerns were identified.

On October 16, 2002, the licensee was issued an Order for implementation of ICMs at a stand alone ISFSI. The inspector reviewed the licensee's responses to the October 16, 2002, Order contained in letters dated November 5, 2002, and September 9, 2003, which contain Safeguards Information. The inspector conducted a similar evaluation of the effectiveness of ICM implementation measures for this ISFSI Order. The inspector verified that modifications to the security program were made in accordance with requirements of the Order. No concerns were identified.

c. Conclusion

YAEC effectively implemented their security program for compliance with the May 23, 2002, Order for ICMs for safeguards and security for Yankee Rowe while spent fuel was stored in the SFP. Following removal of all spent fuel from the SFP, the licensee effectively complied with the October 16, 2002, Order applicable to operational, stand alone ISFSIs. No findings of significance were identified.

II. Plant Support and Radiological Controls

R1 Radiological Protection and Chemistry Controls

R1.1 External Exposure Controls

a. Inspection Scope (83750)

The inspection included touring the RCA and reviewing current radiological surveys of various work locations to determine the adequacy of the licensee's occupational program to monitor and control external radiation exposure to employees. The impact of the relocation of the RCA access control point on the radiation protection program was evaluated.

b. Observations

During tours of the facility, the inspector observed that all areas in the RCAs were appropriately posted and labeled for radioactive materials. The inspector also noted that there were only three areas within the RCAs that were required to be posted as high radiation areas. Posting and labeling of radioactive materials and radiation areas continued to meet regulatory requirements. Portal monitors and frisking instruments were located in the facility for use by workers as they left radiation areas or contaminated areas.

During this inspection period, the control point providing entry into and egress from the RCA was relocated due to the demolition of the service building annex. The inspector reviewed the plan for relocation with radiation protection department personnel to ensure that proper access controls were implemented during the move. The licensee completed the relocation of equipment during low personnel traffic times. The inspector reviewed logs of manual RCA entries during the relocation period and verified proper management of personnel and tools entering and exiting the RCA. The inspector examined calibration records of whole body friskers, counting room equipment, dosimetry equipment, the whole body counter, and respirator fit testing equipment and verified that following the relocation, all required radiation protection and chemistry instrumentation was properly brought back into service. No safety concerns were identified.

c. Conclusions

The licensee has provided good controls to limit exposures of workers to external sources of radiation. The relocation of the RCA control point was completed in a safe manner and in accordance with regulatory requirements. Radiation protection and chemistry laboratory equipment were adequately recalibrated and returned to service following the relocation.

R1.2 Spent Fuel Pool Draindown

a. Inspection Scope (60801 and 84750)

The inspector reviewed activities associated with the draindown and release of the SFP water.

b. Observations

Following transfer of spent fuel and greater than class C (GTCC) waste to the ISFSI (Inspection Report No. 50-029/2003-001, August 7, 2003, ADAMS accession No. ML032190487), the licensee proceeded with removal of structures from the pool, decontamination of the pool surfaces, and filtering of the water so that it could be released to the Deerfield River. In order to discharge SFP water in an efficient manner, regulatory relief from the State's NPDES permit (high boron concentration) was required. In addition, a temporary system was needed to drain and process the water and discharge the water via the Auxiliary Service Water System (ASW). The inspector reviewed Design Change Request (DCR) YA-DCR-00-003-03, "Spent Fuel Pit Water Discharge", and the accompanying 10CFR50.59 safety evaluation. The temporary discharge system instructions for use were detailed and adequate to ensure compliance with regulatory requirements. The licensee received a revised NPDES permit dated July 24, 2003, which contained restrictions on flow rates, boron concentrations, pH ranges, and total suspended solids. Operating procedure OP-2000.358, Processing the Spent Fuel Pool", adequately addressed ODCM and NPDES limits of operation.

The inspector reviewed liquid discharge permits (Nos. 03-03 thru 03-27) which controlled the release. Approximately 150,000 gallons of water was released from the SFP to the Deerfield River between August 20 and October 30, 2003. The inspector reviewed records which indicated that a total of $1.24\text{E-}1$ curies of Tritium and $4.68\text{E-}5$ curies of gamma emitters was released. The estimated offsite dose calculated in accordance with ODCM was less than $1.0\text{E-}4$ mrem. The inspector discussed with licensee representatives controls established to ensure compliance with discharge permit requirements and verified by review of discharge documents that no NPDES criteria were exceeded. No safety concerns were identified.

During the draindown, the inspector observed hydrolazing operations to clean the pool walls and discussed controls established to ensure that the filters could be shipped as low level radwaste. During the draindown and filtering process, higher than normal dose rates were identified on the High Dirt Load Filter (HDLF). A CR was written to document the higher than expected dose rates and the inspector reviewed the root cause determination for this event. The analysis was detailed and thorough. The condition was a result of vacuuming loose items of high specific activity that were not anticipated to be in the SFP. The source of the material was debris released or redistributed to the SFP during removal of the Filter Demineralizer. No worker was exposed to radiation in excess of administrative limits. The inspector reviewed surveys of the pool surfaces and noted that the pool is posted as a high radiation area.

Adequate procedures were established to control entries into the drained pool. No safety concerns were identified.

c. Conclusions

The licensee completed processing and drain down of the SFP water in a safe and compliant manner. Discharge of the SFP water to the Deerfield River was conducted in accordance with NPDES and ODCM requirements.

R1.3 Independent Confirmatory Survey of Turbine Building

a. Inspection Scope (83801 and 83750)

The inspector reviewed licensee activities associated with the survey and release of the turbine building for unrestricted use. An independent confirmatory survey was performed by ORISE.

b. Observations

Region I contracted with ORISE to perform independent measurements and sampling of the turbine building to support its release for unrestricted use and disposal in a local landfill. The licensee previously had completed detailed surveys of the building in accordance with the requirements of NUREG/CR-5849, "Manual for Conducting Radiological Surveys in Support of License Termination", June, 1992. In addition, the licensee more recently resurveyed the structure. The inspectors reviewed available survey measurements and performed a visual inspection of the facility. On September 24 and 25, 2003, health physics personnel from ORISE conducted on site measurements and collected samples from the turbine building for off site laboratory analysis. Approximately 25% of the surface walls and floors in the building were scanned for total beta/gamma radiation. Direct measurements for total beta/gamma activity were taken at 152 locations that were identified by surface scans or corresponded to licensee measurement areas. Samples were analyzed by ORISE in their laboratory in Oak Ridge, TN. Results of the independent confirmatory survey were received in Region I on January 2, 2004. The final report can be found in ADAMS (accession No. ML040360630). The results of the independent survey confirmed that the radiological conditions of the turbine building met the site criteria for release for unrestricted use (IE Circular 81-07). No safety concerns were identified.

The ORISE team also independently surveyed portions of the service building annex and rubble that was the turbine building office area. Surface scans and samples were taken in the same manner as used for the turbine building survey. Results of these measurements are reported in the same final confirmatory survey report in ADAMS (accession No. ML040360630). The results of the surveys of the service building annex and the turbine building office area rubble confirm that their radiological conditions satisfy the site criteria for release for unrestricted use. No safety concerns were identified.

c. Conclusions

Results of independent measurements performed by ORISE confirm that the radiological conditions of the turbine building, service building annex, and the turbine building office area rubble satisfy the criteria for release of these structures for unrestricted use.

III. MANAGEMENT MEETINGS

X1 Exit Meeting Summary

The inspector presented the inspection results to members of licensee management periodically during the inspection, and during an exit meeting with Mr. B. Wood and others on January 21, 2004. The licensee acknowledged the findings presented by the inspector. The inspector reviewed with the licensee whether any materials examined during the inspection should be considered proprietary. While proprietary information was reviewed during the inspection, no proprietary information is contained in this report.

X2 Other Meetings

Dr. Ronald Bellamy, Chief, Decommissioning Branch, NRC Region I, and John Wray, NRC Region I inspector attended a Community Advisory Board (CAB) meeting on January 21, 2004. Approximately 30 members of the public and the CAB were in attendance. Mr. Wray provided an update of NRC activities and discussed results of inspection 2003-001. Dr. Bellamy provided an overview of the LTP approval process and the status of the NRC review of the plan.

PARTIAL LIST OF PERSONS CONTACTED

*G. Babineau, YAEC, Safety Oversight Manager
W. Blackadar, YAEC, Radiation Protection Oversight
D. Calsyn, YAEC, Quality Assurance Manager
J. Darman, YAEC, Radiological Engineer
R. Dee, YAEC, Scheduler
*S. Garvie, YAEC, Security Supervisor
E. Heath, YAEC, Asst. Safety Oversight Manager
R. Kacich, President, YAEC
*J. Kay, YAEC-Regulatory Affairs
C. Lloyd, YAEC, Lead QA Engineer
K. LaDuke, YAEC, QA Auditor
*G. Maret, Vice President of Decommissioning, YAEC
*K. Myers, Health Physics Manager. DEMCO
*D. Montt, HP and Chemistry Supervisor, YAEC
M. Niehoff, Project Director, DEMCO
*N. Rademacher, YAEC Decommissioning Project Manager
S. Racz, YAEC, Sr. Technical Specialist-QA
W. Schaab, Vice President-Operations, DEMCO
K. Smith, YAEC, Communications Manager
M. Vandale, YAEC Oversight
*F. Williams, YAEC, Plant Superintendent
*B. Wood, YAEC, Site Manager

* These individuals participated in the exit briefing held on January 21, 2004.

LIST OF ACRONYMS

ADAMS	Agency Document Access and Management System
ASW	Auxiliary Service Water
CAB	Community Advisory Board
CFR	Code of Federal Regulations
CR	Condition Report
DCR	Design Change Request
EP	Emergency Preparedness
GTCC	Greater Than Class C
HDLF	High Dirt Load Filter
ICM	Interim Compensatory Measures
ISFSI	Independent Spent Fuel Storage Installation
LTP	License Termination Plan
NCV	Non-cited Violation
NPDES	National Pollutant Discharge Elimination System
ODCM	Offsite Dose Calculations Manual
PDR	Public Document Room
QA	Quality Assurance
QSR	Quality Surveillance Reports
RCA	Radiological Controlled Area
SFP	Spent Fuel Pool
TI	Temporary Instruction
VCC	Vertical Concrete Cask
VEMA	Vermont Emergency Management Agency
YAEC	Yankee Atomic Electric Company
YNPS	Yankee Nuclear Power Station

INSPECTION PROCEDURES USED

IP 40801	Self Assessment, Auditing, and Corrective Action
IP 60801	Spent Fuel Pool Safety
IP 60855	Operation of ISFSI
IP 62801	Maintenance and Surveillance
IP 71714	Cold Weather Preparations
IP 71801	Decommissioning Performance and Review Status
IR 83801	Inspection of Final Surveys at Permanently Shutdown Reactor Facilities
IP 83750	Occupational Radiation Exposure Control
IP 84750	Radioactive Waste Treatment and Effluent & Environmental Monitoring
TI 2561/004	Interim Compensatory Measures at Decommissioning Nuclear Power Plants

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

NONE

Closed

NONE

Discussed

NONE