

### 3.3 INSTRUMENTATION

#### 3.3.4.1 End of Cycle Recirculation Pump Trip (EOC-RPT) Instrumentation

- LCO 3.3.4.1      a. Two channels per trip system for each EOC-RPT instrumentation Function listed below shall be OPERABLE:
1. Turbine Stop Valve (TSV) - Closure; and
  2. Turbine Control Valve (TCV) Fast Closure, Trip Oil Pressure - Low.

OR

- b. LCO 3.2.2, "MINIMUM CRITICAL POWER RATIO (MCPR)," limits for Inoperable EOC-RPT as specified in the COLR are made applicable; and
- c. LCO 3.2.3, "LINEAR HEAT GENERATION RATE (LHGR)," limits for Inoperable EOC-RPT as specified in the COLR are made applicable.

APPLICABILITY:    THERMAL POWER  $\geq$  30% RTP.

ACTIONS

-----NOTE-----

Separate Condition entry is allowed for each channel.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more channels Inoperable.	A.1 Restore channel to OPERABLE status.	72 hours
	<u>OR</u> A.2 -----NOTE----- Not applicable if inoperable channel is the result of an inoperable breaker. ----- Place channel in trip.	72 hours
B. One or more Functions with EOC-RPT trip capability not maintained.  <u>AND</u>  MCPR and LHGR limit for inoperable EOC-RPT not made applicable.	B.1 Restore EOC-RPT trip capability.	2 hours
	<u>OR</u> B.2 Apply the MCPR and LHGR limit for Inoperable EOC-RPT as specified in the COLR.	2 hours
C. Required Action and associated Completion Time not met.	C.1 Reduce THERMAL POWER to < 30% RTP.	4 hours

### 3.7 PLANT SYSTEMS

#### 3.7.5 Main Turbine Bypass System

LCO 3.7.5            The Main Turbine Bypass System shall be OPERABLE.

OR

The following limits are made applicable:

- a. LCO 3.2.1, "AVERAGE PLANAR LINEAR HEAT GENERATION RATE (APLHGR)," limits for an inoperable Main Turbine Bypass System, as specified in the COLR; and
- b. LCO 3.2.2, "MINIMUM CRITICAL POWER RATIO (MCPR)," limits for an inoperable Main Turbine Bypass System, as specified in the COLR; and
- c. LCO 3.2.3, "LINEAR HEAT GENERATION RATE (LHGR)," limits for an inoperable Main Turbine Bypass System, as specified in the COLR.

APPLICABILITY:    THERMAL POWER  $\geq$  25% RTP.

#### ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Requirements of the LCO not met.	A.1    Satisfy the requirements of the LCO.	2 hours
B. Required Action and associated Completion Time not met.	B.1    Reduce THERMAL POWER to < 25% RTP.	4 hours

## 5.6 Reporting Requirements (continued)

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### 5.6.4 Monthly Operating Reports

Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis no later than the 15th of each month following the calendar month covered by the report.

### 5.6.5 CORE OPERATING LIMITS REPORT (COLR)

- a. Core operating limits shall be established prior to each reload cycle, or prior to any remaining portion of a reload cycle, and shall be documented in the COLR for the following:
  - (1) The APLHGRs for Specification 3.2.1;
  - (2) The LHGR for Specification 3.2.3;
  - (3) The MCPR Operating Limits for Specification 3.2.2; and
  - (4) The RBM setpoints and applicable reactor thermal power ranges for each of the setpoints for Specification 3.3.2.1, Table 3.3.2.1-1.
- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:
  1. NEDE-24011-P-A, General Electric Standard Application for Reactor Fuel.
  2. XN-NF-81-58(P)(A), RODEX2 Fuel Rod Thermal-Mechanical Response Evaluation Model.
  3. XN-NF-85-67(P)(A), Generic Mechanical Design for Exxon Nuclear Jet Pump BWR Reload Fuel.
  4. EMF-85-74(P)(A), RODEX2A (BWR) Fuel Rod Thermal-Mechanical Evaluation Model.
  5. ANF-89-98(P)(A), Generic Mechanical Design Criteria for BWR Fuel Designs.

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## 5.6 Reporting Requirements (continued)

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6. XN-NF-80-19(P)(A) Volume 1, Exxon Nuclear Methodology for Boiling Water Reactors - Neutronic Methods for Design and Analysis.
7. XN-NF-80-19(P)(A) Volume 4, Exxon Nuclear Methodology for Boiling Water Reactors: Application of the ENC Methodology to BWR Reloads.
8. EMF-2158(P)(A), Siemens Power Corporation Methodology for Boiling Water Reactors: Evaluation and Validation of CASMO-4/MICROBURN-B2.
9. XN-NF-80-19(P)(A) Volume 3, Exxon Nuclear Methodology for Boiling Water Reactors, THERMEX: Thermal Limits Methodology Summary Description.
10. XN-NF-84-105(P)(A) Volume 1, XCOBRA-T: A Computer Code for BWR Transient Thermal-Hydraulic Core Analysis.
11. ANF-524(P)(A), ANF Critical Power Methodology for Boiling Water Reactors.
12. ANF-913(P)(A) Volume 1, COTRANSA2: A Computer Program for Boiling Water Reactor Transient Analyses.
13. ANF-1358(P)(A), The Loss of Feedwater Heating Transient in Boiling Water Reactors.
14. EMF-2209(P)(A), SPCB Critical Power Correlation.
15. EMF-2245(P)(A), Application of Siemens Power Corporation's Critical Power Correlations to Co-Resident Fuel.
16. EMF-2361(P)(A), EXEM BWR-2000 ECCS Evaluation Model.
17. EMF-2292(P)(A), ATRIUM<sup>TM</sup>-10: Appendix K Spray Heat Transfer Coefficients.

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