

December 15, 2003

Mr. D. M. Jamil  
Vice President  
Catawba Nuclear Station  
Duke Energy Corporation  
4800 Concord Road  
York, South Carolina 29745

SUBJECT: CATAWBA NUCLEAR STATION, UNITS 1 AND 2 RE: REQUEST FOR  
ADDITIONAL INFORMATION (TAC NOS. MC0498 AND MC0499)

Dear Mr. Jamil:

By letter dated August 19, 2003, you submitted an application for amendment of the Technical Specifications (TS) for the Catawba Nuclear Station, Units 1 and 2, to revise the TS requirements for the Containment Pressure Control System. The U. S. Nuclear Regulatory Commission technical staff has reviewed the application and has determined that additional information is required, as identified in the Enclosure.

We discussed these issues with your staff on December 11, 2003. Your staff indicated that you would attempt to provide your response by January 15, 2004.

Please contact me at (301) 415-1842, if you have any other questions on these issues.

Sincerely,

**/RA/**

Sean E. Peters, Project Manager, Section 1  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosure: As stated

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION

DUKE POWER COMPANY

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-413 AND 50-414

The Nuclear Regulatory Commission (NRC) staff has reviewed the licensee's submittal dated August 19, 2003, regarding proposed changes to the Technical Specifications for the Containment Pressure Control System (CPCS). The NRC staff has identified the following information that is needed to enable the continuation of its review.

1. Title 10 of the *Code of Federal Regulations*, Section 50.36(c)(1)(ii)(A), defines limiting safety system setting (LSSS) as a setting that must be so chosen that automatic protective action will correct the abnormal situation before a safety limit is exceeded. The improved technical specifications (TS) Bases define the allowable value (AV) to be equivalent to the LSSS and defines that a channel is operable if the trip setpoint is found not to exceed the AV during channel operational testing. Any request for safety-related instrument setpoint modification should provide a reference to the NRC approved setpoint methodology for the licensed plant as the basis for the modification. Provide the setpoint methodology reference for this TS amendment request. If you use method 3 specified in ISA S67.04.02, then confirm that a check calculation is performed to account for all loop uncertainty not measured during the Channel Operational Test/Channel Functional Test. Provide a sample calculation to demonstrate this.
2. In the submittal dated August 19, 2003, Attachment 3, Page 3, it states that the proposed solution to the CPCS circuit fluctuation problem will: 1) widen the deadband for the CPCS start permissive, and 2) narrow the span viewed by the CPCS pressure instrument. The proposed CPCS start permissive AV changed from  $\leq 0.45$  psid to  $\leq 1.0$  psid, and the Nominal Trip Setpoint changed from 0.4 psid to 0.9 psid. Additionally, the CPCS termination Nominal Trip Setpoint changed from 0.3 psid to 0.35 psid. Provide a summary of the setpoint calculation to demonstrate that the instrument uncertainties of the new pressure transmitters for the CPCS start permissive circuit will not affect any of the safety significant functions (i.e. to start containment spray), and the termination uncertainties will not cause the containment pressure to fall to the negative region.
3. Please discuss any effects that the proposed changes will have on the containment safety analyses. If there are no changes to the containment safety analyses, please explain why this is so.

Enclosure

Catawba Nuclear Station

cc:

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Catawba Nuclear Station

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