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November 7, 2003

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555
Attn: Mr. Robert Clark (Mail Stop O-8-E9)
Project Directorate I-1

Subject: Revision to Emergency Plan Implementing Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Gentlemen:

In accordance with 10 CFR 50.4(b)(5), enclosed is a revision to a Ginna Station Emergency Plan Implementing Procedure (EPIP).

We have determined, per the requirements of 10 CFR 50.54(q), that the procedure change does not decrease the effectiveness of our Nuclear Emergency Response Plan.

Very truly yours,

Richard J. Watts
Manager, Nuclear Training Department

Enclosures

xc: USNRC Region 1 (2 copies of letter and 2 copies of each procedure)
Resident Inspector, Ginna Station (1 copy of letter and 1 copy of each procedure)
RG&E Nuclear Safety and Licensing (1 copy of letter)
Dr. Robert C. Mecredy (2 copies of letter only)

RJW/jtw

A045

U.S. Nuclear Regulatory Commission
November 7, 2003
Page 2 of 2

PROCEDURE

EPIP 2-5

REVISION NUMBER

15

INPUT PARAMETERS: TYPE: PREPIP STATUS VALUE(S): EF 5 YEARS ONLY:

PREPIP EMERGENCY PLAN IMPLEMENTING PROCEDURE

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
EPIP-1-0	GINNA STATION EVENT EVALUATION AND CLASSIFICATION	032	09/22/2003	06/20/2003	06/20/2008	EF
EPIP-1-1	UNUSUAL EVENT	004	05/23/2003	05/23/2003	05/23/2008	EF
EPIP-1-2	ALERT	004	11/02/2001	11/02/2001	11/02/2006	EF
EPIP-1-3	SITE AREA EMERGENCY	005	12/09/1996	04/09/2003	04/09/2008	EF
EPIP-1-4	GENERAL EMERGENCY	006	05/05/2003	05/05/2003	05/05/2008	EF
EPIP-1-5	NOTIFICATIONS	056	09/22/2003	09/22/2003	09/22/2008	EF
EPIP-1-6	SITE EVACUATION	017	06/20/2003	06/20/2003	06/20/2008	EF
EPIP-1-7	ACCOUNTABILITY OF PERSONNEL	009	11/02/2001	11/02/2001	11/02/2006	EF
EPIP-1-8	SEARCH AND RESCUE OPERATION	006	05/23/2003	05/23/2003	05/23/2008	EF
EPIP-1-9	TECHNICAL SUPPORT CENTER ACTIVATION	025	08/08/2003	08/08/2003	08/08/2008	EF
EPIP-1-10	OPERATIONAL SUPPORT CENTER (OSC) ACTIVATION	013	05/23/2003	05/23/2003	05/23/2008	EF
EPIP-1-11	SURVEY CENTER ACTIVATION	029	02/25/2003	02/25/2003	02/25/2008	EF
EPIP-1-12	REPAIR AND CORRECTIVE ACTION GUIDELINES DURING EMERGENCY SITUATIONS	009	12/20/2001	12/20/2001	12/20/2006	EF
EPIP-1-13	LOCAL RADIATION EMERGENCY	005	10/14/2003	10/14/2003	10/14/2008	EF
EPIP-1-15	USE OF THE HEALTH PHYSICS NETWORK HPN	005	04/24/1996	03/03/1999	03/03/2004	EF
EPIP-1-16	RADIOACTIVE LIQUID RELEASE TO LAKE ONTARIO OR DEER CREEK	005	02/25/2003	02/25/2003	02/25/2008	EF
EPIP-1-17	PLANNING FOR ADVERSE WEATHER	004	05/23/2003	05/23/2003	05/23/2008	EF
EPIP-1-18	DISCRETIONARY ACTIONS FOR EMERGENCY CONDITIONS	006	05/05/2003	05/05/2003	05/05/2008	EF
EPIP-2-1	PROTECTIVE ACTION RECOMMENDATIONS	021	08/08/2003	08/08/2003	08/08/2008	EF
EPIP-2-2	OBTAINING METEOROLOGICAL DATA AND FORECASTS AND THEIR USE IN EMERGENCY DOSE AS	013	12/03/2002	12/03/2002	12/03/2007	EF
EPIP-2-3	EMERGENCY RELEASE RATE DETERMINATION	015	07/01/2002	07/01/2002	07/01/2007	EF
EPIP-2-4	EMERGENCY DOSE PROJECTIONS - MANUAL METHOD	014	08/08/2003	08/08/2003	08/08/2008	EF
EPIP-2-5	EMERGENCY DOSE PROJECTIONS PERSONAL COMPUTER METHOD	015	11/07/2003	11/07/2003	11/07/2008	EF
EPIP-2-6	EMERGENCY DOSE PROJECTIONS - MIDAS PROGRAM	013	10/08/2003	10/08/2003	10/08/2008	EF
EPIP-2-7	MANAGEMENT OF EMERGENCY SURVEY TEAMS	011	08/09/2002	08/09/2002	08/09/2007	EF
EPIP-2-8	VOLUNTARY ACCEPTANCE OF EMERGENCY RADIATION EXPOSURE	006	09/22/2003	09/22/2003	09/22/2008	EF
EPIP-2-9	ADMINISTRATION OF POTASSIUM IODIDE (KI)	008	05/23/2003	05/23/2003	05/23/2008	EF
EPIP-2-10	INPLANT RADIATION SURVEYS	004	08/09/2002	08/09/2002	08/09/2007	EF
EPIP-2-11	ONSITE SURVEYS	019	05/15/2002	05/15/2002	05/15/2007	EF
EPIP-2-12	OFFSITE SURVEYS	022	05/15/2002	05/15/2002	05/15/2007	EF

INPUT PARAMETERS: TYPE: PREP STATUS VALUE(S): EF 5 YEARS ONLY:

PREP EMERGENCY PLAN IMPLEMENTING PROCEDURE

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
EPIP-2-13	IODINE AND PARTICULATE ACTIVITY DETERMINATION FROM AIR SAMPLES	008	07/27/1999	07/27/1999	07/27/2004	EF
EPIP-2-14	POST PLUME ENVIRONMENTAL SAMPLING	015	10/08/2002	10/08/2002	10/08/2007	EF
EPIP-2-15	POST PLUME EVALUATION OF OFFSITE DOSES DUE TO DEPOSITION	006	10/08/2002	10/08/2002	10/08/2007	EF
EPIP-2-16	CORE DAMAGE ESTIMATION	013	12/03/2002	12/03/2002	12/03/2007	EF
EPIP-2-17	HYPOTHETICAL (PRE-RELEASE) DOSE ESTIMATES	008	09/22/2003	09/22/2003	09/22/2008	EF
EPIP-2-18	CONTROL ROOM DOSE ASSESSMENT	015	05/23/2003	05/23/2003	05/23/2008	EF
EPIP-3-1	EMERGENCY OPERATIONS FACILITY (EOF) ACTIVATION AND OPERATIONS	024	10/08/2003	10/08/2003	10/08/2008	EF
EPIP-3-2	ENGINEERING SUPPORT CENTER (ESC)	010	08/09/2002	08/09/2002	08/09/2007	EF
EPIP-3-3	IMMEDIATE ENTRY	010	08/08/2003	08/08/2003	08/08/2008	EF
EPIP-3-4	EMERGENCY TERMINATION AND RECOVERY	009	02/25/2003	02/25/2003	02/25/2008	EF
EPIP-3-7	SECURITY DURING EMERGENCIES	010	10/08/2002	10/08/2002	10/08/2007	EF
EPIP-4-1	PUBLIC INFORMATION RESPONSE TO AN UNUSUAL EVENT	007	08/08/2003	08/08/2003	08/08/2008	EF
EPIP-4-3	ACCIDENTAL ACTIVATION OF GINNA EMERGENCY NOTIFICATION SYSTEM SIRENS	013	08/08/2003	08/08/2003	08/08/2008	EF
EPIP-4-6	JOINT EMERGENCY NEWS CENTER ACTIVATION	009	08/31/2001	08/31/2001	08/31/2006	EF
EPIP-4-7	PUBLIC INFORMATION ORGANIZATION STAFFING	023	05/23/2003	05/23/2003	05/23/2008	EF
EPIP-4-8	SILENT TESTING OF THE GINNA SIRENS FROM THE TECHNICAL SUPPORT CENTER	001	02/25/2003	02/25/2003	02/25/2008	EF
EPIP-4-9	ACTIVATION OF GINNA EMERGENCY SIRENS FROM THE TECHNICAL SUPPORT CENTER	002	05/08/2003	05/08/2003	05/08/2008	EF
EPIP-4-10	SILENT TESTING OF THE GINNA SIRENS FROM THE COUNTY ACTIVATION POINTS	000	02/25/2003	02/25/2003	02/25/2008	EF
EPIP-4-11	ACTIVATION OF THE GINNA SIRENS FROM THE COUNTY ACTIVATION POINTS	001	05/08/2003	05/08/2003	05/08/2008	EF
EPIP-5-1	OFFSITE EMERGENCY RESPONSE FACILITIES AND EQUIPMENT PERIODIC INVENTORY CHECKS AND	028	06/20/2003	06/20/2003	06/20/2008	EF
EPIP-5-2	ONSITE EMERGENCY RESPONSE FACILITIES AND EQUIPMENT PERIODIC INVENTORY CHECKS AND	031	06/20/2003	06/20/2003	06/20/2008	EF
EPIP-5-5	CONDUCT OF DRILLS AND EXERCISES	015	05/23/2003	05/23/2003	05/23/2008	EF
EPIP-5-6	ANNUAL REVIEW OF NUCLEAR EMERGENCY RESPONSE PLAN (NERP)	004	05/28/1999	05/28/1999	05/28/2004	EF
EPIP-5-7	EMERGENCY ORGANIZATION	040	05/23/2003	05/23/2003	05/23/2008	EF
EPIP-5-9	TESTING THE OFF HOURS CALL-IN PROCEDURE AND QUARTERLY TELEPHONE NUMBER CHECK	008	10/08/2003	10/08/2003	10/08/2008	EF
EPIP-5-10	EMERGENCY RESPONSE DATA SYSTEM (ERDS)	007	12/03/2002	12/03/2002	12/03/2007	EF
NERP	GINNA STATION NUCLEAR EMERGENCY RESPONSE PLAN	022	07/31/2003	07/31/2003	07/31/2005	EF

PREP TOTAL: 57

GRAND TOTAL: 57

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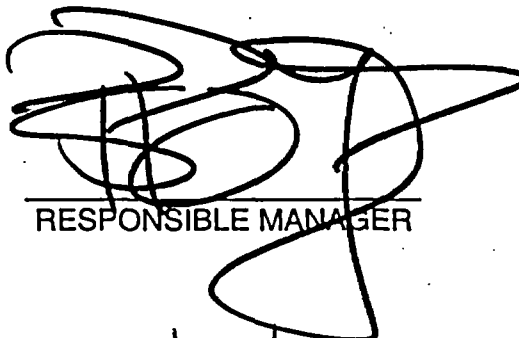
GINNA STATION

CONTROLLED COPY NUMBER 23

PROCEDURE NO. EPIP 2-5

REV. NO. 15

EMERGENCY DOSE PROJECTIONS - PERSONAL COMPUTER METHOD



RESPONSIBLE MANAGER

11/07/03

EFFECTIVE DATE

Category 1.0

This procedure contains 10

EPIP 2-5EMERGENCY DOSE PROJECTIONS - PERSONAL COMPUTER METHOD**1.0 PURPOSE**

The purpose of this procedure is to provide a personal computer-based method for performing projections of downwind dose rates and doses. Such information is needed to decide upon protective actions to be recommended to limit the exposure of the general public and emergency workers.

2.0 RESPONSIBILITY

The TSC or EOF Dose Assessment Manager is responsible for implementing this procedure.

3.0 REFERENCES**3.1 Developmental References****3.1.1 Nuclear Emergency Response Plan****3.1.2 EPA-400, Manual of Protective Action Guides and Protective Actions for Nuclear Incidents (1991)****3.1.3 Ginna UFSAR, Chapter 15****3.1.4 Regulatory Guide 1.109****3.1.5 Vertechs Corporation, EOF8, Estimated Offsite Dose Assessment Program Documentation Binder****3.2 Implementing References****3.2.1 EPIP 2-1, Protective Action Recommendations****3.2.2 EPIP 2-2, Obtaining Meteorological Data and Forecasts and Their Use in Emergency Dose Assessment****3.2.3 EPIP 2-3, Emergency Release Rate Determination****3.2.4 EPIP 2-4, Emergency Dose Projections - Manual Method****3.2.5 EPIP 1-0, Ginna Station Event Evaluation and Classification****3.2.6 EPIP 2-17, Hypothetical (Pre-Release) Dose Estimates****4.0 PRECAUTIONS**

None.

5.0 PREREQUISITES

- 5.1 The following equipment and data sources are available for use in performing computerized dose projections:
- 5.1.1 Plant Process Computer System (PPCS) - EVENT 2 Report.
 - 5.1.2 Personal Computer in TSC and EOF (for obtaining 15-minute meteorological data averages from Ginna primary weather tower).
 - 5.1.3 Back-up wind speed and direction indicators at Station 13A (accessible from TSC or EOF Personal Computer) and National Weather Service.
 - 5.1.4 Field readings from survey teams, including gamma dose rate and air sample measurements taken in the release plume.
 - 5.1.5 Personal Computer for performing dose projection routines detailed in this procedure.

DOSE ASSESSMENT USE OF GINNA COMPUTER DATA

NOTE: COMPARE ALL DOSE PROJECTIONS AGAINST EPIP 1-0, SECTION 5.2.

6.0 ACTIONS

6.1 Part "A" - Release Assessments

- 6.1.1 During a Ginna Refueling Outage, R12 could be an effluent monitor. Check with your technical support for this condition. If R12 is an effluent monitor and IS NOT on alarm or increasing, go to step 6.1.2. If R12 is an effluent monitor and IS on alarm or increasing, check:

EPIP 1-0 Section 5.1 and

R12A5 - SPING Containment Vent LOW Range Gas Monitor	<input type="checkbox"/> Yes	<input type="checkbox"/> No
R12A7 - SPING Containment Vent MID Range Gas Monitor	<input type="checkbox"/> Yes	<input type="checkbox"/> No
R12A9 - SPING Containment Vent HIGH Range Gas Monitor	<input type="checkbox"/> Yes	<input type="checkbox"/> No

Alarm

Use the SPING sheet (Attachment 1) to determine which channel to use and enter the value in the DOWNCALC program (Go to step 6.1.2).

NOTE: R15 PROBLEMS USUALLY INDICATE THAT THERE IS A TUBE IN THE STEAM GENERATOR THAT IS LEAKING RADIOACTIVE REACTOR COOLANT SYSTEM WATER INTO THE NON-RADIOACTIVE STEAM PLANT WATER.

- 6.1.2 R15 - Condenser Air Ejector Monitor. If R15 IS NOT on alarm continue to Step 6.1.3. If R15 IS on alarm check:

Alarm

EPIP 1-0 Section 5.1 and

R15A5 - SPING Condenser Air Ejector LOW Range Gas Monitor ☐ Yes ☐ No

R15A7 - SPING Condenser Air Ejector MID Range Gas Monitor ☐ Yes ☐ No

R15A9 - SPING Condenser Air Ejector HIGH Range Gas Monitor ☐ Yes ☐ No

Use the sping sheet (Attachment 1) to determine which channel to use and enter that value in the downcalc program. (Go to step 6.1.3).

NOTE: R14 PROBLEMS USUALLY INDICATE THAT THERE IS SOMETHING LEAKING IN THE AUXILIARY OR INTERMEDIATE BUILDINGS.

- 6.1.3 R14 - Plant Vent Gas Monitor. If R14 IS NOT on alarm go to step 6.1.4 If R14 IS on alarm check:

Alarm

EPIP 1-0 Section 5.1 and

R14A5 - SPING Plant Vent LOW Range Gas Monitor ☐ Yes ☐ No

R14A7 - SPING Plant Vent MID Range Gas Monitor ☐ Yes ☐ No

R14A9 - SPING Plant Vent HIGH Range Gas Monitor ☐ Yes ☐ No

Use the sping sheet (Attachment 1) to determine which channel to use and enter that value in the downcalc program. (Go to step 6.1.4).

NOTE: R31 AND R32 PROBLEMS USUALLY INDICATE THAT THERE IS A TUBE IN THE STEAM GENERATOR THAT IS LEAKING RADIOACTIVE REACTOR COOLANT SYSTEM WATER INTO THE NO-RADIOACTIVE STEAM PLANT WATER. AS LONG AS THE ARVS AND SAFETY VALVES ARE SHUT, THERE IS NO RELEASE.

- 6.1.4 R31 and R32 - "A" & "B" Steam Line Monitors. If R31 and R32 ARE NOT on alarm, go to step 6.2 for plant assessments. If R31 or R32 ARE on alarm, determine, from the event 2 printout, if any ARVs or Safety Valves are open. Also, compare R-31/32 readings to EPIP 1-0, Section 5.1. If the associated ARV or Safety Valve for the alarming monitor is open, enter the reading in the DOWNCALC program. (Go to step 6.3.) The event 2 report also lists a computer calculated 15 minute average of Ci/sec released. This can be directly entered into the DOWNCALC program also.

- 6.1.5 For unmonitored releases from containment, go to EPIP 2-17 to calculate the release rate.

6.2 Part "B" - Plant Assessments

- 6.2.1 Check R12 - Containment Gas Monitor. If R12 IS NOT on alarm or increasing then the containment atmosphere is clean of radioactivity. If R12 IS on alarm or increasing, then the Reactor Coolant System is leaking water out into the containment atmosphere. Go to step 6.2.2.

- 6.2.2 Check R9 - Letdown Monitor. If R9 IS on alarm or increasing then the Reactor Fuel is leaking into the Reactor Coolant System water. Go to Step 6.2.4.

- 6.2.3 If you have reached this step the plant and reactor, most probably, are in a stable condition and no release is occurring. If the plant is in an outage, check with your Technical Group to learn about any unusual conditions that could pose special problems.
- 6.2.4 Check R29 and R30 - Containment HIGH Range Area Monitors. Check and compare the readings. If R29 or R30 read >100 R/hr, declare a Site Area Emergency (EAL#2.3.2). If R29 or R30 read >1000 R/hr, declare a General Emergency (EAL#2.3.3). Continue to check R29 and R30 for increases due to degraded plant conditions.
- 6.3 Dose Calculations Using Personal Computer
- 6.3.1 If using the computers at RG&E, log in to corporate desktop using "User: Ginna", "Password: lakela".
- 6.3.1.1 Select the EOF8 icon to start session information.
- 6.3.1.2 Choose "new session".
- 6.3.1.3 Enter "session date" in MM/DD/YY format. Enter "session time" in HHMM format.
- 6.3.1.4 Enter your name.
- 6.3.1.5 Enter a short description of the event.
- 6.3.1.6 Enter a reactor shutdown date and time if the reactor is shutdown. If the reactor is not shutdown, do not enter any data.
- 6.3.1.7 Select the "save" button. A message should appear in the upper right portion of the screen. Click anywhere on the screen to clear the "save" message.
- 6.3.2 Downwind dose calculations
- 6.3.2.1 Select the downcalc button along the left side of the screen.
- 6.3.2.2 Review the release flowrates. Contact Operations personnel to determine current flowrates. On the initial calculation, if Operations is busy, use the normal flowrates in the program so the initial assessment is not delayed. Select the normal or emergency flowrates. Adjust the values as necessary. Select OK when done. Ensure TSC/EOF Dose Assessment and offsite responders are using the same flowrates.
- NOTE: TO SCROLL THROUGH FIELDS, PRESS THE TAB KEY.**
- 6.3.2.3 Enter the shutdown date and time if the reactor is shutdown. If the reactor is not shutdown, then press tab to scroll through these 2 fields.
- 6.3.2.4 Enter the calculation date and the calculation time.
- 6.3.2.5 To use the last saved values, click on the button labeled "use the last saved values". If new data is to be used, continue.

6.3.2.6 Enter the temperature at 250 feet.

6.3.2.7 Enter the temperature at 33 feet.

NOTE: THE WIND SPEED INDICATOR AT THE 33 FOOT LEVEL IS DESIGNED TO MEASURE ONLY TO 50 MILES PER HOUR.

6.3.2.8 Enter the wind speed at 33 feet.

NOTE: IF "WHAT IF" CALCULATIONS ARE DESIRED, REFER TO EPIP 2-17.

6.3.2.9 If a radioiodine value for the containment vent (R10A) or plant vent (R10B) has been calculated using the "ventconc" program, enter the values for the appropriate monitor. If no value has been calculated, the program will use the default radioiodine release rate based on the noble gas concentration.

6.3.2.10 Enter the value (in mR/hr) if R-31 is in alarm condition and is a release path.

6.3.2.11 Enter the value in (in mR/hr) if R-32 is in alarm condition and is a release path.

6.3.2.12 Enter value (in uCi/cc from R-12A) if R-12 is in alarm condition and is a release path.

6.3.2.13 Enter value (in uCi/cc from R-14A) if R-14 is in alarm condition.

6.3.2.14 Enter value (in uCi/cc from R-15A) if R-15 is in alarm condition and is a release path.

6.3.2.15 Enter date and time of data.

6.3.2.16 Enter exposure duration, if the release duration is known. If the release duration is unknown, enter the default of 4 hours.

NOTE: THE SITE BOUNDARY IS 0.3 MILES FROM THE REACTOR.

6.3.2.17 Enter "X" miles to 5.0. If PARs have been issued, ensure that 5 miles is adequate for evacuation. This can be changed in later assessments for other areas of interest.

6.3.2.18 Select the "save and report" button on the bottom of the screen.

6.3.2.19 Review the file name and select OK.

6.3.2.20 Review the data, then print the report and exit.

CAUTION: DO NOT PRINT THE "EMERG DATA FORM" WITH N/A OR ZEROS (0) IN ALL OF THE RELEASE POINTS.

6.3.2.22 Select "emerg data form (part II)". Report will print. Give this report to the Dose Assessment Manager. This report should be reviewed and faxed to RG&E, Wayne County, Monroe County and New York State.

- 6.3.2.23 Return to step 6.3.2.2 for more downwind calculations. To perform other calculations, select "exit" from the top of the screen.
- 6.3.3 Survey Team Data Dose Projections
- 6.3.3.1 Select the sample button from the left hand side of the screen.
- 6.3.3.2 Enter the shutdown date and time if the reactor is shutdown. If the reactor is not shutdown, then press enter to scroll through these 2 fields.
- 6.3.3.3 Enter the calculation date and the calculation time.
- 6.3.3.4 Enter the team number, color or other identification.
- 6.3.3.5 Enter the team location where the sample was obtained.
- 6.3.3.6 Enter the date and time that the sample was started.
- 6.3.3.7 Enter the date and time that the sample was stopped.
- 6.3.3.8 Enter the units of measurement of the air sample.
- 6.3.3.9 Enter the air sample flow rate at the start of the sample.
- 6.3.3.10 Enter the air sample flow rate when the sample was stopped.
- 6.3.3.11 Enter the count rate (CPM) of the iodine cartridge.
- 6.3.3.12 Enter the count rate (CPM) of the particulate filter.
- 6.3.3.13 Enter the background count rate (CPM) in the area that the sample was counted. (This should be performed outside of the plume).
- 6.3.3.14 The bottom right of the screen will show you:
- radioiodine concentration in the air (in uCi/cc)
 - radioactive particulate in the air (in uCi/cc)
 - the dose rate (in REM/hr) to a child's thyroid from the radioiodine concentration in the air.
- 6.3.3.15 Select the "save and report" button on the bottom of the screen.
- 6.3.3.16 Enter description of reason for calculation and select ok.
- 6.3.3.17 Review the data, then print the report and exit.
- 6.3.3.18 Select "exit" from the top of the screen.
- 6.3.3.19 Select the "backcalc" button along the left side of the screen.
- 6.3.3.20 Enter the shutdown date and time if the reactor is shutdown. If the reactor is not shutdown, then press enter to scroll through these 2 fields.

6.3.3.21 Enter the calculation date and the calculation time.

6.3.3.22 Enter the temperature at 250 feet.

6.3.3.23 Enter the temperature at 33 feet.

NOTE: THE WIND SPEED INDICATOR AT THE 33 FOOT LEVEL IS DESIGNED TO MEASURE ONLY TO 50 MILES PER HOUR.

6.3.3.24 Enter the wind speed at 33 feet.

6.3.3.25 Enter the team number, color or other identification.

6.3.3.26 Enter the team location where the sample was obtained.

6.3.3.27 Enter the distance (in miles) away from the Ginna Plant.

6.3.3.28 Enter the sample date and sample time.

6.3.3.29 Enter the closed window dose rate that the survey team reported at the sample location. This dose rate will be in R/hr or mR/hr. (DO NOT use the background CPM value of the radioiodine analysis. That was a different screen that you have already completed.)

6.3.3.30 Enter the radioiodine concentration (in uCi/cc) from the SAMPLE program printout.

6.3.3.31 Enter A & B points of interest.

6.3.3.32 You have now projected doses from the point where your survey team collected sample data.

6.3.3.33 Select the "save and report" button on the bottom of the screen.

6.3.3.34 Review the file name and select OK.

6.3.3.35 Review the data, then print the report and exit.

6.3.3.36 Select "report".

6.3.3.37 Select "emerg data form (part II)". Report will print. Give this report to the Dose Assessment Manager. This report should be reviewed and faxed to RG&E, Wayne County, Monroe County and New York State.

6.3.3.38 Return to step 6.3.2 for more survey team calculations. To perform other calculations, select "exit" from the top of the screen.

6.4 Use of RASCAL for determination of exposure due to field samples.

6.4.1 Start RASCAL by clicking on the icon labeled. Select "Source Term to Dose Model". Click OK.

6.4.2 Select "Event Type".

- 6.4.2.1 Select "Nuclear Power Plant Reactor" then OK.
- 6.4.3 Select "Event Location".
 - 6.4.3.1 Under Site Names select "Ginna". Modify the average the Reactor Power Level input, if appropriate, then select OK
- 6.4.4 Select "Source Term"
 - 6.4.4.1 Select "Monitored Releases - mixtures" then OK.
 - 6.4.4.2 Enter if Reactor is shutdown.
 - 6.4.4.3 Enter sample ID.
 - 6.4.4.4 Enter sample time.
 - 6.4.4.5 Enter effluent monitor reading (usually in mCi/cc).
 - 6.4.4.6 Enter effluent flowrate.
 - 6.4.4.7 Enter noble gas percentage as 99.999.
 - 6.4.4.8 Enter halogen percentage as 0.001.
 - 6.4.4.9 Select OK when all of the data has been entered.
 - 6.4.4.10 Select "Release Path".
 - 6.4.4.11 Select "not an isolated stack" and ensure release height is set at zero.
 - 6.4.4.12 Enter the release start date and time.
 - 6.4.4.13 Select "release duration" and set duration to 4 hours.
- 6.4.5 Select "Meteorology".
 - 6.4.5.1 Select "Actual Observations and Forecasts" then "Create New".
 - 6.4.5.2 Ensure Station is "GINN" then select "Enter Data".
 - 6.4.5.3 Enter the date, time, wind direction, wind speed, Stability, precipitation and 33 ft temp.
 - 6.4.5.4 Select OK.
 - 6.4.5.5 Select "Save and Process data".
 - 6.4.5.6 Enter file name to save met data (name the file using Ginna then a number , i.e., 1,2,3) then OK.
 - 6.4.5.7 Select "Return".

| 6.4.5.8 Select "OK".

6.4.6 Select "Calculate Doses"

6.4.6.1 Under Distance Calculations select

- Close-in + out to 10 miles"
- "Defaults"

6.4.6.2 Enter case description then OK.

6.4.7 Save the case using the button on the bottom left part of the screen.

6.4.8 Print Results by using the Print button on the right hand side of the screen.

7.0 ATTACHMENTS

7.1 SPING Functional Ranges

SPING FUNCTIONAL RANGES

12
R-14A9 (High Range)
15

12
R-14A7 (Mid Range)
15

12
R-14A5 (Low Range)
15

