

November 13, 2003

Mr. J. A. Scalice  
Chief Nuclear Officer and  
Executive Vice President  
Tennessee Valley Authority  
6A Lookout Place  
1101 Market Street  
Chattanooga, Tennessee 37402-2801

SUBJECT: WATTS BAR NUCLEAR PLANT, UNIT 1 — ISSUANCE OF AN AMENDMENT  
REGARDING TECHNICAL SPECIFICATION SECTION 5.0, "ADMINISTRATIVE  
CONTROLS" (TAC NO. MB7030)

Dear Mr. Scalice:

The Commission has issued the enclosed Amendment No. 49 to Facility Operating License No. NPF-90 for Watts Bar Nuclear Plant, Unit 1. The amendment consists of changes to Section 5.0 Technical Specifications (TSs), "Administrative Controls," and is in response to your application dated December 19, 2002.

The requested changes would modify TS 5.1, "Responsibility"; TS 5.2.2, "Unit Staff"; TS 5.3, "Unit Staff Qualifications"; TS 5.7.2.4, "Primary Coolant Sources Outside Containment"; TS 5.7.2.7, "Radioactive Effluent Controls Program"; TS 5.9.4, "Monthly Operating Reports"; and TS 5.11, "High Radiation Area." The proposed changes are based on the following TS Task Force (TSTF) Travelers: TSTF-258, Revision 4, "Changes to Section 5.0, Administrative Controls"; TSTF-299, Revision 0, "Administrative Controls Program 5.5.2.b Test Interval and Exception"; and TSTF-308, Revision 1, "Determination of Cumulative and Projected Dose Contributions in the Radioactive Effluent Controls Program." Additionally two editorial changes are incorporated to clarify personnel titles and shift manning requirements.

A copy of the safety evaluation is also enclosed. Notice of issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Margaret H. Chernoff, Project Manager, Section 2  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-390

Enclosures: 1. Amendment No. 49 to NPF-90  
2. Safety Evaluation

cc w/enclosures: See next page

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TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-390

WATTS BAR NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 49  
License No. NPF-90

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Tennessee Valley Authority (the licensee) dated December 19, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-90 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 49, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, and shall be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION

***/RA by B. Mozafari for/***

Allen G. Howe, Chief, Section 2  
Project Directorate II  
Division of Project Licensing Management  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 13, 2003

ATTACHMENT TO AMENDMENT NO. 49  
FACILITY OPERATING LICENSE NO. NPF-90  
DOCKET NO. 50-390

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages

5.0-1  
5.0-3  
5.0-4  
5.0-5  
5.0-11  
5.0-12  
5.0-13  
5.0-31  
5.0-37  
5.0-38  
5.0-39  
5.0-40

Insert Pages

5.0-1  
5.0-3  
5.0-4  
5.0-5  
5.0-11  
5.0-12  
5.0-13  
5.0-31  
5.0-37  
5.0-38  
5.0-39  
5.0-40  
5.0-41

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 49 TO FACILITY OPERATING LICENSE NO. NPF-90  
TENNESSEE VALLEY AUTHORITY  
WATTS BAR NUCLEAR PLANT, UNIT 1  
DOCKET NO. 50-390

1.0 INTRODUCTION

By letter dated December 19, 2002, the Tennessee Valley Authority (TVA) submitted a request for changes to the Technical Specifications (TSs) for Watts Bar Nuclear Plant (WBN), Unit 1. The amendment consists of changes to Section 5.0 of the TS, "Administrative Controls," and is in response to your application dated December 19, 2002.

The requested changes would modify TS 5.1, "Responsibility"; TS 5.2.2, "Unit Staff"; TS 5.3, "Unit Staff Qualifications"; TS 5.7.2.4, "Primary Coolant Sources Outside Containment"; TS 5.7.2.7, "Radioactive Effluent Controls Program"; TS 5.9.4, "Monthly Operating Reports"; and TS 5.11, "High Radiation Area." The proposed changes are based on the following Technical Specification Task Force (TSTF) Travelers: TSTF-258, Revision 4, Changes to Section 5.0, Administrative Controls; TSTF-299, Revision 0, Administrative Controls Program 5.5.2.b Test Interval and Exception; and TSTF-308, Revision 1, Determination of Cumulative and Projected Dose Contributions in the Radioactive Effluent Controls Program. Two additional editorial changes are proposed.

2.0 REGULATORY EVALUATION

The WBN TSs were initially developed as Improved Standard Technical Specifications (ISTSS) based on Revision 0 of NUREG-1431, "Standard Technical Specifications Westinghouse Plants," and proposed changes to the NUREG incorporated into Revision 1. The TSTF process is an industry and U.S. Nuclear Regulatory Commission (NRC, the Commission) controlled process for proposing and incorporating improvements to the ISTS. The revisions to the WBN TS proposed in this amendment impact Chapter 5.0, "Administrative Controls", and the majority of the revisions proposed in this amendment incorporate three approved TSTFs: TSTF-258, Revision 4, TSTF-299, Revision 0, and TSTF-308, Revision 1. The applicant states that the industry justification provided for NRC approval of the TSTFs are applicable to WBN and is sufficient to justify the changes.

The amendment includes two other changes to Section 5 of the TS. Both of these changes are editorial in nature. One of the changes updates the titles for operations personnel. The other editorial change clarifies staffing requirements for nonlicensed personnel.

The applicable regulatory requirements and guidelines are:

1. NUREG-1431, Revision 2, Westinghouse, Improved Standard Technical Specifications (ISTS).
2. Technical Specification Task Force Travelers: TSTF-258, Revision 4, Changes to Section 5.0, Administrative Controls.
3. TSTF-299, Revision 0, Administrative Controls Program 5.2.2b Test Interval and Exception.
4. TSTF-308, Revision 1, Determination of Cumulative and Projected Dose Contributions in the Radioactive Effluent Controls Program to 5.0 Administrative TS.
5. Title 10 of the *Code of Federal Regulations* (10 CFR) Section 20.1601(c).
6. Generic Letter 89-01, NUREG-1301 and 2 - "Offsite Dose Calculation Manual Guidance: Standard Radiological Effluent Controls for PWRs [pressurized water reactors] and BWRs [boiling water reactors]."
7. 40 CFR Part 190, "Environmental Radiation Protection Standards for Nuclear Power Operations."
8. 10 CFR Part 20, "Standards for Protection Against Radiation"
9. Regulatory Guide 8.38, "Control of Access to High and Very High Radiation Areas in Nuclear Power Plants."

### 3.0 TECHNICAL EVALUATION

#### Change Number 1: 5.1.2, "Responsibility"

The proposed TS amendment is to change the title of "Shift Operations Supervisor (SOS)" to "Shift Manager (SM)." The new title at WBN is consistent with titles used at all TVA nuclear sites, and with WBN's organization information provided in its Updated Final Safety Analysis Report. The staff finds that this change is administrative with no impact on the safety of the unit, and is, therefore, acceptable.

#### Change Number 2: 5.2.2, "Unit Staff"

1. In Section 5.2.2.a, the last sentence: "With both units shutdown or defueled, a total of three nonlicensed operators are required." is deleted. This change is consistent with NUREG-1431, Revision 2. In addition, WBN Unit 1 functions as a single unit plant. The original TS phrase "with both units shutdown" implied that WBN had two units. The staff finds that this change is administrative with no impact on the safety of the unit, and is, therefore, acceptable.



2. Section 5.2.2.b is deleted, and TS 5.2.2.c through TS 5.4.4.g are renumbered as TS 5.5.2.b through TS 5.2.2.f. This revision is to incorporate the NRC-approved changes in TSTF-258, Revision 4, in which 5.2.2.b is deleted. This TS subsection which states, "At least one licensed Reactor Operator (RO) shall be present in the control room when fuel is in the reactor. In addition, while the unit is in MODE 1, 2, 3 or 4, at least one licensed Senior Reactor Operator (SRO) shall be present in the control room" is being deleted in accordance with TSTF-258, Revision 4.

The requirements in 10 CFR 50.54(m)(2)(iii) and 50.54(k) adequately provide for shift manning. As stated in 10 CFR 50.54(m)(2)(iii), "when a nuclear power unit is in an operational mode other than cold shutdown or refueling, as defined by the unit's technical specifications, each licensee shall have a person holding a senior operator license for the nuclear power unit in the control room at all times. In addition to this senior operator, for each fueled nuclear power unit, a licensed operator or senior operator shall be present at the controls at all times." As stated in 10 CFR 50.54(k), "an operator or senior operator licensed pursuant to part 55 of this chapter shall be present at the controls at all times during the operation of the facility." All licensees are required to meet the requirements of 10 CFR 50.54(m)(2)(iii) and 50.54(k). It is not necessary to duplicate these requirements in the TSs. This is consistent with TSTF-258, Revision 4.

The NRC staff finds that the 5.2.2.b requirements will be met through compliance with the requirements of 10 CFR 50.54 (m)(2)(iii) and 50.54(k), which adequately provide for shift manning. Therefore, duplication of the requirements in TS 5.2.2.b is not necessary and the deletion of TS 5.2.2.b from the WBN TSs is acceptable.

3. Section 5.2.2.e was partially deleted and renumbered to 5.2.2.d with additional inserts from TSTF-258, Revision 4 incorporated. The specific working hour limits in existing TS Section 5.2.2.e are modified to reference administrative procedures as the means of controlling working hours. The title "health physicist" used in the TSTF was retained as "radiological controls technician" in the specification.

The NRC staff finds that the proposed TS change, which delegates the details for working hour controls to site processes, is considered an administrative change which will continue to provide reasonable assurance that impaired performance caused by excessive working hours will not jeopardize safe plant operation. Specific working hours limits are not required under 10 CFR 50.36(c)(5). Therefore, this change is consistent with TSTF-258, Revision 4, and 10 CFR 50.36(c)(5), and is acceptable.

4. Section 5.2.2.g was renumbered to 5.2.2.f and revised to replace the "Shift Technical Advisor" with the word "individual" and "Shift Operations Supervisor (SOS)" with "unit operations staff."

Shift Technical Advisors (STAs) are not used at all plants (the function may be fulfilled by one of the other on-shift individuals). TS Subsection 5.2.2.g is revised so that it does not imply that the STA and the Shift Supervisor must be different individuals. Option 1 of the Commission Policy statement on Engineering Expertise

on Shift is satisfied by assigning an individual with specified educational qualifications to each operating crew as one of the SROs (preferably the Shift Supervisor) required by 10 CFR 50.54(m)(2)(i). The wording is revised so that the STA function may be provided by either a separate individual or the individual who also fulfills another role in the shift command structure.

The NRC staff finds that the proposed TS 5.2.2.f meets the requirements of 10 CFR 50.54(m)(2)(i) and is consistent with TSTF-258, Revision 4, and is, therefore, acceptable.

Change Number 3: 5.3, "Unit Staff Qualifications"

A new Subsection 5.3.2 is added to clarify staffing requirements based on TSTF-258, Revision 4. The new TS paragraph is as follows:

For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of TS 5.3.1, perform the functions described in 10 CFR 50.54(m).

The new TS Subsection 5.3.2 would incorporate the regulatory definitions for the SRO and RO positions for the purpose of applying 10 CFR 55.4, which provides the stipulation, "Actively performing the functions of an operator or senior operator means that an individual has a position on the shift crew that requires the individual to be licensed as defined in the facility's technical specifications, and that. . . ." Adding Subsection 5.3.2 ensures that there is no misunderstanding when complying with 10 CFR 55.4 requirements.

The minimum staffing requirements stipulated in 10 CFR 50.54(m), for unit members actively performing the functions of an operator or senior operator, can be exceeded by stipulating the enhanced staffing requirements in paragraph 5.3.2. This means the site can take credit for more than the minimum number of watchstanders required by TSs, provided there are administrative controls that assure functions and duties are divided and rotated in a manner that provides each watchstander meaningful and significant opportunity to maintain proficiency in the performance of the functions of an RO and/or SRO.

The NRC staff finds that the minimum shift staffing required by 10 CFR 50.54(m) will be satisfied by appropriately licensed personnel who are actively carrying out the responsibilities of their assigned positions. The change provides the licensee with the flexibility of taking credit for more than the minimum number of watchstanders. This change is administrative in nature and is acceptable.

Change Number 4: 5.7.2.4, "Primary Coolant Sources Outside Containment"

TS 5.7.2.4.b, concerning integrated leak test requirements is revised from "at refueling cycle intervals" to "at least once per 18 months." Additionally, the sentence "The provisions of SR [Surveillance Requirement] 3.0.2 are applicable." would be added at the end of TS 5.7.2.4.

The existing TS 5.7.2.4 provides integrated leak test requirements for each system at refueling cycle intervals or less. The proposed change affects only the interval at which leak rate tests are performed. Under the proposed change, leak rate testing will be performed at 18-month intervals regardless of actual refueling cycle lengths, and if an extension of that interval becomes necessary due to scheduling considerations, the provisions of SR 3.0.2 will provide the necessary flexibility. The TS basis for SR 3.0.2 states that the 25-percent extension facilitates surveillance scheduling and considers plant operating conditions that may not be suitable for conducting the surveillance. Therefore, the maximum extension that can be applied to those portions of systems outside of containment subject to being leak tested under TS Section 5.7.2.4b would be 25 percent of 18 months or 4.5 months. The applicability of TS SR 3.0.2 must be explicitly stated in TS 5.7.2.4 since TS SR 3.0.2 only applies to the TS Limiting Condition for Operation Sections.

The NRC staff finds that specification of an 18-month frequency for leak testing is equivalent to the current nominal refueling cycle. Thus, there is no reduction in the requirement. The application of SR 3.0.2 will provide the flexibility to address cycle variations due to operational issues. Treating the testing as a surveillance requirement will result in less uncertainty and provide better administrative control. Therefore, the proposed change is acceptable.

Change No. 5: TS 5.7.2.7, "Radioactive Effluent Controls Program"

1. TS 5.7.2.7.g is being revised to use wording consistent with the terminology for dose rates contained in NUREG-1431. The staff finds that the change is administrative in nature and is in accordance with NUREG-1431. Therefore, the proposed change is acceptable.
2. TS 5.7.2.7.j is being revised to use wording consistent with the wording regarding dose commitments to the public at site boundaries with NUREG-1431. The staff finds that the change is administrative in nature and is in accordance with NUREG-1431 and 40 CFR Part 190. Therefore, the proposed change is acceptable.
3. TS 5.7.2.7 is being revised to add the following statement after TS 5.7.2.7.j: "The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Radioactive Effluent Controls Program surveillance frequency." This addition clarifies that the provisions of SRs 3.0.2 and 3.0.3 are applied to the Radiological Environmental Controls Program (RECP) surveillance frequencies of SRs 3.0.2 and 3.0.3 to allow for scheduling flexibility. SR 3.0.2 permits a 25 percent extension of the interval specified in the frequency (31 days). Allowing a 25 percent extension in the frequency of performing the monthly cumulative dose and projected dose calculation for the current quarter/year will have no affect on the outcome of the calculations.

As applied to the 31-day Frequency, SRs 3.0.2 and 3.0.3 would allow up to 31 days to complete the surveillance (dose calculation) if it is discovered that the surveillance was not performed within 38 days and 18 hours (the specified

interval plus the 25 percent extension). Allowing 31 days to complete the cumulative dose and projected dose calculation for the current quarter/year is acceptable because it will have no effect on the outcome of the calculations and has no impact on the risk associated with plant operation. In addition, operating experience has shown that the calculated dose is usually well within limits. Thus, it is considered unlikely that a potential greater time interval between dose calculations will result in inadvertent effluent releases exceeding the specified limits. TS 5.7.2.7 requires the RECP, which is contained in the Offsite Dose Calculation Manual (ODCM), to include remedial measures established in the event dose limits are exceeded.

The NRC staff finds this change is consistent with the guidance contained in Generic Letter 89-01 and NUREG-1431. Therefore, the proposed change is acceptable.

4. TS 5.7.2.7.e is revised to clarify that determination of projected dose contributions from radioactive effluents in accordance with the methodology in the ODCM is required at least once every 31 days. The existing TS could be interpreted as requiring determining projected dose contribution of the current calendar quarter and current calendar year every 31 days.

The staff finds that this change is administrative in nature and is consistent with TSTF-308, Revision 1, and is, therefore, acceptable.

#### Change Number 6: 5.9.4, "Monthly Operating Reports"

The following is deleted from TS 5.9.4:

Including documentation of all challenges to the pressurizer power operated relief valves (PORV) or pressurizer safety valves.

This revision is based on TSTF-258, Revision 4 which states that NRC Generic Letter 97-02, "Revised Contents of the Monthly Operating Report," does not specifically identify the need to report challenges to the pressurizer safety and relief valves. This information is not required for the Performance Indicator Program and, therefore, does not need to be reported.

The staff finds that the proposed change is consistent with the Generic Letter requirements and with TSTF-299, Revision 0, and is, therefore, acceptable.

#### Change No. 7: TS 5.11, "High Radiation Area"

TS 5.11 is revised in accordance with 10 CFR 20.1601(c) and updates the acceptable alternate controls to those given in 10 CFR 20.1601.

TS 5.11.1.a. is being changed from the statement that "barricades may be breached only during periods of entry or exit" to the statement that "barricades may be opened as necessary to permit entry or exit of personnel or equipment." This change is consistent with the wording in the NUREG-1431.

TS 5.11.1.c., TS 5.11.1.d.1., TS 5.11.1.d.2., and TS 5.11.1.d.4.(i) are being edited to be consistent with the wording in NUREG-1431.

TS 5.11.1.d.4.(ii) is revised to allow, in the event communications are lost between an individual worker and the Radiation Protection (RP) staff providing the remote surveillance, the worker should be able to continue to work in the area provided that the worker can communicate with other workers in the same area who are working on the same job and under the same Radiation Work Permit (RWP), and provided that the communications remain satisfactory between these workers and the RP staff providing the remote surveillance.

TS 5.11.1.e and 5.11.2.e are revised to allow any individual or group of individuals to enter a high radiation area (dose rates not exceeding 1 Rem/hr at 30 centimeters) and a high-high radiation area (dose rates greater than 1 Rem/hr at 30 centimeters) when accompanied by an individual qualified in radiation protection procedures with a radiation dose rate monitoring device. The qualified individual is responsible for providing positive control and shall perform periodic radiation surveillances at the frequency specified in the RWP. Furthermore, these continuously escorted personnel will receive a pre-job briefing prior to entry into such areas. This dose rate determination, knowledge, and pre-job briefing does not require documentation prior to initial entry. Many plants' TS requirements allow this option, which compliments the plants' practices of requiring qualified individual escort at all times during the work in a high-high radiation area. This option would provide adequate protection while (keeping with As Low As Reasonably Achievable practices) minimizing exposure to the qualified individual.

TS 5.11.2.a is revised to state "Each entryway to such an area shall be conspicuously posted as a high radiation area and shall be provided with a locked or continuously guarded door or gate. . . ." This change is consistent with Regulatory Guide 8.38 Section 2.5, which indicates that the use of a locked door or one control point where positive control over personnel entry is exercised. Posting an individual to monitor a door provides positive controls over a high-radiation area.

TS 5.11.2.a.1 is revised to indicate that the Shift Manager is only one of the many possible operations shift management positions who may be designated for the key control function.

TS 5.11.2.c., TS 5.11.2.d., and TS 5.11.2.e are being edited to be consistent with the wording in NUREG-1431.

TS 5.11.2.f is revised to delete "that is controlled as a high radiation area." The provision in 5.11.2.f has applied (in previous STS as well as ISTS NUREGs) without the added constraint of having the larger area controlled as a high radiation area. It is not always practical to control such areas as a High Radiation Area (outside of these High-High Radiation Areas).

The NRC staff finds that the proposed changes are administrative in nature and are in accordance with NUREG-1431, 10 CFR 20.1601(a), (b), and (c), and Regulatory Guide 8.38. Therefore, the NRC staff finds the proposed changes acceptable.

### 3.1 SUMMARY

NRC staff has reviewed the applicant's assessment of the regulatory requirements applicable to these changes. The staff concludes that operation of the facility in accordance with the requested changes would remain in compliance with the regulations. The method of compliance will not be altered. These changes are consistent with existing regulatory processes and procedures.

### 5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Tennessee State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 6.0 ENVIRONMENTAL CONSIDERATION

The amendment changes administrative requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Angela Chu  
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Date: November 13, 2003

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**WATTS BAR NUCLEAR PLANT**

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