

November 6, 2003

Mr. Craig G. Anderson
Vice President, Operations ANO
Entergy Operations, Inc.
1448 S. R. 333
Russellville, AR 72801

SUBJECT: ARKANSAS NUCLEAR ONE, UNIT NO. 1 - RE: RELIEF REQUEST TO USE
ALTERNATIVE TECHNIQUES FOR REACTOR PRESSURE VESSEL
CLOSURE HEAD NOZZLES (TAC NO. MB6599)

Dear Mr. Anderson:

By letter dated October 28, 2002, as supplemented by letters dated November 26 and December 16, 2002, Entergy Operations, Inc. (Entergy), submitted two requests for relief from the requirements of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code) Sections III and XI as applied to reactor pressure vessel closure head nozzles at Arkansas Nuclear One, Unit 1 (ANO-1). Specifically, in ANO1-R&R-003 and ANO1-R&R-004, Entergy proposed using an alternative temper bead welding requirement and alternatives to ASME Code non-destructive examinations and flaw evaluation requirements.

The U.S. Nuclear Regulatory Commission (NRC) staff has completed its review as documented in the enclosed safety evaluation. The NRC staff determined that Entergy has demonstrated that the proposed alternatives to the Code requirements provide an acceptable level of quality and safety and that it is impractical for the licensee to comply with the requirements. Further, the NRC staff finds that the alternatives provide reasonable assurance of structural integrity. Therefore, pursuant to 10 CFR 50.55a(a)(3)(i) and 10 CFR 50.55a(g)(6)(i), the NRC staff authorizes the relief requests for the third 10 year interval through the fall 2005 refueling outage. NRC's review and acceptance of these relief requests is limited to this time period because the licensee has scheduled to replace the reactor vessel closure head during the fall 2005 refueling outage.

Granting relief pursuant to 10 CFR 50.55a(a)(g)(6)(i) is authorized by law and will not endanger life or property or the common defense and security, and is otherwise in the public interest given due consideration to the burden upon the licensee that could result if the requirements were imposed on the facility.

In its review of the need for the reliefs during the past refueling outage, the NRC staff verbally authorized the use of ANO1-R&R-003 and ANO1-R&R-004 on November 6, 2002, due to the undue regulatory burden associated with the delay inherent in a written authorization. This letter documents our written authorization for the relief request.

Mr. Craig G. Anderson

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Pursuant to 10 CFR 2.790, we have determined the enclosed Safety Evaluation (SE) does not contain proprietary information. However, we will delay placing the SE in the public document room for five working days from the date of this letter to provide you with the opportunity to comment on the proprietary aspects only. If you believe that any information in the enclosure is proprietary, please identify such information line by line and define the basis pursuant to the criteria of 10 CFR 2.790.

Sincerely,

/RA/

Robert Gramm, Chief, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-313

Enclosure: As stated

cc w/o encl: See next page

Mr. Craig G. Anderson

- 2 -

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PDIV-1 Reading

RidsNrrDlpmLpdiv1 (HBerkow)

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RidsNrrLADJohnson

RidsOgcRp

RidsAcrcAcnwMailCenter

GHill (2)

ATKeim

Mr. Craig G. Anderson

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EVAndruskiewicz
ABoland, EDO/RIV Plants
RidsRgn4MailCenter (AHowell)

*No significant change from SE input

Accession No. ML033100287

**See previous concurrence

OFFICE	PDIV-1/PM	PDIV-1/LA	DE/EMCB	OGC	PDIV-1/SC
NAME	JMinns	DJohnson**	TChan/SCoffin*	RHoefling** w/changes	RGramm
DATE	11/1/03	11/5/03	08/18/2003	09/12/2003	11/6/03

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Arkansas Nuclear One

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