

November 4, 2003

Mr. Roy A. Anderson  
President & Chief Nuclear Officer  
PSEG Nuclear, LLC - X04  
Post Office Box 236  
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NO. 2, REQUEST FOR  
ADDITIONAL INFORMATION (RAI) RE: REQUEST FOR CHANGES TO  
LICENSE CONDITION 2.C.10 (TAC NO. MB9896)

Dear Mr. Anderson:

By letters dated June 16 and July 1, 2003, PSEG Nuclear, LLC (PSEG) submitted a request for changes to the Salem Nuclear Generating Station (Salem), Unit No. 2, Technical Specifications. The proposed changes would amend Salem, Unit No. 2, License Condition 2.C.10, to document changes to the Salem Post-Fire Safe Shutdown strategy for Fire Areas 2-FA-AB-64B, 2-FA-AB-84C, and 2-FA-AB-84B.

The U.S. Nuclear Regulatory Commission staff is reviewing your request, and has determined that additional information is necessary in order to complete its evaluation. We discussed the enclosed RAI, via telephone, with your staff on October 29, 2003. During this call, PSEG agreed to respond to the enclosed RAI within 30 days from the date of this letter. If circumstances result in the need to revise the target date, please contact me at (301) 415-1324.

Sincerely,

*/RA/*

Robert J. Fretz, Project Manager, Section 2  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-311

Enclosure: As stated

cc w/encl: See next page

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Salem Nuclear Generating Station, Unit Nos. 1 and 2

cc:

Mr. A. Christopher Bakken, III  
Senior Vice President - Site Operations  
PSEG Nuclear - X15  
P.O. Box 236  
Hancocks Bridge, NJ 08038

Mr. John T. Carlin  
Vice President - Nuclear Assessments  
PSEG Nuclear - N10  
P.O. Box 236  
Hancocks Bridge, NJ 08038

Mr. David F. Garchow  
Vice President - Eng/Tech Support  
PSEG Nuclear - N28  
P.O. Box 236  
Hancocks Bridge, NJ 08038

Mr. Gabor Salamon  
Manager - Licensing  
PSEG Nuclear - N21  
P.O. Box 236  
Hancocks Bridge, NJ 08038

Jeffrie J. Keenan, Esquire  
PSEG Nuclear - N21  
P.O. Box 236  
Hancocks Bridge, NJ 08038

Ms. R. A. Kankus  
Joint Owner Affairs  
PECO Energy Company  
Nuclear Group Headquarters KSA1-E  
200 Exelon Way  
Kennett Square, PA 19348

Lower Alloways Creek Township  
c/o Mary O. Henderson, Clerk  
Municipal Building, P.O. Box 157  
Hancocks Bridge, NJ 08038

Dr. Jill Lipoti, Asst. Director  
Radiation Protection Programs  
NJ Department of Environmental  
Protection and Energy  
CN 415  
Trenton, NJ 08625-0415

Brian Beam  
Board of Public Utilities  
2 Gateway Center, Tenth Floor  
Newark, NJ 07102

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406

Senior Resident Inspector  
Salem Nuclear Generating Station  
U.S. Nuclear Regulatory Commission  
Drawer 0509  
Hancocks Bridge, NJ 08038

## REQUEST FOR ADDITIONAL INFORMATION

RE: REQUEST FOR CHANGES TO LICENSE CONDITION 2.C.10

### SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

By letters dated June 16 and July 1, 2003, PSEG Nuclear, LLC (PSEG) submitted a request for changes to the Salem Nuclear Generating Station (Salem), Unit No. 2, Technical Specifications (TSs). The proposed changes would amend Salem, Unit No. 2, License Condition 2.C.10, to document changes to the Salem Post-Fire Safe Shutdown strategy for Fire Areas 2-FA-AB-64B, 2-FA-AB-84C, and 2-FA-AB-84B.

In Attachment 3 to its June 16, 2003, letter, PSEG credits numerous manual actions that are grouped by fire area. Additional information relating to manual actions is also provided in Attachment 8 to the June 16, 2003, submittal. The U.S. Nuclear Regulatory Commission staff has determined that additional information is necessary in order to complete its evaluation.

1. In its June 16, 2003, letter, PSEG stated that “[m]anual actions relied upon to achieve hot standby and cold shutdown are limited and are practical, reasonable and achievable under the expected environmental conditions.” Please describe how PSEG determined that the credited manual actions are practical, reasonable and achievable, and whether or not these actions were verified in the field. Also, briefly describe what environmental conditions were considered in your evaluations.
2. In Attachment 8 to the June 16, 2003, letter, PSEG provided estimated times for specific operator actions. What does the “estimated time” column represent? That is, do the estimated time values provided in Attachment 8 indicate the time an operator takes to complete the action, or the time available to take the action? Also, how were these estimates determined?
3. For those actions taken outside the control room (e.g., in the auxiliary building, switchgear room, etc.), do the time estimates given in Attachment 8 include travel time for the operator to access the specific location (i.e., does the 0.5 hours estimated to trip the turbine-driven auxiliary feedwater pump (2MS52) include the time it takes to dispatch the operator to the specified location, plus the time to access the pump control and actually trip the pump)?
4. In Attachment 8, are the “available personnel” a part of the minimum shift staff, or are they in addition to the minimum shift staff? Will the required number of personnel identified in the specific scenarios always be available to perform the required manual actions?
5. What are the consequences, if any, of an operator failing to take the manual actions that are being credited in the analyses for the three fire protection areas (e.g., if the operator fails to de-energize 21 and 22 charging pumps for the reactor coolant system inventory control in the 2FA-AB-84B sequence)? Also, what defense-in-depth measures are available to prevent adverse consequences to the plant or the public?

Enclosure