



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
SAM NUNN ATLANTA FEDERAL CENTER  
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ATLANTA, GEORGIA 30303-8931

Carolina Power & Light Company  
ATTN: Mr. James Scarola  
Vice President - Harris Plant  
Shearon Harris Nuclear Power Plant  
P. O. Box 165, Mail Code: Zone 1  
New Hill, NC 27562-0165

SUBJECT: SHEARON HARRIS NUCLEAR POWER PLANT - NRC SUPPLEMENTAL  
INSPECTION REPORT 50-400/02-08

Dear Mr. Scarola:

On May 31, 2002, the Nuclear Regulatory Commission (NRC) completed a supplemental inspection at your Shearon Harris Nuclear Power Plant. The enclosed report documents the inspection findings which were discussed on May 31, 2002, with you and other members of your staff.

This supplemental inspection was an examination of your problem identification, root cause evaluation, extent of condition determination, and corrective actions associated with a White finding identified in the mitigating systems cornerstone. The White finding involved a violation of your fire protection program resulting from a Thermo-Lag fire barrier assembly (wall) which had an indeterminate fire resistance rating.

Based on the results of the inspection, the NRC determined that your corrective actions (both planned and already completed) are appropriate to resolve the deficiency in your fire protection program concerning the Thermo-Lag fire barriers which did not comply with 10 CFR 50, Appendix R requirements. However, the White finding will remain open pending further NRC review of your corrective actions associated with the use of local manual operator actions in the new ACP room fire area to achieve safe shutdown in the event of a fire.

For administrative purposes, a Severity Level III violation associated with the Thermo-Lag fire barrier assembly between the B train switchgear room/auxiliary control panel room and the A train cable spreading room which was previously dispositioned in NRC's letter dated April 16, 2002, is identified in the enclosed report.

No findings of significance were identified during the inspection.

*problem identification*  
*portions of your actions to resolve this issue were appropriate. However, the NRC noted that your planned corrective actions included the use of manual operator actions as opposed to one of the protection schemes identified in 10 CFR 50 Appendix R. This use of manual action in this situation is currently under review.*  
*not inspectors evidence to NRC inspection. Additionally, we understand Additional following*  
*Corrections MADE 9/5/02 H/24*

B

On May 31, 2002, the Nuclear Regulatory Commission (NRC) completed the onsite portion of a supplemental inspection at your Shearon Harris Nuclear Power Plant. A preliminary exit was held with you and other members of your staff on May 31, 2002, <sup>(to discuss the results of that effort)</sup>. Following completion of additional in-office review, a final exit on this inspection was held with Mr. Eric McCartney and other members of your staff on September 6, 2002. The enclosed report documents the <sup>our</sup> ~~inspection~~ findings from this inspection.

A.

Based on the results of this inspection, the NRC determined that portions of your action to resolve this issue were appropriate. However the NRC noted that your planned corrective actions included the use of <sup>local</sup> manual actions as opposed to one of the protection schemes identified in 10 CFR 50, Appendix R. <sup>Given this,</sup> ~~However~~ the white finding will remain open pending ~~further review of your~~ ~~your decision~~ further NRC review of your planned corrective actions and the development of internal NRC inspection guidance in this area.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

Charles R. Ogle, Chief  
Engineering Branch 1  
Division of Reactor Safety

Docket No.: 50-400  
License No.: NPF-63

Enclosure: NRC Inspection Report No. 50-400/02-08  
w/Attachment

cc w/encl:  
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Regulatory Affairs CPB 9  
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U. S. NUCLEAR REGULATORY COMMISSION

REGION II

Docket No:  
License No:

50-400  
NPF-63

Report No:

50-400/2002-08

Licensee:

Carolina Power & Light (CP&L)

Facility:

Shearon Harris Nuclear Power Plant, Unit 1

Location:

5413 Shearon Harris Road  
New Hill, NC 27562

Dates:

~~May 29 - 31, 2002~~ — September 6, 2002

Inspectors:

J. Lenahan, Senior Reactor Inspector

Approved By:

Charles R. Ogle, Chief  
Engineering Branch 1  
Division of Reactor Safety

ENCLOSURE

SUMMARY OF FINDINGS

IR 05000400-02-08; Carolina Power and Light Company; on 5/29-31/02; Shearon Harris Nuclear Plant, Unit 1; Supplemental inspection of Thermo-Lag fire barrier with indeterminate fire resistance rating.

This inspection was conducted by regional inspectors. A White finding, which was previously dispositioned in an NRC letter dated April 16, 2002, is identified in this report for administrative purposes. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using IMC 0609 "Significance Determination Process" (SDP). Findings for which the SDP does not apply are indicated by "Green" or be assigned a severity level after management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 3, dated July, 2000.

Cornerstone: Mitigating Systems

The U.S. Nuclear Regulatory Commission (NRC) performed this supplemental inspection to assess the licensee's evaluation associated with the Thermo-Lag fire barrier separating Train A and Train B safety related cables in the Auxiliary Control Panel (ACP) room. The Thermo-Lag barrier provided a fire resistance of one hour and 48 minutes rather than the minimum three-hour fire resistance rating required by 10 CFR 50, Appendix R regulations. This performance issue was previously documented in NRC Inspection Report Number 50-400/99-13 and later characterized as having low to moderate risk significance (White) in an NRC letter dated April 16, 2002.

During this supplemental inspection, performed in accordance with Inspection Procedure 95001, the inspectors determined that the licensee's problem identification, root cause and extent of condition evaluation were adequate. The licensee's corrective actions to resolve the issues were ongoing and appeared to be adequate. However, the inspectors identified that the licensee intended to use local manual operator actions to lieu of protecting Train A and Train B cables in the new ACP room fire area as required by 10 CFR 50, Appendix R. The licensee entered this issue into its corrective action program and resolution is pending.

As a result of this concern, the white performance issue associated with the Thermo-Lag fire barrier will not be closed at this time. The issue will remain open pending further NRC review of the licensee's corrective actions associated with the use of local manual operator actions in the new ACP room fire area to achieve safe shutdown in the event of a fire.

White. A violation of the fire protection program required by 10 CFR 50.48 and License Condition 2.F was identified in an NRC letter dated April 16, 2002. The violation was for failure to implement the NRC approved fire protection program safe shutdown system separation requirements by maintaining a three-hour fire rated area separation barrier between the B Train switchgear room/auxiliary control panel room and the A train cable spreading room. The existing Thermo-Lag barrier had an indeterminate fire resistance rating (Section 03.01).

and the development of NRC inspection guidance on the use of local manual actions.

currently this approach is not consistent with an approved  
The inspectors identified that this is not consistent with the protection schemes identified in 10 CFR 50 Appendix R.



## Report Details

### 01 INSPECTION SCOPE

The U.S. Nuclear Regulatory Commission (NRC) performed this supplemental inspection to assess the licensee's evaluation associated with the Thermo-Lag fire barrier separating Train A and Train B safety-related cables in the Auxiliary Control Panel (ACP) room. The Thermo-Lag barrier provided a fire resistance of one hour and 48 minutes rather than the minimum three-hour fire resistance rating required by 10 CFR 50, Appendix R regulations. This performance issue was previously documented in NRC Inspection Report number 50-400/99-13 and later characterized as having low to moderate risk significance (White) in an NRC letter dated April 16, 2002.

The inspectors assessed problem identification, the licensee's root cause and extent of condition evaluation, and the licensee's corrective actions to resolve the White finding in their fire protection program. The inspectors reviewed design change documents, including design inputs, assumptions, and design evaluations; licensee self assessments; and changes to operating procedures, fire protection procedures, and the safe shutdown analysis. The inspector also examined in process plant modifications being performed on fire barriers.

### 02 EVALUATION OF INSPECTION REQUIREMENTS

#### 02.01 Problem Identification

##### a. Determination of who identified the issue and under what conditions

During the initial pilot fire protection inspection, which was conducted from November 1 - 5, 1999, and documented in NRC Inspection Report No. 50-400/99-13, two unresolved items (URLs) were identified by NRC inspectors. One of the URLs concerned the adequacy of a Thermo-Lag wall which was designated as a three-hour fire barrier between the Train B switchgear room and Train A safety related cables routed in the Auxiliary Control Panel (ACP) room. The potential problem of using Thermo-Lag as a fire barrier material was identified to industry by the NRC in 1992 in Generic Letter (GL) 92-08 and NRC Bulletin 92-01. In response to NRC Bulletin 92-01 and GL 92-08, the licensee conducted testing on a mock-up of the Thermo-Lag wall. This testing showed the barrier failed fire endurance test acceptance criteria at 1 hour 48 minutes. The licensee then performed an engineering evaluation which concluded the as-constructed Thermo-Lag wall met 10 CFR 50, Appendix R requirements for a three-hour barrier. The licensee initiated Problem Evaluation Report (PER) 99-006863 to document and disposition the URI identified in the pilot fire protection inspection.

##### b. Determination of how long the issue existed, and prior opportunities for identification

The degraded fire barrier was installed during original plant construction. Prior opportunities for identification were in response to NRC Bulletin 92-01 and GL 92-08. However, licensee actions to address GL 92-08 resulted in the acceptance of an inadequate Thermo-Lag fire barrier in 1997 (ESR 95-00620, "Thermo-lag Fire Protection Issues Resolution," Revision 1). There were several opportunities to find this problem. The final

provided sufficient protection for the area. Hence this issue was not identified by the NRC. Hence this issue was identified by the licensee.

*Opportunities to identify this included a*

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response to the GL provided the Harris final plan and included the safety evaluation for the modification. The 1998 triennial fire protection Nuclear Assessment Section audit inspected a sample of Thermo-Lag barriers and included the required, independent evaluation performed by a contractor. ~~Self-assessments~~ of the fire protection program after 1997, also had the opportunity to find the problem.

Determination of the plant-specific risk consequences (as applicable) and compliance concerns associated with the issue

*A less than  
engine fire  
would not  
affect the  
barrier  
wall.  
See*

The degraded fire barrier was evaluated for a postulated fire in the B switchgear room that ~~would engulf the room and~~ cause the Thermo-Lag fire barrier wall in the ACP area of the B switchgear room fire area to fail. Failure of this wall exposed Train A cables, located within the enclosed barrier area, to the postulated fire. The Train A cables behind the Thermo-Lag wall affected the Train A Auxiliary feedwater function and the A steam generator pressure operated relief valve functions. The NRC assessment concluded that this event was in the increased regulatory response (White) band. The results of the NRC's final significance determination for the degraded fire barrier were documented in an NRC letter to CP&L dated April 16, 2002, Subject: Final Significance Determination For a White Finding and Notice of Violation (Shearon Harris Nuclear Power Plant - NRC Inspection Report 50-400/00-09).

## 02.02 Root Cause and Extent of Condition Evaluation

### a. Evaluation of methods used to identify root causes and contributing causes.

The licensee initiated a Significant Adverse Condition Report, Action Request number 53063, to evaluate and correct the problem associated with the Thermo-Lag wall. The evaluation was performed using the systematic methods specified in the licensee's corrective action program specified in CP&L procedure CAP-NGGC-0200. The licensee concluded that the root cause of the White finding was ~~the inappropriate use of an~~ *Problem* analysis to accept changes to the NRC approved fire protection. *①. What inadequate fire tests* causes were inadequate testing of installed fire barriers, evaluations, and failure to address design requirements.

### b. Level of detail of the root cause evaluation

The inspectors determined that the root cause evaluation support the identified root and contributing causes. To identify why the inadequate fire testing of the installed or why there was too much reliance on GL 86-10 engine (contributing cause), or why there was a failure to adequately address design (contributing cause). ~~However, licensee personnel pointed out that the most important factor was that fire ratings must be tied to acceptable fire qualification. Additional layer of "why" questions would lose the fundamental qualification must be tied to an acceptable test. The inspection~~ *② Root cause above not the same as the root cause below.* *What does this mean* assessment.

- c. Consideration of prior occurrences of the problem and knowledge of prior operating experience

The licensee's investigation identified 218 matches from the INPO database including GLs, NRC Bulletins and Information Notices, and LERs related to similar Thermo-Lag issues at various nuclear power plants. The root cause evaluation found that the method of resolution at HNP was unique from the others identified.

- d. Consideration of potential common causes and extent of condition of the problem

In late 2000, the NRC identified additional Thermo-Lag fire barriers in the cable spreading rooms which did not meet the requirements for three-hour fire barriers. Because fire suppression systems (sprinklers) existed in most of these areas, the licensee was able to downgrade the barriers to a one-hour fire rating and comply with 10 CFR 50, Appendix R requirements. However, some areas did not have sufficient spatial coverage by these sprinklers. The licensee also did not have sufficient test data/documentation to adequately demonstrate that some as-built Thermo-Lag barrier configurations in the cable spreading room provided a one-hour barrier. As a result, the licensee has initiated a program to evaluate existing sprinkler coverage, install new sprinklers as required, and test the existing barrier configurations to show they will meet the one-hour requirement. These corrective actions are scheduled to be completed by December 2002. The licensee performed a detailed self-assessment to examine the overall fire protection program and compare it to applicable regulatory requirements. (Appendix R, UFSAR, SERs, etc.) The evaluation portion of the assessment has been completed. Numerous findings were identified. Examples of findings include: illegible UFSAR drawings, typographical errors in the UFSAR, unclear statements in the UFSAR and SERs, factual errors in the UFSAR (e.g., location of hose stations), and lack of test documentation to demonstrate the mullions door frames and transoms over fire doors meet 10 CFR 50, Appendix R requirements. ~~These findings are being reviewed to determine how they will be resolved. The licensee plans to issue the self-assessment report in August, 2002.~~

## 02.03 Corrective Actions

- a. Appropriateness of corrective actions

*portion of the* *using a protection scheme specified in*

The inspector determined that the licensee's corrective actions were adequate to resolve the issue. Corrective action included modifying the ACP fire area to eliminate the need ~~for the Thermo-Lag barrier. This would be done by upgrading an existing concrete wall to a three-hour fire barrier.~~ Design procedures were revised to emphasize the requirement that adequate qualification testing be performed prior to changing the approved fire protection program. However, the inspectors identified that the licensee intended to use local manual operator actions ~~in lieu of~~ protecting Train A and Train B cables in the new ACP room fire area as required by 10 CFR 50, Appendix R. The licensee entered this issue into its corrective action program as NCR 20020823, "Fire Protection Manual Action Discrepancies."

*This reduced the potential for fires ~~starting~~ in the Train B switchgear room from spreading to the ACP room. In addition*

As a result of this concern, the White performance issue associated with the Thermo-Lag fire barrier will not be closed at this time. The issue will remain open pending further NRC review of the licensee's corrective actions associated with the use of local manual operator actions in the new ACP room fire area to achieve safe shutdown in the event of a fire. *and the development of internal NRC inspection guidance in this area.*

b. ~~Prioritization of corrective actions~~

The inspector concluded that the licensee's corrective actions were properly prioritized to address the risk. The changes to the ACP fire area were being worked concurrent with completion of the engineering change. The inspectors reviewed Engineering Change (EC) 48802 to upgrade an existing concrete wall between Switchgear Room B and the ACP room to a three-hour rated fire barrier. This work included sealing the existing penetrations in the wall. The above actions transformed the ACP room into a new fire area. The licensee has revised its fire plan (OP-036, Safe Shutdown Following a Fire) to address the newly created area. *This plan includes the use of local manual operator actions to achieve safe shutdown for some fire scenarios in the new ACP room fire area.* ~~as discussed above.~~ *However, as discussed above,*

c. Establishment of schedule for implementing and completing the corrective actions

The inspector verified that the licensee's corrective action program identified assigned individuals, completion dates, and reference numbers to the licensee's corrective action tracking program to ensure that corrective actions would be completed in accordance with their priority. Procedure changes to ensure that review criteria exist to ensure that fire barrier modifications do not invalidate test results in the future was complete. Although the EC for the ACP room had not been issued, the field work had been performed at risk and was essentially completed prior to completion of the EC. *Reviews of other fire barriers were in progress, and initial walk downs were complete. Resolution of other fire protection issues were in progress and were being prioritized based on risk.*

d. Establishment of quantitative or qualitative measures of success for determining the effectiveness of the corrective actions to prevent recurrence

The effectiveness of these measures will be determined through direct observation of the completed modifications and changes to the fire protection program. The licensee has scheduled an effectiveness review for July 2003.

The inspectors performed plant walk downs and conducted document reviews in determining that the extent of condition for the Thermo-Lag wall issue included all Thermo-Lag usage in the plant. Thermo-Lag usage in the plant included various transoms and mullions on the 216', 236', 261', and 305' elevations of the reactor auxiliary building and control building; and Thermo-Lag wall and tunnel assemblies in the A and B cable spreading rooms, the B switchgear room and the ACP area. The problem with the Thermo-Lag issue was related to the use of engineering evaluations for determining the applicability of fire test results to qualify plant features as rated fire barriers. In the case of the Thermo-Lag, a failed three-hour qualification fire test was used to accept Thermo-Lag features by engineering evaluation as adequate for the hazard. The use of failed tests for fire rating determination appears specific to the resolution of this Thermo-Lag

issue. In other cases, hardware modifications were made to reconcile the configurations to the test configurations that passed rating tests. Upgrading the concrete wall between the B switchgear room and the ACP room to a three-hour barrier decreases the risk of damaging the A train cables in the ACP room as a result of a fire in the B switchgear room.

### 03 OTHER ACTIVITIES

#### 03.01 Other

(Closed) AV 50-400/00-09-01: Failure to Maintain the Fire Area Separation Barrier Between The B Train Switchgear Room/Auxiliary Control Panel Room and the A Train Cable Spreading Room as a 3-hour Rated Barrier

This apparent violation was identified to the licensee in a letter dated December 18, 2001, Subject: NRC Inspection Report 50-400/00-09; Preliminary White Finding. The apparent violation concerned the Thermo-Lag fire barrier assembly which <sup>separated</sup> the B train switchgear room and the A train cable spreading room. The results of testing performed by the licensee in 1994 and 1995 showed that the fire barrier did not have the required three-hour fire resistance rating. After reviewing the test results, the licensee changed the fire protection program by revising the rating of the fire barrier from three hours to that suitable for the hazard. This issue had been initially identified by NRC as unresolved item 50-400/99-13-01. In a letter dated April 16, 2002, Subject: Final Significance Determination For a White Finding and Notice of Violation (Shearon Harris Nuclear Power Plant - NRC Inspection Report 50-400/00-09), the licensee was informed that NRC had determined that this issue was a violation of License Condition 2F of the Harris Operating License. A Notice of Violation, Severity Level III, was included as an Enclosure to the April 16, 2002 letter. For tracking purposes, this Severity Level III violation is identified as Violation (VIO) 50-400/02-08-01: Failure to Implement and Maintain NRC Approved Fire Protection Program Safe Shutdown System Separation Requirements. Apparent Violation 50-400/00-09-01 is closed.

(Closed) AV 50-400/00-09-02: Failure to Obtain NRC Approval Prior to Implementing a Change to the Approved Fire Protection Program

This apparent violation was also identified to the licensee in a letter dated December 18, 2001, Subject: NRC Inspection Report 50-400/00-09; Preliminary White Finding. This apparent violation concerned the fact that the licensee made changes to the approved fire protection program by accepting the degraded fire barrier, through an engineering evaluation, without prior Commission approval. This issue had been initially identified by NRC as unresolved item 50-400/99-13-02. In a letter dated April 16, 2002, Subject: Final Significance Determination For a White Finding and Notice of Violation (Shearon Harris Nuclear Power Plant - NRC Inspection Report 50-400/00-09), the licensee was informed that NRC had determined that it was appropriate to cite this issue as part of Violation (VIO) 50-400/02-08-01: Failure to Implement and Maintain NRC Approved Fire Protection Program Safe Shutdown System Separation Requirements. Apparent Violation 50-400/00-09-02 is closed.

## 04 MANAGEMENT MEETINGS

Exit Meeting Summary

The inspector presented interim results of the inspection to Mr. J. Scarola and other members of his staff at the conclusion of the inspection on May 31, 2002. ~~A second exit meeting was held via telephone conference with Mr. J. Caves on September 6, 2002, to~~ present the final results of the inspection. The licensee acknowledged the findings presented. Proprietary information is not included in the inspection report.

Another  
Final

E. McCartney and other  
members of the staff

**SUPPLEMENTAL INFORMATION****PARTIAL LIST OF PERSONS CONTACTED****Licensee**

D. Alexander, Manager, Nuclear Assessment Section  
 J. Caves, Supervisor, Licensing  
 M. Fletcher, Fire Protection System Engineer  
 C. Georgeson, Electrical Engineer  
 J. Holt, Site Support Services Manager  
 A. Khanpour, Superintendent of Design, Harris Engineering Support Services (HESS)  
 D. McAfee, Fire Protection Program Manager  
 E. McCartney, Superintendent Of Technical Services, HESS  
 J. Scarola, Harris Plant Vice President  
 M. Wallace, Senior Analyst, Licensing

**NRC**

J. Brady, Senior Resident Inspector

**ITEMS OPENED, CLOSED AND DISCUSSED****Opened**

50-400/02-08-01	VIO	Failure to Implement and Maintain NRC Approved Fire Protection Program Safe Shutdown System Separation Requirements (Section 03.01)
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**Closed**

50-400/00-09-01	AV	Failure to Maintain the Fire Area Separation Barrier Between The B Train Switchgear Room/Auxiliary Control Panel Room and the A Train Cable Spreading Room as a 3-hour Rated Barrier (Section 03.01)
50-400/00-09-02	AV	Failure to Obtain NRC Approval Prior to Implementing a Change to the Approved Fire Protection Program(Section 03.01)

## **LIST OF BASELINE INSPECTIONS PERFORMED**

The following procedure was used to perform the inspection during the report period.  
Documented findings are contained in the body of the report.

IP 95001      Supplemental Inspection For One Or Two White Inputs In A Strategic  
Performance Area.

**ATTACHMENT**



## **LIST OF DOCUMENTS REVIEWED**

CP&L Procedure CAP-NGGC-200, Corrective Action Program, Rev. 4

Engineering Service Request 99-0180, Evaluation of Low Density Silicone Elastomer (LDSE) Seals, Rev. 0, 6/16/99

Engineering Change 48802, Make ACP Room Separate Fire Area, Revision 0, 7/17/02

Design Basis Document

Calculation E-5525, Safe Shutdown Analysis in Case of Fire, Rev. 5, 5/7/02, and pending changes resulting from EC 48802

Calculation E-5524, Safe Shutdown Analysis Changes to the Cable Function Report and Essential Cable Analysis, Rev. 5, 6/2/02

Abnormal Operating Procedure AOP-036, Safe Shutdown Following A Fire, Rev. 19, and pending revisions resulting from EC 48802

Drawing Numbers SK-48802-C-1000, ACP Door Transom Detail, Sheets 1 and 2

Drawing Number SK-48802-C-1000, ACP Wall Penetrations, Sheet 1

Work Orders 233062 01 and 233062 02, Make ACP Room Separate Fire Area

**ATTACHMENT**

Carolina Power & Light Company  
ATTN: Mr. James Scarola  
Vice President - Harris Plant  
Shearon Harris Nuclear Power Plant  
P. O. Box 165, Mail Code: Zone 1  
New Hill, NC 27562-0165

**SUBJECT: SHEARON HARRIS NUCLEAR POWER PLANT - NRC SUPPLEMENTAL  
INSPECTION REPORT 50-400/02-08**

Dear Mr. Scarola:

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This supplemental inspection was an examination of your problem identification, root cause evaluation, extent of condition determination, and corrective actions associated with a White finding identified in the mitigating systems cornerstone. The White finding involved a violation of your fire protection program resulting from a Thermo-Lag fire barrier assembly (wall) which had an indeterminate fire resistance rating.

Based on the results of the inspection, the NRC determined that your corrective actions (both planned and already completed) are appropriate to resolve the deficiency in your fire protection program concerning the Thermo-Lag fire barriers which did not comply with 10 CFR 50, Appendix R requirements. However, the White finding will remain open pending further NRC review of your corrective actions associated with the use of local manual operator actions in the new ACP room fire area to achieve safe-shutdown in the event of a fire.

For administrative purposes, a Severity Level III violation associated with the Thermo-Lag fire barrier assembly between the B train switchgear room/auxiliary control panel room and the A train cable spreading room which was previously dispositioned in NRC's letter dated April 16, 2002, is identified in the enclosed report.

No findings of significance were identified during the inspection.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

Charles R. Ogle, Chief  
Engineering Branch 1  
Division of Reactor Safety

Docket No.: 50-400  
License No.: NPF-63

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w/Attachment

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Regulatory Affairs CPB 9  
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(cc w/encl cont'd - See page 3)

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