

October 28, 2003

Mr. Mano K. Nazar
American Electric Power
Senior Vice President and Chief Nuclear Officer
Indiana Michigan Power Company
Nuclear Generation Group
500 Circle Drive
Buchanan, MI 49107

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNIT 1 - CORRECTION TO STAFF
SAFETY EVALUATION FOR AMENDMENT NO. 278 (TAC NO. MB7162 AND
MB7163)

Dear Mr. Nazar:

By letter dated July 18, 2003, the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 278 to Facility Operating License No. DPR-58 for the Donald C. Cook Nuclear Plant, Unit 1. On Page 5 of the NRC staff's safety evaluation for this amendment (Enclosure 2 to the July 18th letter) the staff incorrectly made a reference to Westinghouse WCAP 15789. This reference was made in relation to the staff's review of pressurized thermal shock reference temperature RT_{PTS} . The correct reference is Westinghouse WCAP-15879. Enclosed is a corrected Page 5 to be used in place of the page in the safety evaluation issued with Amendment No. 278.

Sincerely,

/RA by DHood for/

Mohammed A. Shuaibi, Senior Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-315

Enclosure: Corrected Page 5 of Staff Safety Evaluation for Amendment No. 278

cc w/encls: See next page

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DATE	10/28/03	10/28/03	10/28/03

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Donald C. Cook Nuclear Plant, Units 1 and 2

cc:

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1.8×10^{19} n/cm² which is the peak value at the inside surface of the vessel, thus, is acceptable. The PT limit curves, the low-temperature overpressure protection setting, and the RT_{PTS} to the end of the current license are determined by the properties of the intermediate to lower shell circumferential weld.

RT_{PTS} to 32 EFPYs

10 CFR 50.61 requires recalculation of the RT_{PTS} value whenever there are significant changes in the conditions of the beltline region. The planned power uprate and the change in critical element warrant recalculation of the RT_{PTS} .

The related analysis was performed by Westinghouse and is documented in WCAP-15879 which was included in the licensee's December 10, 2002, application. The 32 EFPY fluence was derived from WCAP-12483 and is an acceptable value. The evaluation of RT_{PTS} followed the guidance in RG 1.99. The estimated value is 242 °F. The relevant screening criterion in 10 CFR 50.61 is lower than 300 °F, therefore, the RT_{PTS} is acceptable.

4.0 SUMMARY

The NRC staff has determined that the proposed P-T limit curves are consistent with the alternate assessment criteria and methods of ASME Code Case N-641, and satisfy the requirements of 10 CFR 50.60(a) and (b), "Acceptance Criteria for Fracture Prevention Measures for Light-water Nuclear Power Reactors for Normal Operation;" Appendix G to 10 CFR Part 50, "Fracture Toughness Requirements;" and Appendix G to Section XI (1995 Edition), as exempted by the methods of analyses in the code case. The proposed curves for D.C. Cook, Unit 1 are, therefore, approved for incorporation into the technical specifications.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Michigan State official was notified of the proposed issuance of the amendment. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendment changes the requirements with respect to installation or the use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change the surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (68 FR 15762). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.