



# ***Cladding Behavior during Dry Cask Handling and Storage***

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Office of Science Laboratory  
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# ***Cladding Behavior during Dry Cask Handling and Storage***

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- **Maintaining SNF cladding integrity is important for cask performance**
  - Fuel retrievability, cask surface dose rate, criticality.
- **Factors that may affect cladding behavior include**
  - Thermal creep
  - Hydrogen in cladding – hydride redistribution under the influence of temperature and stress.
- **Objective of our work is to provide data to support**
  - Extending storage time (>20 years), and
  - Extending burnup (>45 GWd/MTU).

# ***Cladding Behavior during Dry Cask Handling and Storage***

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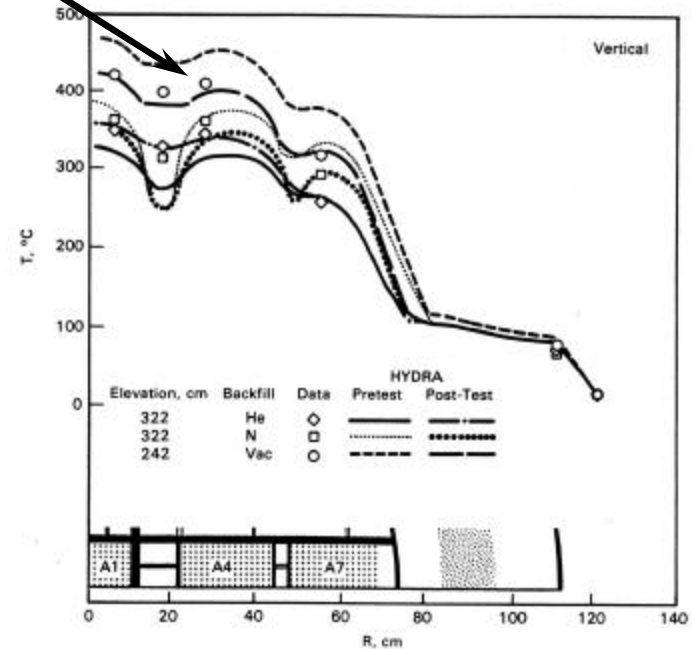
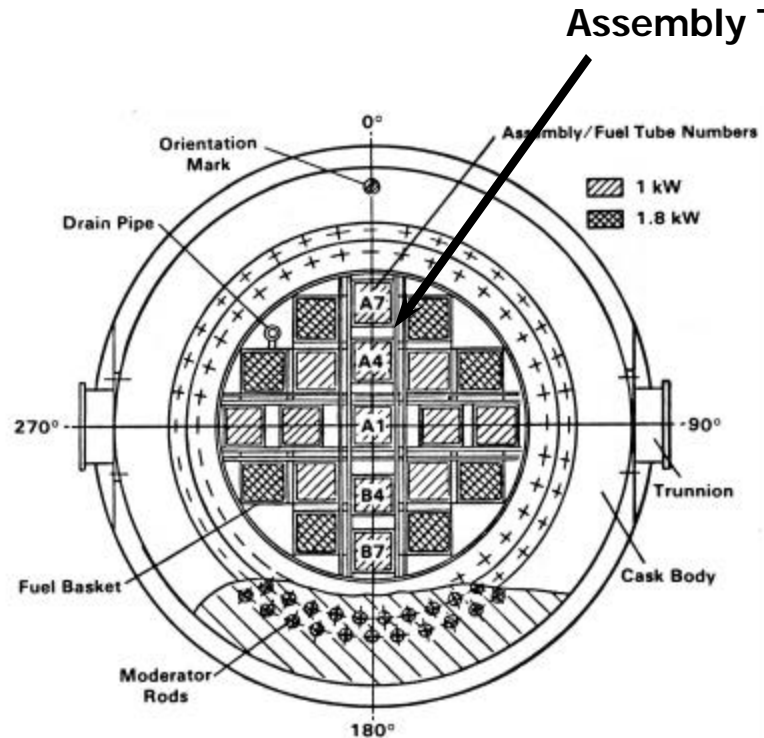
**We have performed the following:**

- **Characterization of medium-burnup (36 GWd/MTU) Surry PWR rods after 15 y-storage in a Castor-V/21 dry cask**
  - Extensive in-cask thermal benchmark tests, some emulated vacuum drying.
- **Isothermal annealing of cladding from high-burnup (67 GWd/MTU) H. B. Robinson rods**
  - Conditions relevant to vacuum drying (420-500°C, 2-72 h).
  - Post-annealing microhardness and hydride morphology determinations.
- **Thermal creep tests of both Surry and Robinson cladding**
  - Creep ductility and hydride reorientation.

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# Effects of 15-y Dry Cask Storage (and thermal benchmark tests) on Surry Rods

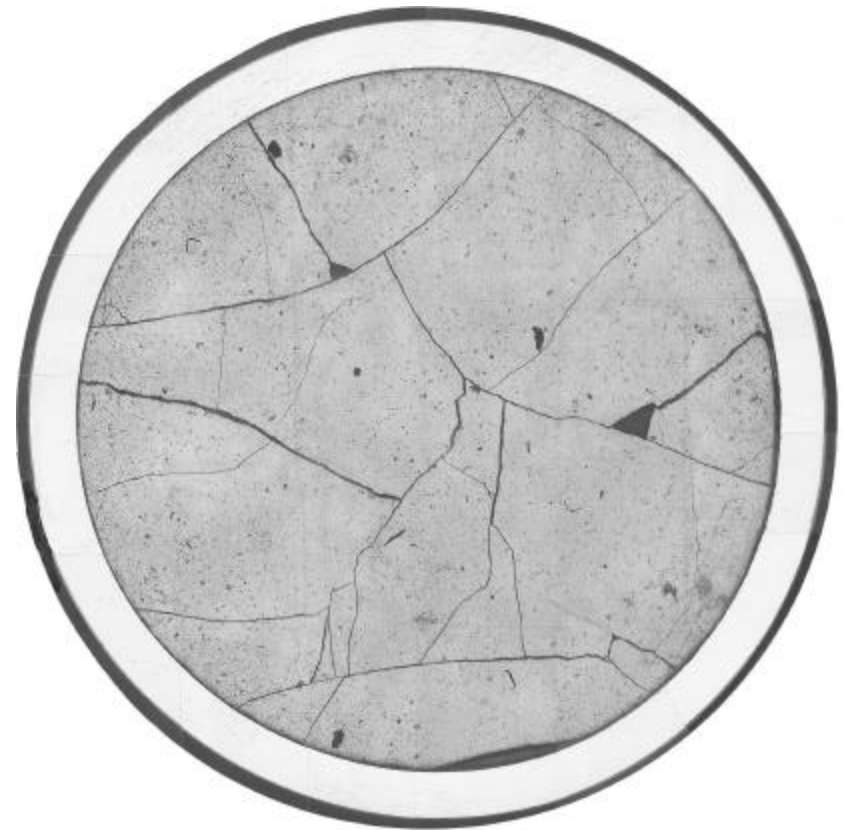
# Surry Rods (Assembly T11) in Castor-V/21



# ***Surry Post-Storage Characterization***

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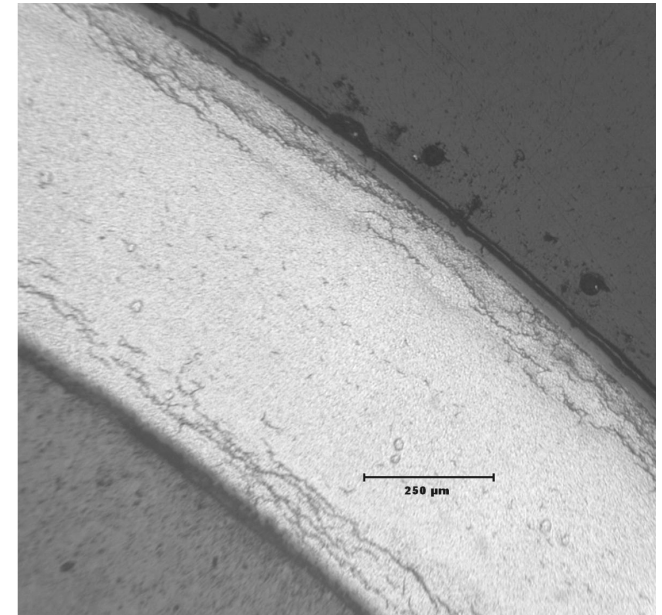
- **Effect of 15-y storage is benign**
  - **Gas release: ~ 0.5-1.0 %**
    - **No additional release**
  - **Fuel microstructure**
    - **No obvious changes**
  - **$DD/D_{as-built}$ : ~ -0.6%**
    - **Little or no in-storage creep**



# ***Surry Post-Storage Characterization*** (cont'd)

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- **Effect of 15-y storage is benign.**
  - **Cladding microhardness:**  
**235-240 DPH**
    - No apparent annealing in storage
  - **OD oxide thickness**
    - Normal ( $\sim 24\text{-}33\ \mu\text{m}$ )
  - **Cladding hydrogen content**
    - Normal ( $\sim 250\text{-}300\ \text{wppm}$ )
    - Axial migration - tbd
  - **Hydride reorientation**
    - None observed



# ***Surry Post-Storage Characterization*** (cont'd)

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- **Summary**
  - 15-y dry-cask storage (with extensive in-cask thermal benchmark tests) produced no apparent deleterious effects on the Surry rods.
  - Segments of Surry cladding were prepared for post-storage thermal creep and tensile tests.



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# Cladding Annealing Tests and Hydride Reorientation

# ***Robinson Cladding Annealing Tests***

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- **During vacuum drying, cladding temperature may be raised to  $> \sim 400^{\circ}\text{C}$  for hours to days. Will this alleviate radiation hardening in the cladding? What effect it has on hydrogen distribution?**
  - **Figure of merit: cladding microhardness**
  - **Test samples: short segments of defueled cladding from center of rod ( $11.3 \times 10^{21} \text{ n/cm}^2$ ,  $E > 1 \text{ MeV}$ ,  $\sim 600 \text{ wppm H}$ )**
  - **Corollary objective: study hydride redistribution under stress-free conditions**
  - **Test environment: high-purity argon**

# ***Robinson Cladding Annealing Tests***

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- Annealing Test Matrix**

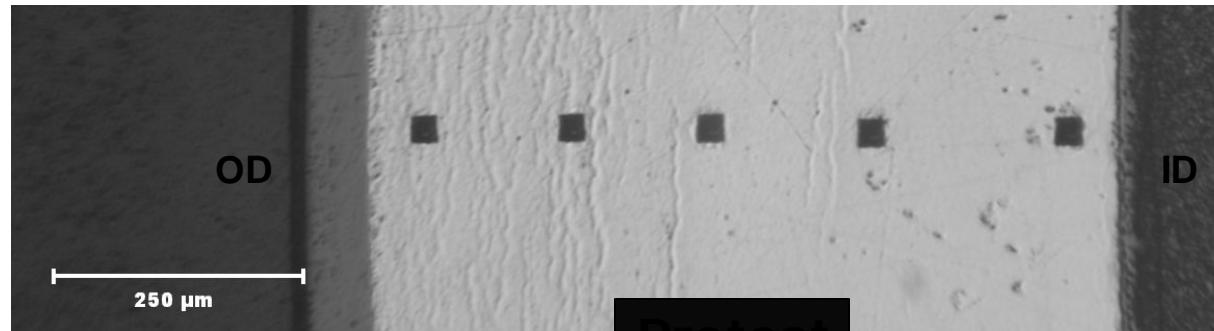
	2 h	10 h	20 h	48 h	72 h
420°C			C6		C7
450°C	C8	C9			
500°C	C10			C11	

# ***Robinson Cladding Annealing Tests***

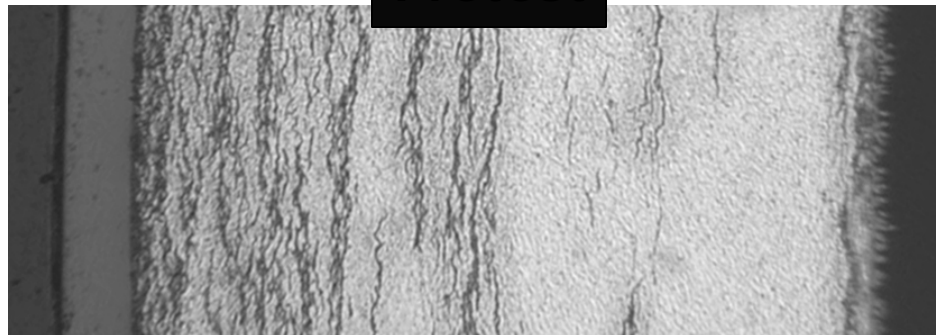
- **Microhardness Determination**

- Apply a known load with a diamond tip, measure the size of the indentation, and convert to microhardness (DPH)

DPH uniform  
across thickness,  
Avg = 252



Etched, showing hydrides  
~ 600 wppm H



# ***Robinson Cladding Annealing Tests***

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- **Microhardness Determination**

- For nonirradiated sibling:  $H_o = 203$
- For as-irradiated sibling:  $H_i = 252$

## **Microhardness after annealing tests**

	<b>2 h</b>	<b>10 h</b>	<b>20 h</b>	<b>48 h</b>	<b>72 h</b>
<b>420°C</b>			<b>226</b>		<b>215</b>
<b>450°C</b>	<b>224</b>	<b>217</b>			
<b>500°C</b>	<b>218</b>			<b>206</b>	

# ***Robinson Cladding Annealing Tests***

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$$\text{Recovery} = \left[ 1 - \frac{H - H_o}{H_i - H_o} \right]$$

## **% Radiation Hardening Recovery**

	<b>2 h</b>	<b>10 h</b>	<b>20 h</b>	<b>48 h</b>	<b>72 h</b>
<b>420°C</b>			<b>54</b>		<b>75</b>
<b>450°C</b>	<b>58</b>	<b>71</b>			
<b>500°C</b>	<b>69</b>			<b>94</b>	

**Conclusion:**    **Given time, significant recovery will occur at  $T > \sim 420^\circ\text{C}$ .**

# ***Robinson Cladding Annealing Tests***

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- **Hydride Morphology Evolution**
  - Strongly governed by hydrogen solubility in Zircaloy

<b>Temperature (°C)</b>	<b>Solubility (wppm)</b>
<b>25</b>	<b>0</b>
<b>200</b>	<b>13</b>
<b>400</b>	<b>200</b>
<b>420</b>	<b>240</b>
<b>450</b>	<b>310</b>
<b>500</b>	<b>460</b>

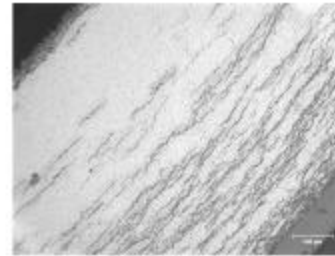
**Surry: 300 wppm**

**Robinson: 600 wppm**

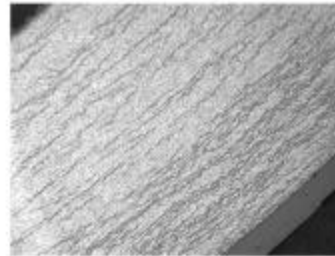
**J. J. Kearns**

# Robinson Cladding Annealing Tests

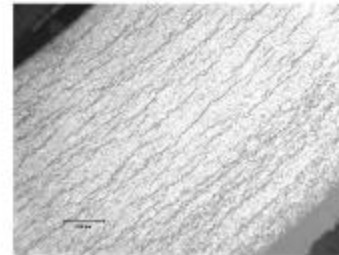
- Hydride Morphology Evolution
  - Precipitates became more uniformly distributed across the thickness
    - *Temperature.*  
*time*
  - No radial reorientation (being stress-free)



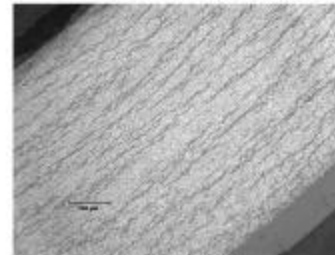
611C2 As-irradiated Control



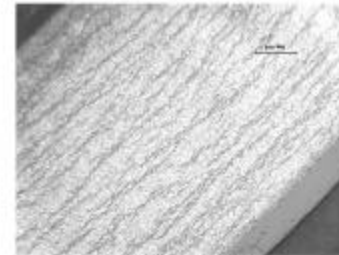
611C6 420°C, 20 h



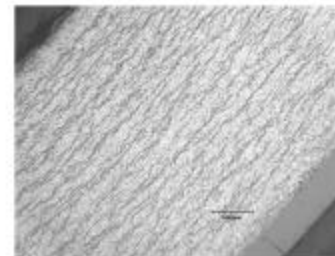
611C7 420°C, 72 h



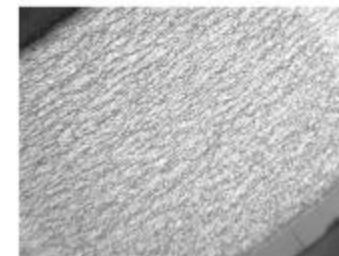
611C8 450°C, 2 h



611C9 450°C, 10 h



611C10 500°C, 2 h



611C11 500°C, 48 h

Hydride Morphology

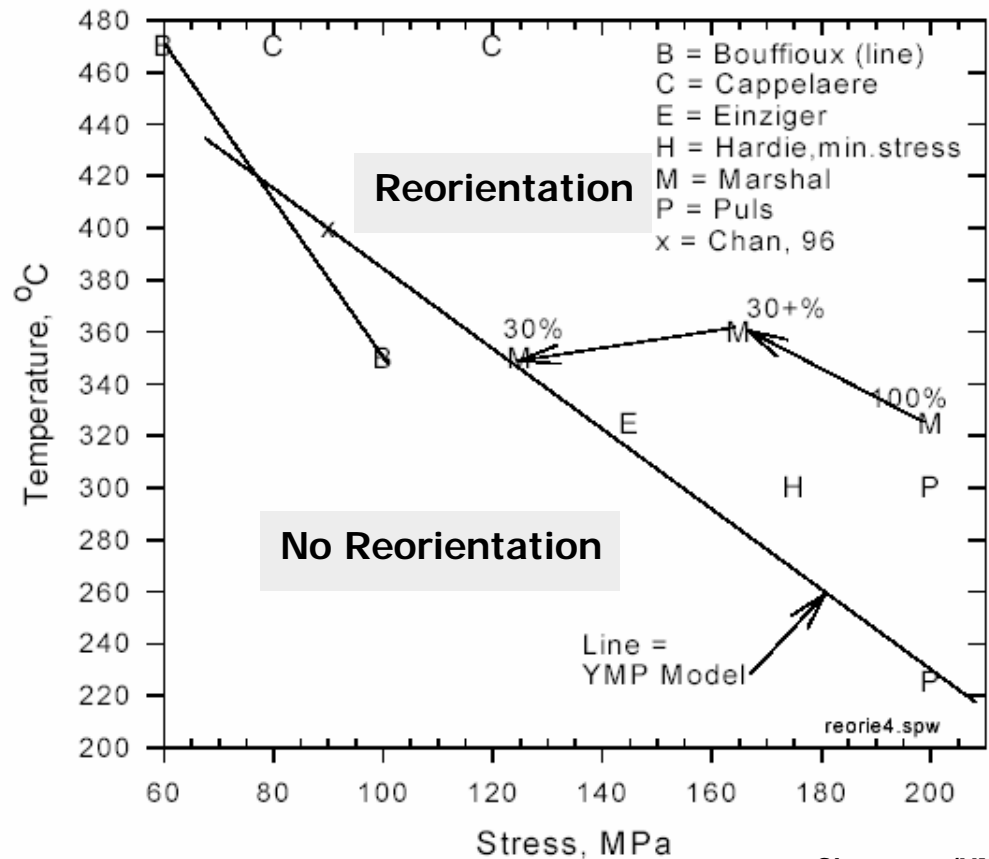
H. B. Robinson  
Cladding Annealing  
Test Samples

$t$   
 $T$



# Hydride Reorientation – Creep Tests

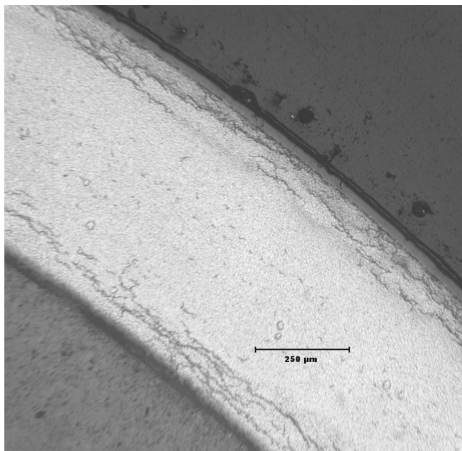
- Radial hydrides, as little as 40 wppm, can significantly degrade cladding's mechanical properties. (Marshall)
- Stress, temperature, cool-down rate, microstructure, H content, etc., all play important roles. (Einziger)
  - Threshold hoop stress for 400°C is ~ 100 MPa.



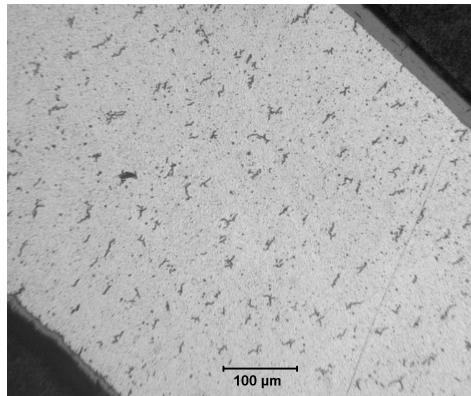
Siegmann (YMP)

# Hydride Reorientation – Creep Tests

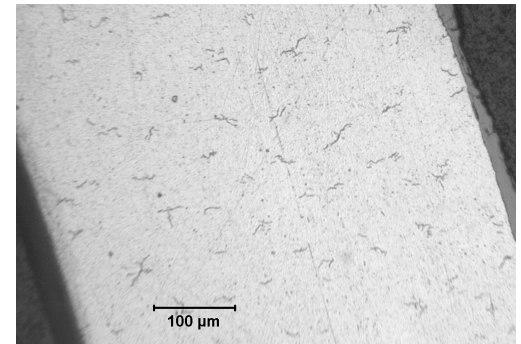
- Two Surry creep tests were intentionally shut down with samples under pressure: C3 (360°C, 220 MPa, 3760 h, 0.22% e) and C6 (380°C, 190 MPa, 2400 h, 0.35% e).
- Both samples survived the shutdown.
- Hydrides redistributed. Some now in radial direction, but no long-range linkage



Pretest



Posttest C3



Posttest C6

# Hydride Reorientation – Creep Tests

CEA (Cappelaere et al, ICEM 2001) – 470°C

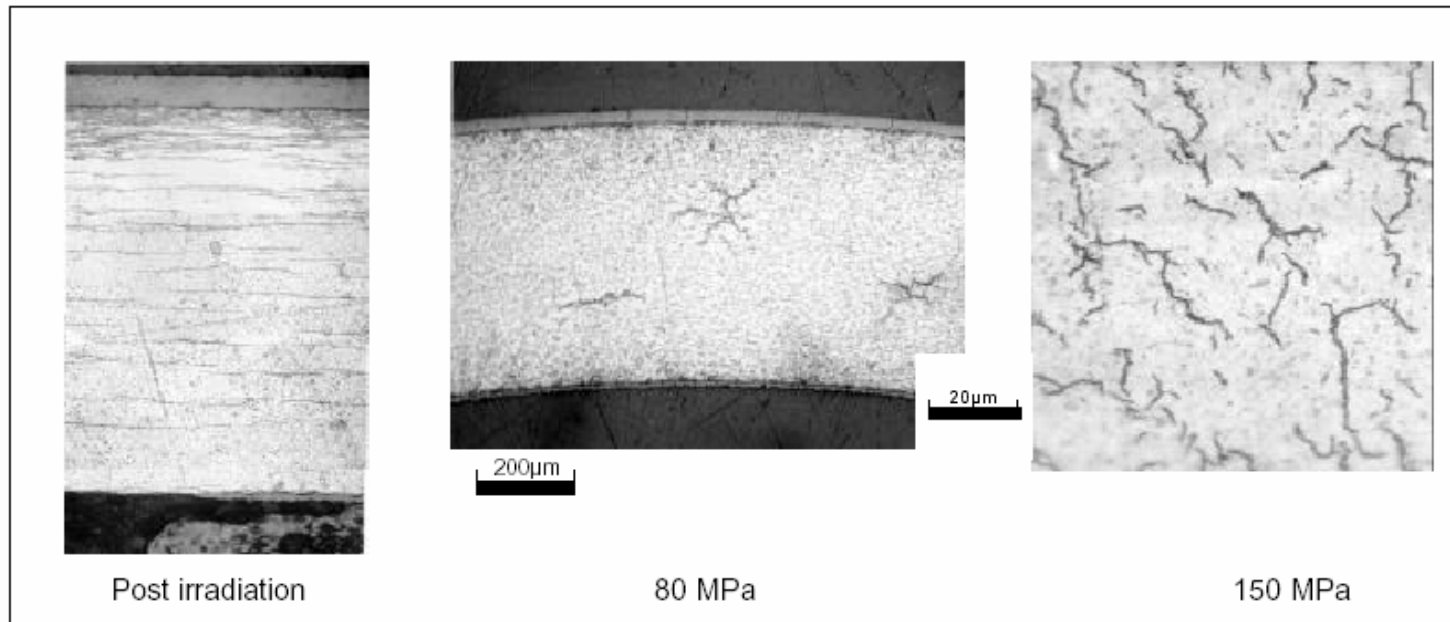
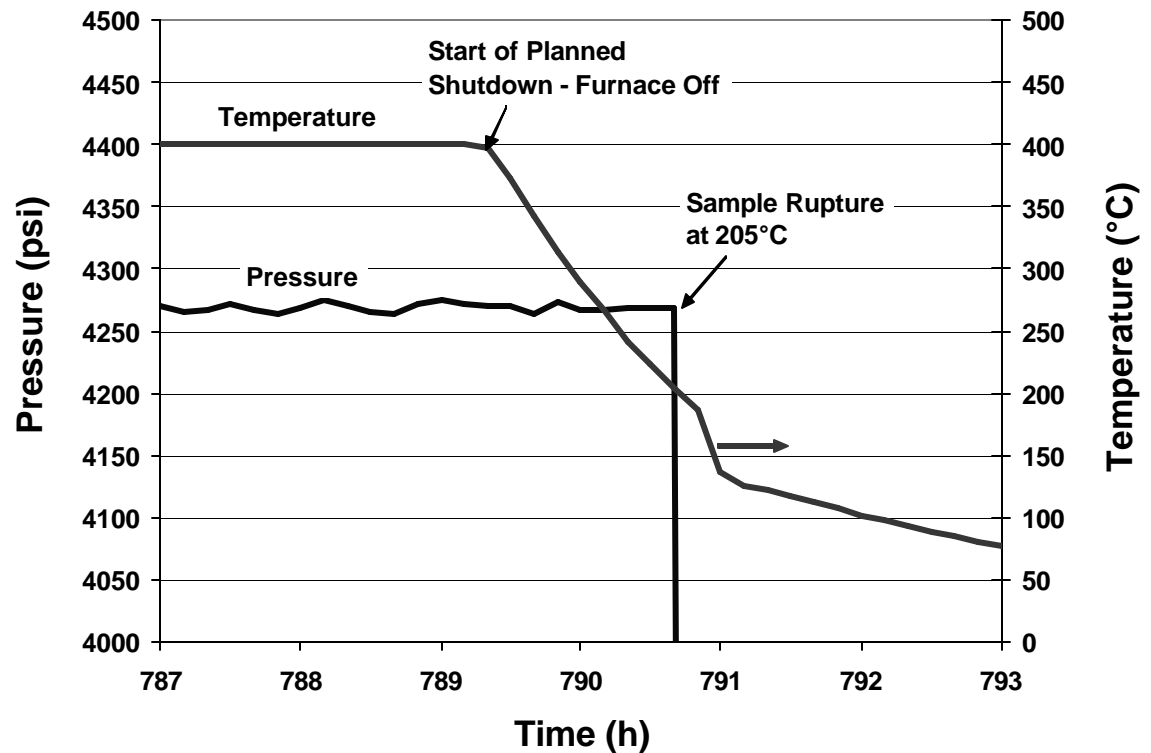


Figure 6 : Impact of creep tests on hydrides morphology, distribution and orientation

# Hydride Reorientation – Creep Tests

- Shutdown-under-pressure was repeated for one of the high-burnup H. B. Robinson creep sample: C15 (400°C, 190 MPa hoop, 2440 h, ~ 3.5% e).
- The sample ruptured during cool-down at 205°C.
- Cause being investigated
  - Hydride Reorientation?



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# Cladding Thermal Creep

# ***Thermal Creep Tests***

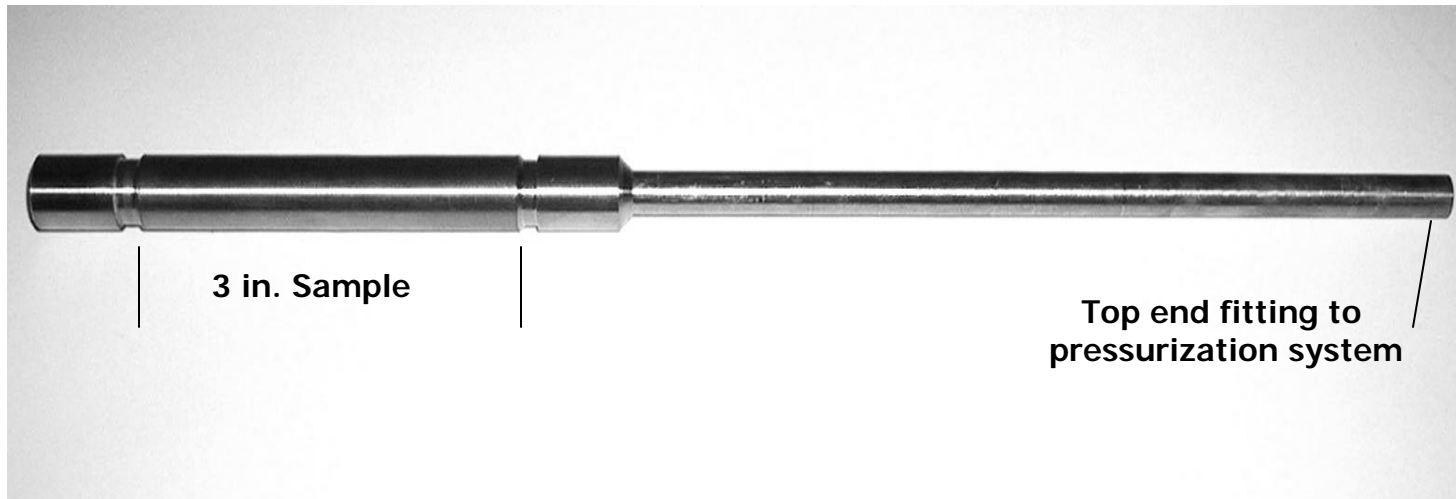
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- **Why studying creep?**
  - Creep is the dominant cladding deformation mechanism under normal conditions of dry storage. The core issue is, of course, cladding integrity.
- **Test objectives**
  - Determining steady-state creep rate and ductility limit.
  - Generating samples to study hydride reorientation and post-creep mechanical properties.

# ***Thermal Creep Tests***

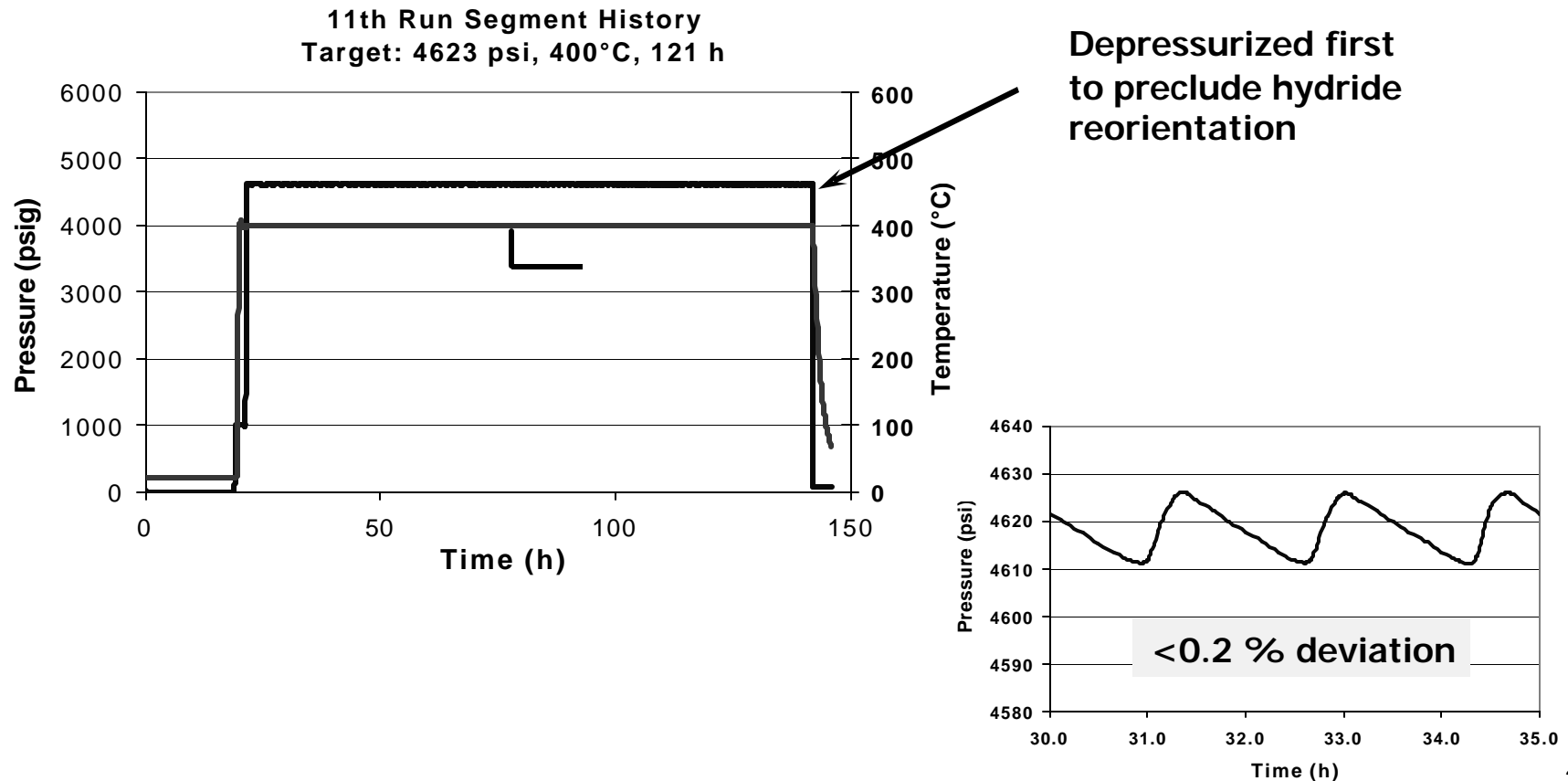
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- **Creep Test Specimen**
  - 76-mm-long segments of irradiated cladding.
  - Welded end fittings.
  - Pressure actively regulated.



# Thermal Creep Tests –Typical Performance

- Good pressure and temperature control
- Periodic shutdowns for laser profilometry

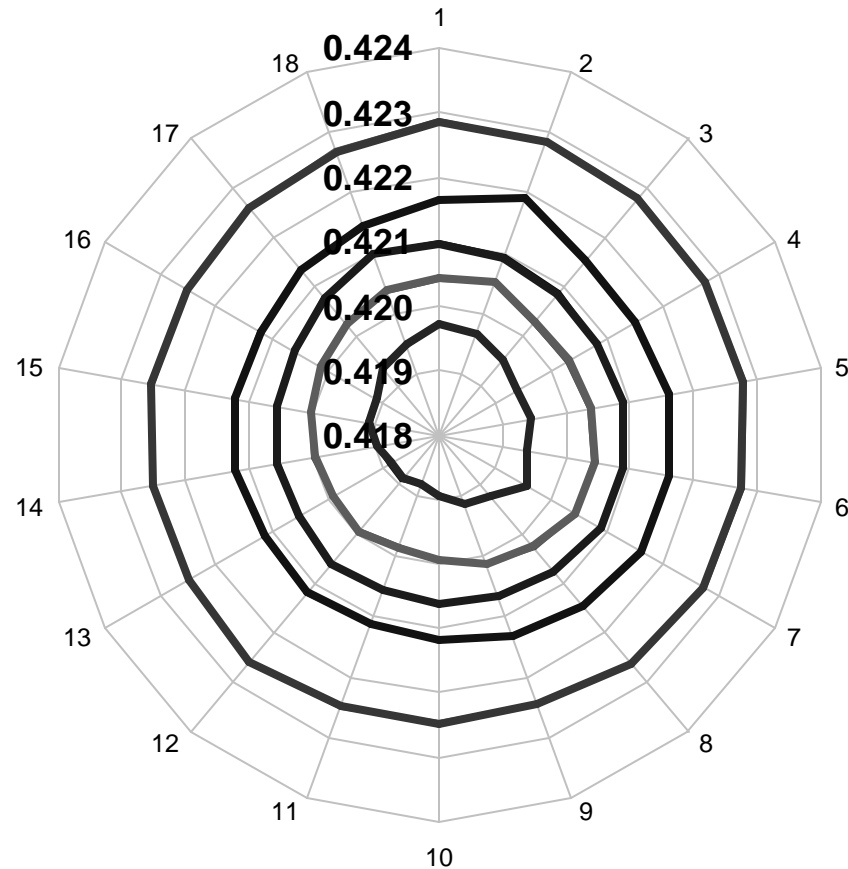




# ***Thermal Creep Tests***

## **Laser Profilometry – Typical Results**

- Midplane cross-sectional profiles of a sample at 0, 335, 671, 1028, and 1820 h. (Dimensions in inches.)



# Thermal Creep of Post-Storage Surry Cladding

## Surry Summary Results

Sample	Temp. (°C)	Stress (MPa)	At End of Test			Sample Disposition
			Hours	Avg. $\epsilon$	Intact?	
C3	360	220	3305	0.22	Yes	DE <sup>(1)</sup>
C6	380	190	2348	0.35	Yes	DE <sup>(1)</sup>
C8	380	220	2180	1.10	Yes	Bend Test
C9	400	190	1873	1.03	Yes	--
		250	693 <sup>(2)</sup>	5.83	Yes	Bend Test
2-C9	400	160	286 <sup>(3)</sup>	0.22	Yes	tbd

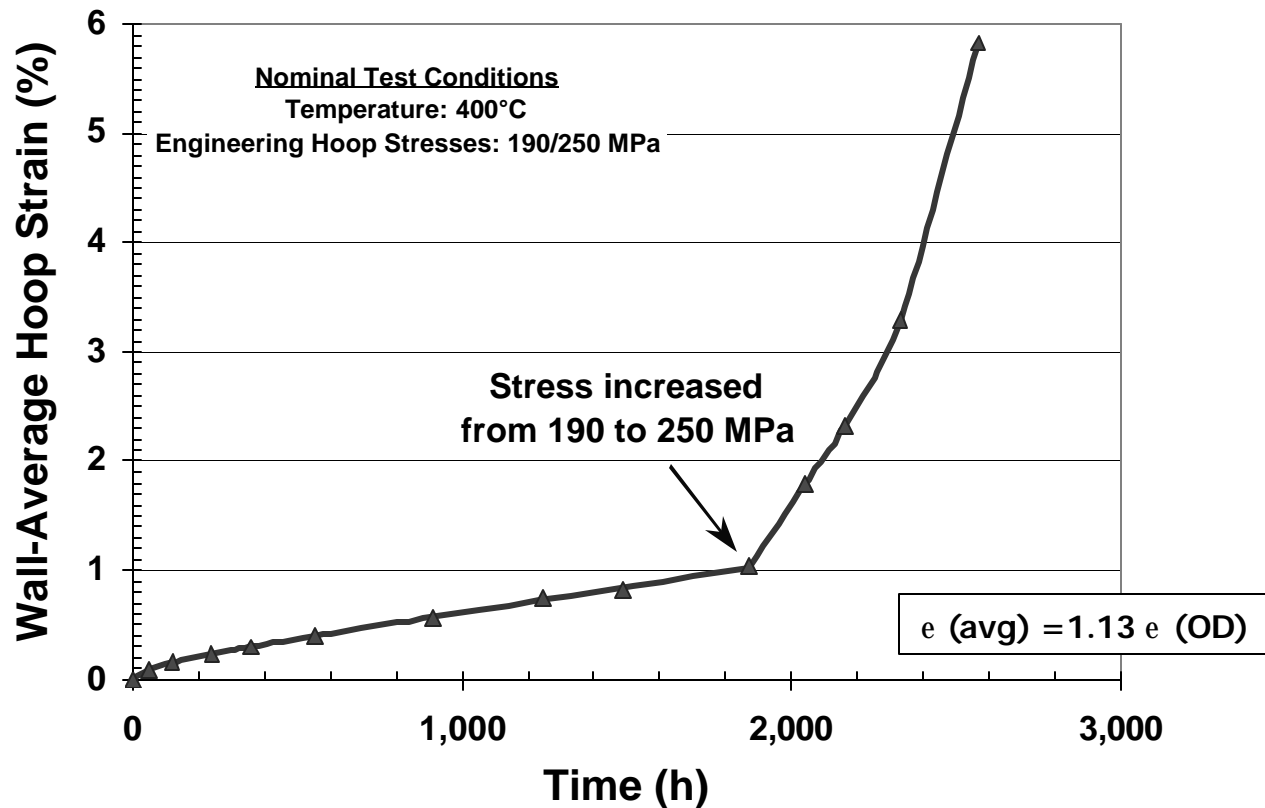
(1) DE: Destructive examination, for hydride orientation determination. For this, the final shutdown was done with sample pressurized.

(2) Incremental hours

(3) On-going

# Thermal Creep Tests – Surry C9

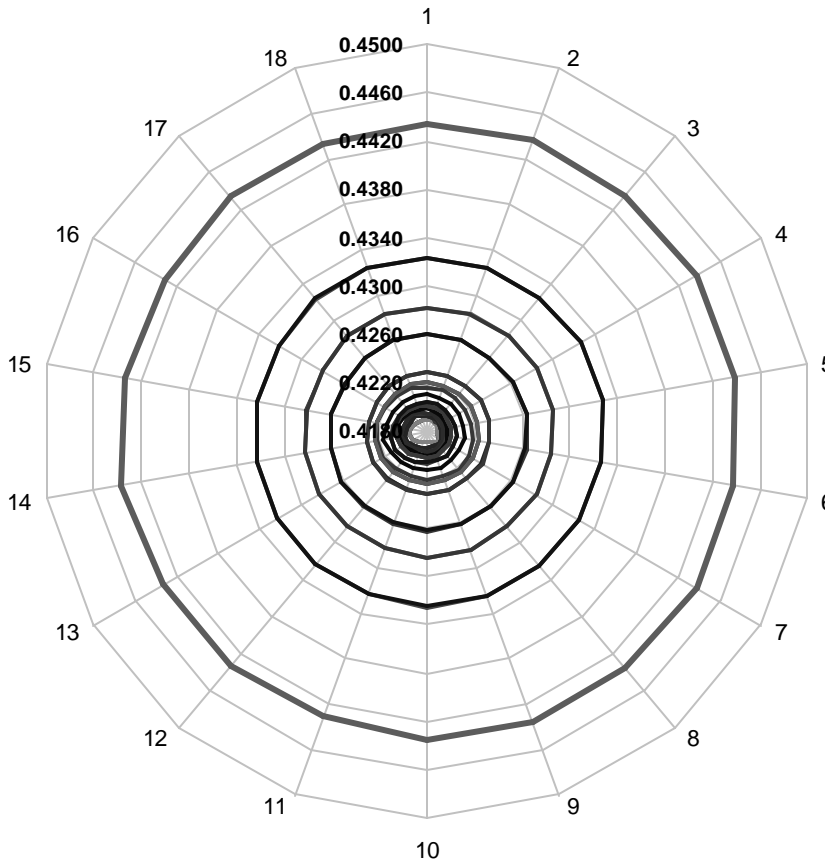
- 400°C, 190/250 MPa engineering hoop stress, 2566 h
- 5.8% average hoop strain, no rupture



# Thermal Creep Tests – Surry C9

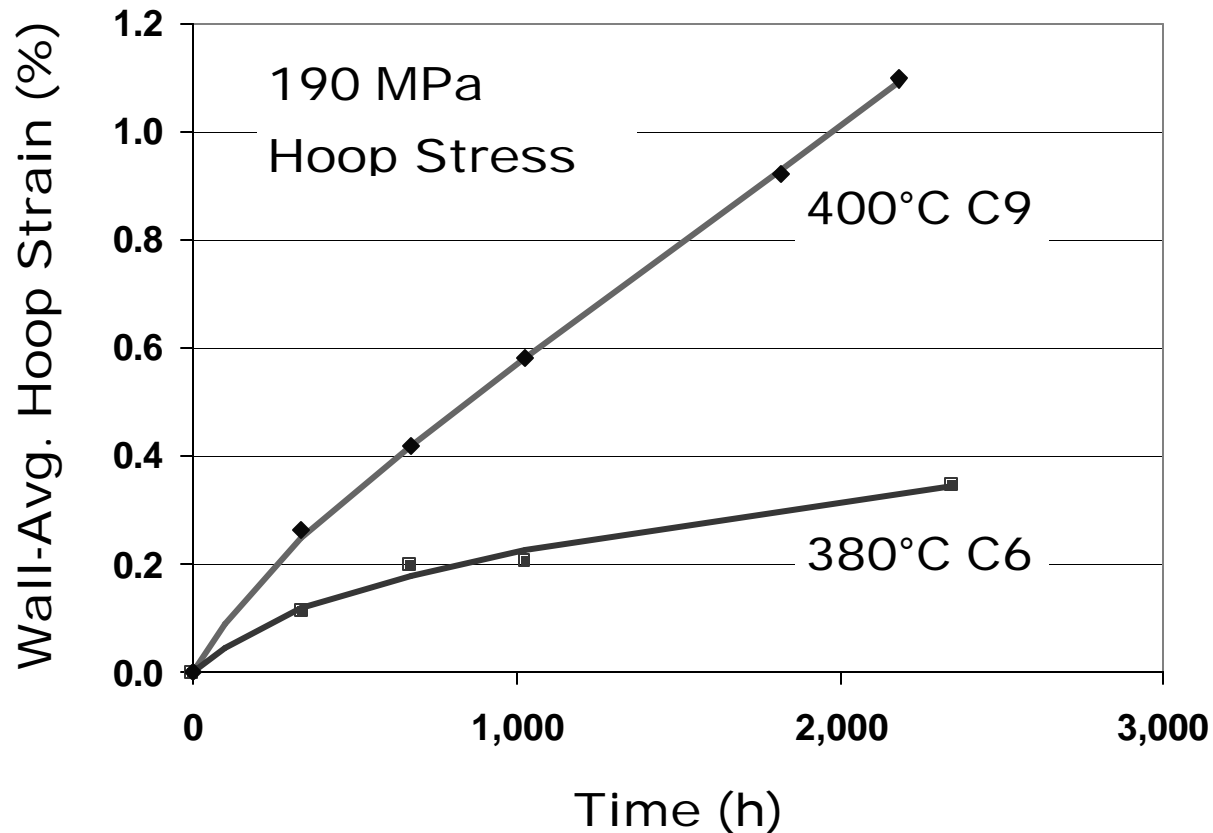
- Deformation uniform even at high strain (5.8%)
- No signs of imminent failure
- Additional creep ductility likely

Run-by-Run Cross Sectional Profiles of C9  
(Dimension in inches)



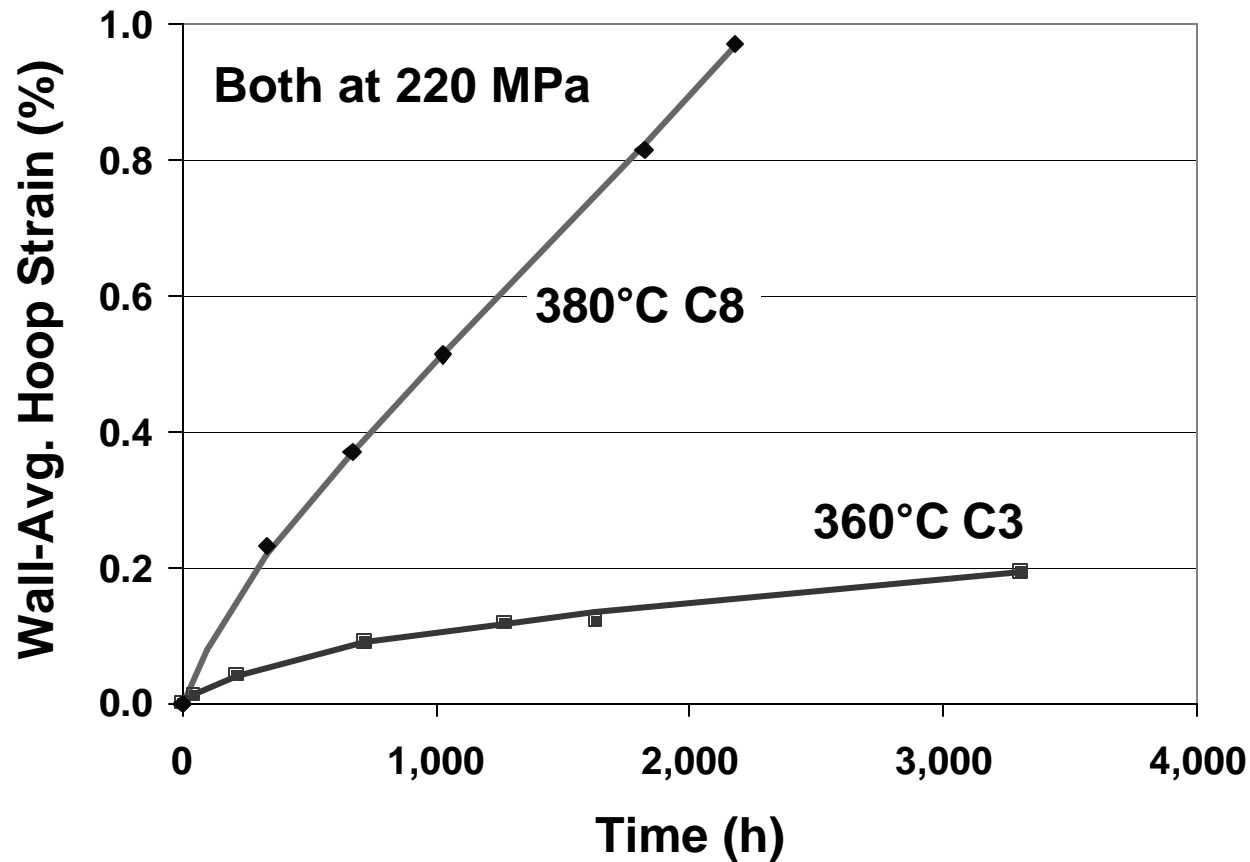
# Thermal Creep of Post-Storage Surry Cladding

## - Temperature Dependency



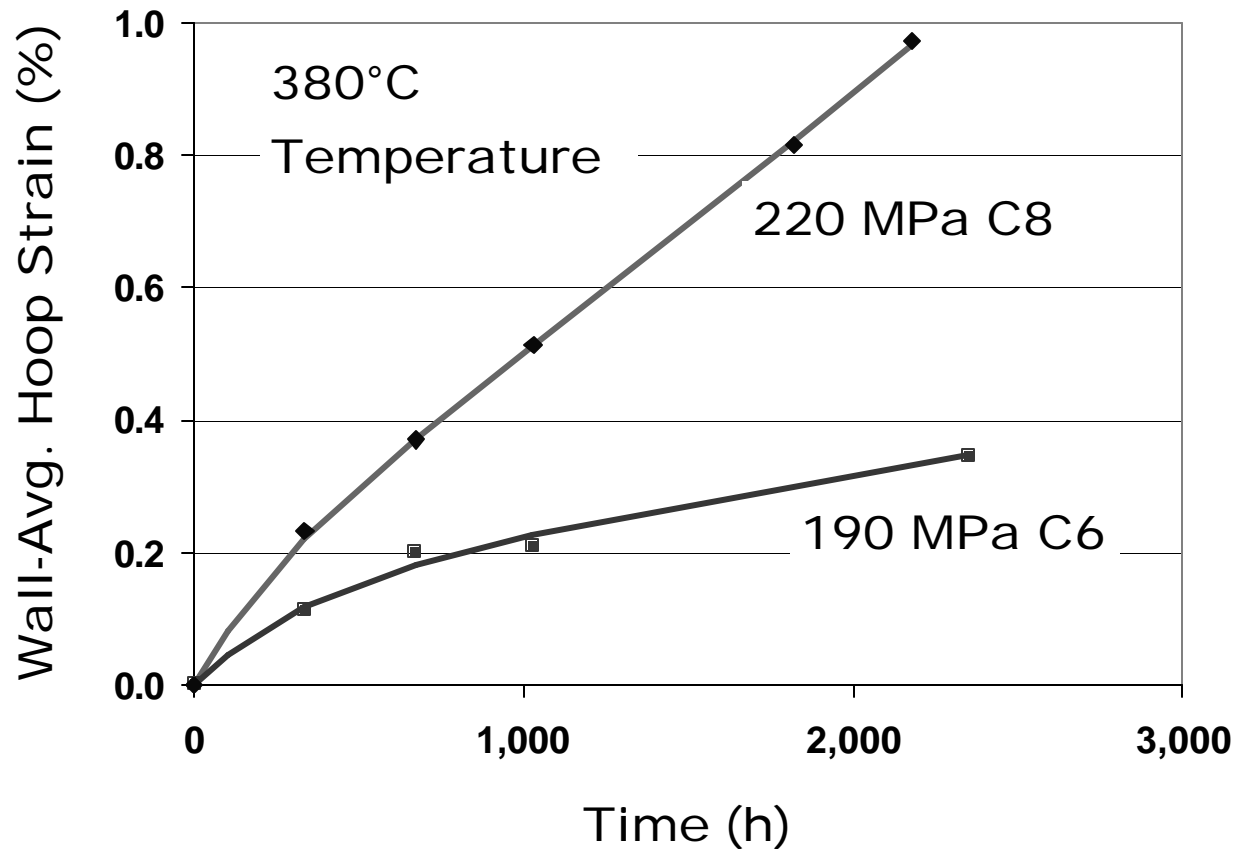
# *Thermal Creep of Post-Storage Surry Cladding*

- Temperature Dependency



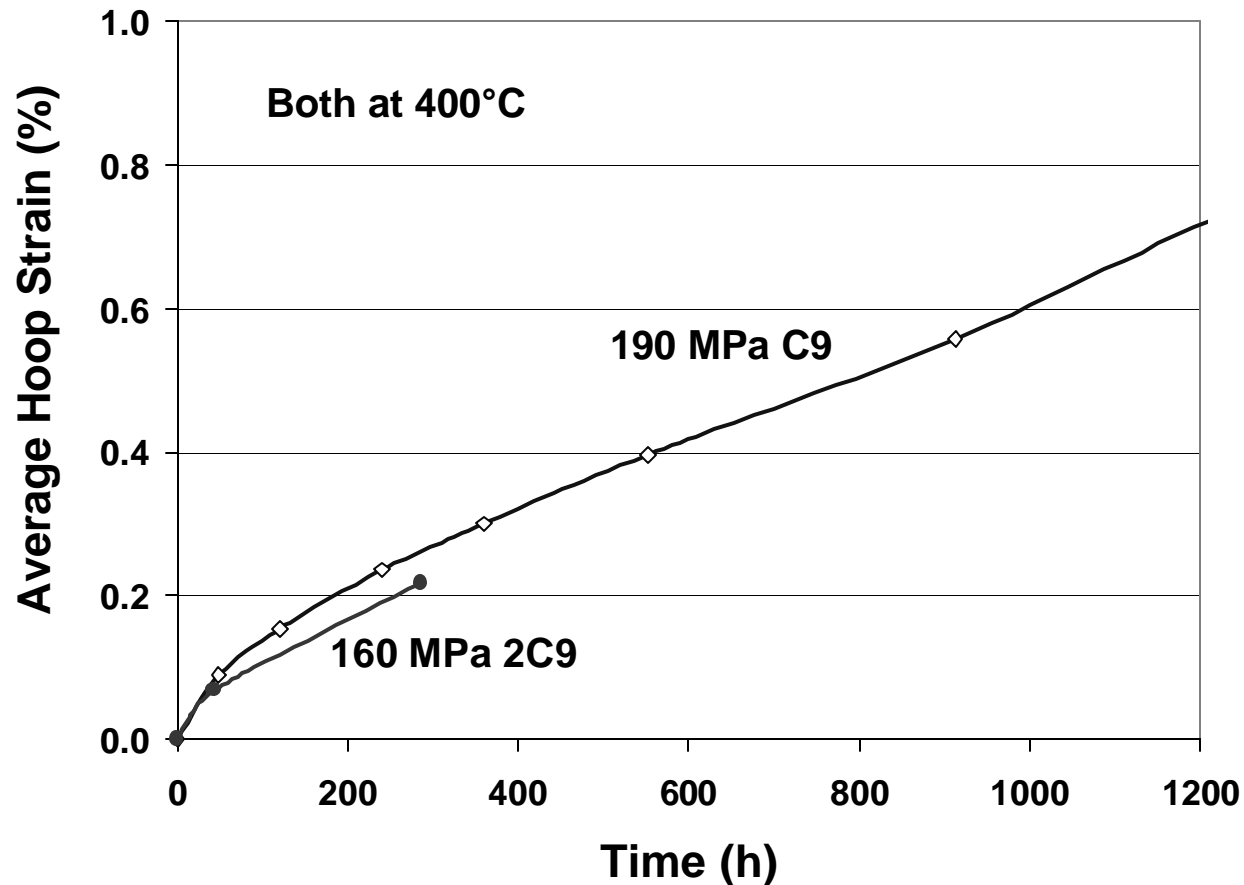
# Thermal Creep of Post-Storage Surry Cladding

## - Stress Dependency at 380°C



# *Thermal Creep of Post-Storage Surry Cladding*

- Stress Dependency at 400°C





# ***Thermal Creep of Post-Storage Surry Cladding***

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## **Secondary Creep Rates**

<b>Test Purpose</b>	<b>Sample</b>	<b>Temp. (°C)</b>	<b>Stress (MPa)</b>	<b>SS De/Dt<sup>(1)</sup> (%/h)</b>
<b>PSC</b>	<b>C3</b>	<b>360</b>	<b>220</b>	<b><math>\sim 1.6 \cdot 10^{-5}</math></b>
<b>PSC</b>	<b>C6</b>	<b>380</b>	<b>190</b>	<b><math>\sim 8.6 \cdot 10^{-5}</math></b>
<b>RCS</b>	<b>C8</b>	<b>380</b>	<b>220</b>	<b><math>\sim 4.6 \cdot 10^{-4}</math></b>
<b>RCS</b>	<b>C9</b>	<b>400</b>	<b>190</b>	<b><math>\sim 4.9 \cdot 10^{-4}</math></b>
			<b>250</b>	<b><math>\sim 4.9 \cdot 10^{-3}</math></b>

(1) e (avg). Values are approximates. Effects of wall thinning and diameter increase on hoop stress not included.

# ***Thermal Creep Tests – H. B. Robinson***

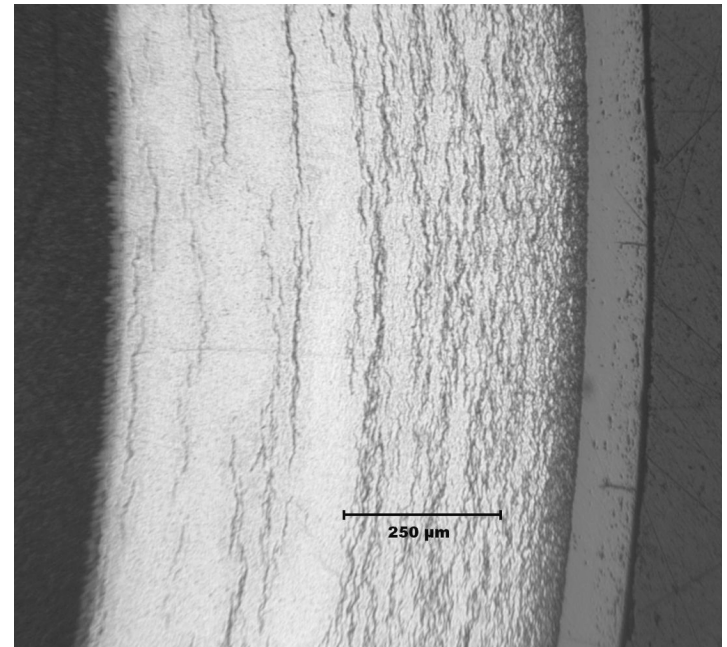
## **Robinson Test Matrix**

		Stress (MPa)				
		100	160	190	220	250
Temp. (°C)	420		1			
	400		1	C14 C15	1	
	380		1	C16	C17	
	360			1	1	
	320					

# *H. B. Robinson Cladding*

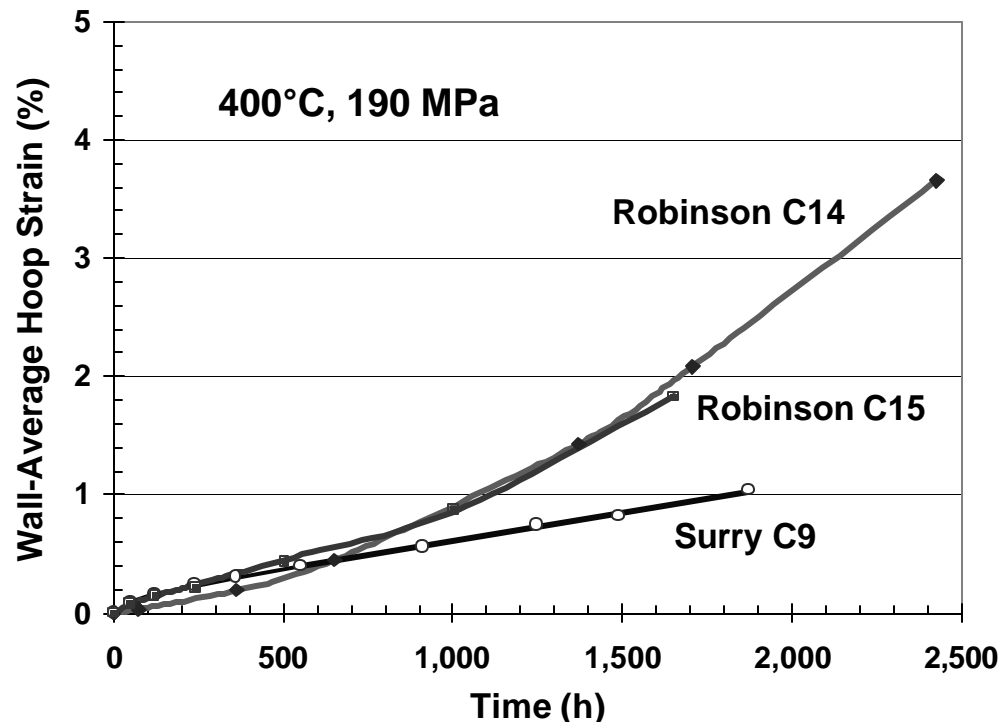
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- Significant corrosion and H uptake from extended operation to high burnup
  - *~ 100  $\mu\text{m}$  max. oxide*
  - *~ 800 wppm max. hydrogen*
  - *Circumferentially-oriented hydrides*
- What are the effects of increased hydrogen and radiation damage on creep?



# Thermal Creep Tests – H. B. Robinson

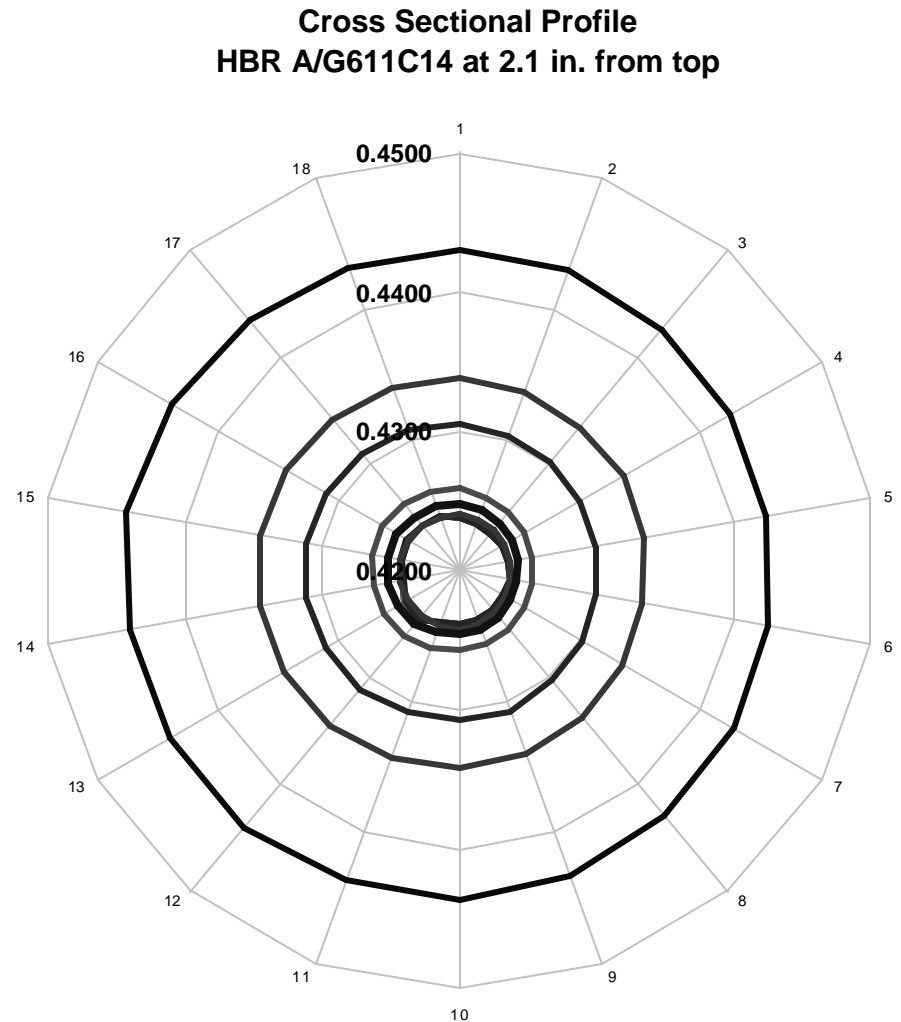
- At 400°C, creep rate of H. B. Robinson appears to be comparable to that of Surry at the onset of test. Rate becomes greater afterwards, possibly due to annealing.
- C14 was terminated at 2450 h at 3.6% e. Sample was intact.



# Thermal Creep Tests – H. B. Robinson

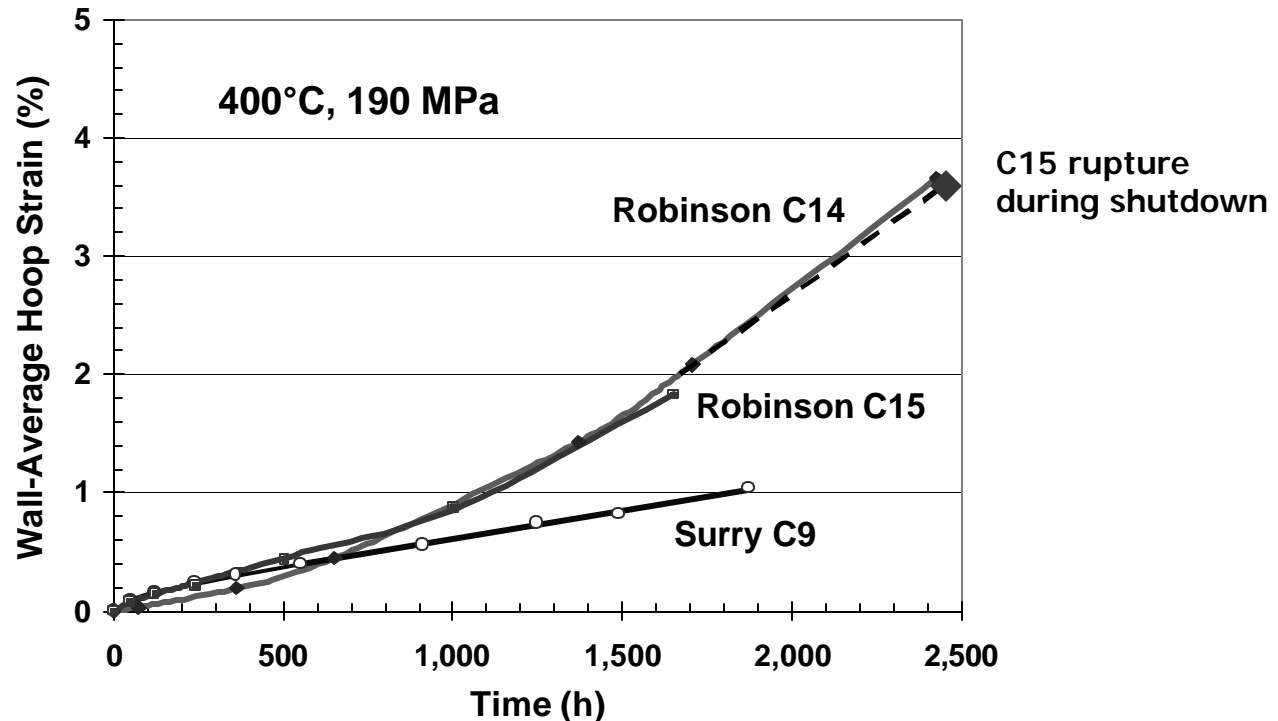
**Robinson C14 Sample  
shows good creep  
ductility: >3.6 % at  
400°C and 190 MPa.**

- Deformation still azimuthally uniform at end of test
- Additional creep life likely



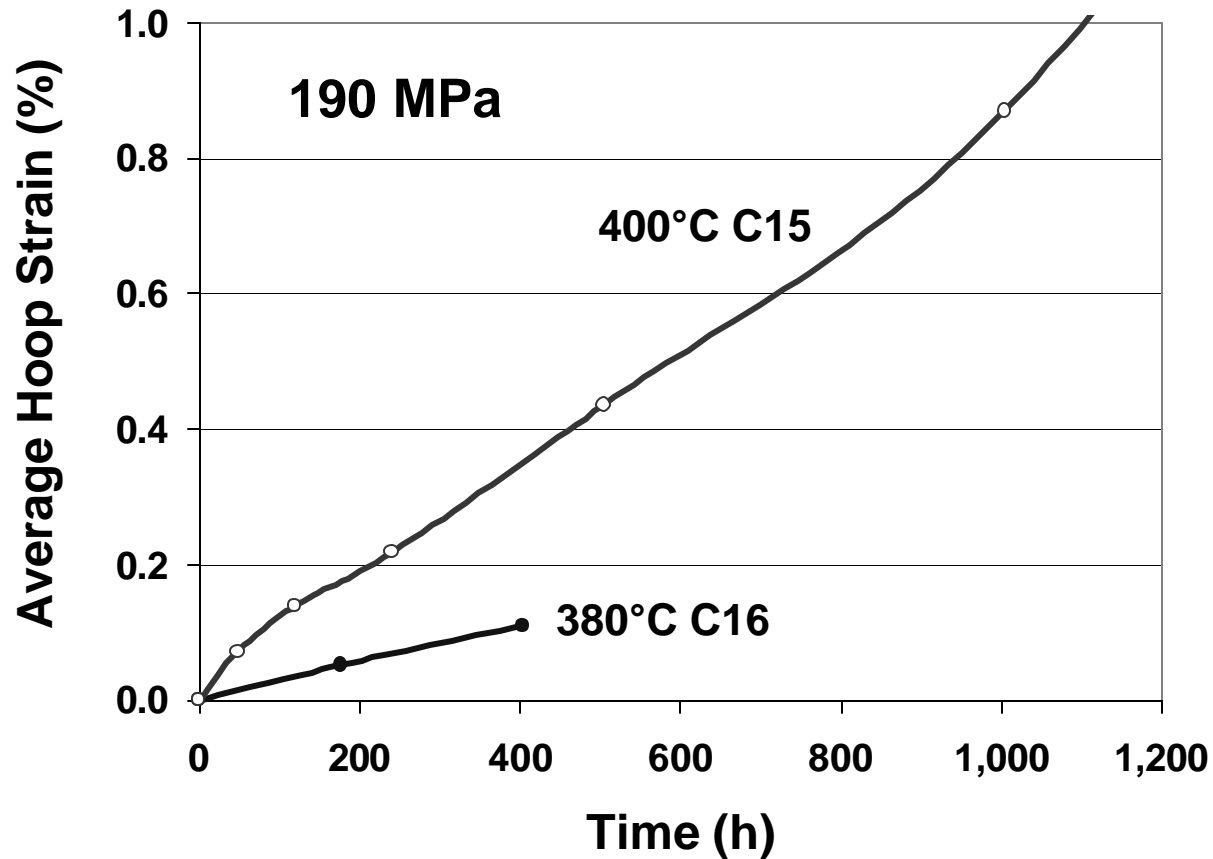
# Thermal Creep Tests – H. B. Robinson

- C15 developed a rupture during the final shutdown, which involved cooling from 400°C under full pressure to yield hydride reorientation data.
- Note, 190 MPa is a significant overtest for PWR rods.



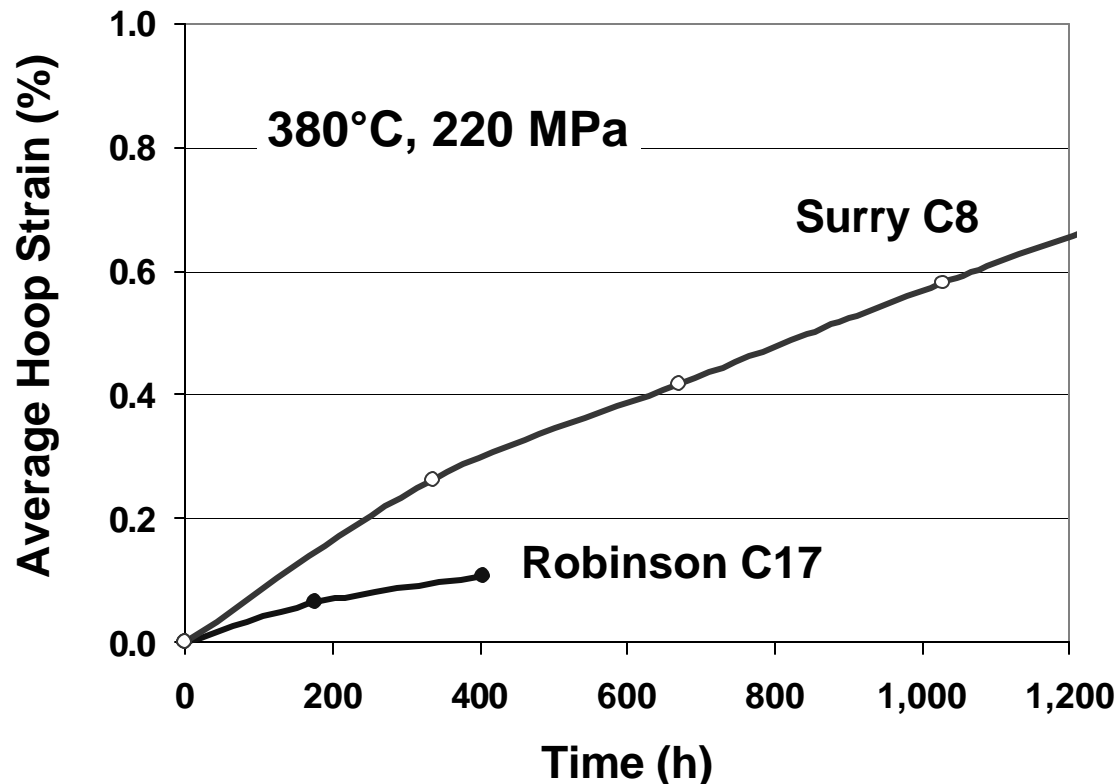
# Thermal Creep Tests – H. B. Robinson

- Temperature Dependency



# Thermal Creep Tests – H. B. Robinson

- Creep rate of H. B. Robinson appears to be smaller than that of Surry at the lower temperature of 380°C.
  - Less recovery at the lower temperature?





# ***Summary and Conclusions***

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- **15-y storage (with extensive thermal benchmark tests) caused no discernible degradation of the Surry rods.**
  - Data useful for dry-cask license extension.
- **Significant residual creep ductility has been demonstrated for the post-storage Surry cladding.**
  - Findings support NRC ISG-11 (Rev. 2).
- **Steady-state creep rates of Surry cladding show strong temperature and stress dependency in the regime tested.**
  - Useful for model development and code benchmarking.

# ***Summary and Conclusions*** (cont'd)

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- Robinson cladding annealing tests showed substantial fraction of radiation hardening can be annealed out at 420-500°C from hours to days.
- Early data show high-burnup Robinson cladding possesses good creep ductility and has a creep rate comparable to that of lower-burnup Surry at 400°C.
  - Because radiation damage has saturated? Annealing/recovering during tests? Insignificant H effect as long as there is no reorientation?
  - More tests are underway.

# ***Summary and Conclusions*** (cont'd)

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- **Unexpected rupture of the H. B. Robinson C15 sample during the final shutdown under pressure requires further investigation**
  - **Was hydride reorientation the cause? If yes, could it happen in real PWR fuel rods? (C15 with full pressure was a significant over-test.)**
- **Hydride reorientation may be a crucial issue for dry-cask storage and transportation, as it can affect cladding integrity. Efforts underway include**
  - **Annealing tests with sealed pressurized samples and with controlled cooling rates.**
  - **Post-creep characterization and mechanical tests.**