

October 14, 2003

Mr. Dale E. Young, Vice President
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing and Regulatory Programs
15760 W. Power Line Street
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER, UNIT 3 - UPCOMING STEAM GENERATOR TUBE
INSERVICE INSPECTION (TAC NO. MC0965)

Dear Mr. Young:

Inservice inspections of steam generator (SG) tubes play a vital role in assuring that adequate structural integrity of the tubes is maintained. As required by the plant's technical specifications, reporting requirements range from submitting a special report, within 15 days following completion of each inservice inspection of SG tubes, that identifies the number of tubes plugged and/or repaired; to submitting a special report, within 12 months following completion of the inspection, that provides complete results of the SG tube inservice inspection. The special report containing the complete results of the inspection shall include the following:

1. Number and extent of tubes inspected.
2. Location and percent of wall-thickness penetration for each indication of an imperfection.
3. Identification of tubes plugged and/or repaired.

A phone conference will be arranged with members of your staff to discuss the ongoing results of the SG tube inspections to be conducted during the upcoming Crystal River, Unit 3, refueling outage. We would like to have this phone call after the majority of the tubes have been inspected, but before the SG inspection activities have been completed. The preferable time would be when the SG inspection is approximately 75 percent complete. Enclosed is a list of discussion points to facilitate this phone conference.

The NRC staff plans to document a brief summary of the conference call as well as any material that you may have provided to the staff in support of the call.

This activity has been discussed with Mr. Sid Powell of your staff. The NRC staff appreciates your support in this matter.

Sincerely,

/RA/

Brenda L. Mozafari, Senior Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosure: List of Discussion Points

cc w/encl: See next page

This activity has been discussed with Mr. Sid Powell of your staff. The NRC staff appreciates your support in this matter.

Sincerely,

/RA/

Brenda L. Mozafari, Senior Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

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Enclosure: List of Discussion Points

cc w/encl: See next page

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STEAM GENERATOR TUBE INSPECTION DISCUSSION POINTS

PREPARED BY THE OFFICE OF NUCLEAR REACTOR REGULATION

PROGRESS ENERGY

CRYSTAL RIVER UNIT 3

DOCKET NO. 50-302

The following discussion points have been prepared to facilitate the phone conference arranged with the Crystal River 3 licensee to discuss the results of the steam generator (SG) tube inspections to be conducted during the upcoming refueling outage. This phone call is scheduled to occur towards the end of the planned SG tube inspection interval, but before the unit exits its refueling outage.

The NRC staff plans to document a brief summary of the conference call as well as any material that you may have provided to the staff in support of the call.

1. Discuss whether any primary-to-secondary leakage existed in this unit prior to shutdown.
2. Discuss the results of secondary side pressure tests.
3. For each steam generator, provide a description of areas examined, including the expansion criteria utilized and type of probe used in each area. Also, be prepared to discuss your inspection of the tube within the tubesheet, particularly the portion of the tube below the expansion/transition region.
4. Discuss any exceptions taken to the industry guidelines.
5. Provide a summary of the number of indications identified to-date of each degradation mode and SG tube location (e.g., tube support plate, top-of-tubesheet, etc.). Also provide information such as voltages and estimated depths and lengths of the most significant indications.
6. Describe repair/plugging plans for the SG tubes that meet the repair/plugging criteria.
7. Discuss the previous history of SG tube inspection results, including any "look backs" performed, specifically for significant indications or indications where look backs are used in support of dispositioning (e.g., manufacturing burnish marks).
8. Discuss, in general, new inspection findings (e.g., degradation mode or location of degradation new to this unit).
9. Discuss your use or reliance on inspection probes (eddy current or ultrasonic) other than bobbin and typical rotating probes, if applicable.

Enclosure

10. Describe in-situ pressure test plans and results, if applicable and available, including tube selection criteria.
11. Describe tube pull plans and preliminary results, if applicable and available; include tube selection criteria.
12. Discuss the assessment of tube integrity for the previous operating cycle (i.e., condition monitoring).
13. Provide the schedule for SG-related activities during the remainder of the current outage.
14. Discuss the following regarding loose parts:
 - what inspections are performed to detect loose parts
 - a description of any loose parts detected and their location within the SG
 - if the loose parts were removed from the SG
 - indications of tube damage associated with the loose parts
 - the source or nature of the loose parts if known
15. Once Through Steam Generators - if you have Babcock and Wilcox (B&W) welded plugs installed in the SGs, be prepared to discuss the actions taken in response to Framatome's notification of the effect of tubesheet hole dilation on the service life of B&W welded plugs.
16. Once Through Steam Generators - describe your inspection/plugging plans with respect to the industry-identified severed tube issue (NRC Information Notice (IN) 2002-02 and IN 2002-02, Supplement 1).

Mr. Dale E. Young
Florida Power Corporation

Crystal River Nuclear Plant, Unit 3

cc:

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