

October 14, 2003

Mr. J. T. Gasser
Vice President - Vogtle Project
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, Alabama 35201-1295

SUBJECT: VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2 RE: REQUEST
FOR ADDITIONAL INFORMATION - RESOLUTION OF GENERIC
LETTER 96-06 - WATER HAMMER ISSUES (TAC NOS. MA8627 AND MA8628)

Dear Mr. Gasser:

On August 19, 2003, during a conference call between Southern Nuclear Operating Company and the NRC staff, we discussed NRC staff's concerns related to our review of your response to Generic Letter 96-06, "Assurance of Equipment Operability and Containment Integrity During Design Basis Accident Conditions." The NRC staff requests the following information in order to continue with its review:

1. As stated in your submittal of March 23, 2003, in analyzing the nuclear service water system for the waterhammer event, you found that two mechanical snubbers exceeded the allowable loads. You then removed the snubber supports from the piping analysis model and a subsequent analysis determined that the remaining supports did not exceed the allowable stresses. The NRC staff has identified some issues regarding your approach in this analysis. The model for the piping system eliminates some supports that are part of the actual piping configuration. The NRC staff does not consider this approach to be conservative. If the overloaded supports are actually loaded to complete failure (i.e., breaking), there would be a dynamic load transfer on to the remaining supports and piping that is not adequately modeled. Particularly, where there are one-way supports, the sudden application of loads could result in gaps that are not predicted utilizing your proposed method. Provide a discussion regarding the configuration of the overloaded supports and the nature of the possible failure modes. Also, provide the margin present in the adjacent supports and in the piping. Provide sufficient details that demonstrate the fact that the structure will not be overloaded.
2. In your submittal of March 23, 2003, you provided the results of an initial engineering assessment and stated that final documentation of these evaluations has not been completed. Provide the schedule for the completion of these evaluations. Further, for the initial assessment, you referenced the Energy Power Research Institute (EPRI) document NP-6041, Revision 1, criteria for the acceptance criteria for stresses in certain piping and support components. However, it should be noted that NP-6041 was developed to assess seismic margins and is not a substitute for the licensing-basis acceptance criteria documented in your plant's Final Safety Analysis Report. Verify that the final evaluation will be based on the plant licensing-basis criteria, not the criteria provided in EPRI document NP-6041, Revision 1.

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This request was discussed with J. Stringfellow of your staff on October 6, 2003, and it was agreed that a response would be provided within 30 days of receipt of this letter.

If you have any questions, please contact me at 301-415-1447.

Sincerely,

/RA/

Frank Rinaldi, Project Manager, Section 1
Project Directorate II-1
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50- 424 and 50-425

Enclosure: As stated

cc w/encl: See next page

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Docket Nos. 50- 424 and 50-425

Enclosure: As stated

cc w/encl: See next page

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Vogtle Electric Generating Plant

cc:

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