

October 29, 2003

Mr. David A. Christian
Sr. Vice President and Chief Nuclear Officer
Dominion Nuclear Connecticut, Inc.
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: REVISION OF RELIEF REQUEST RR-89-34 FOR REPAIR OF REACTOR
PRESSURE VESSEL HEAD PENETRATION NOZZLES, MILLSTONE POWER
STATION, UNIT NO. 2 (TAC NO. MC0742)

Dear Mr. Christian:

By letter dated February 25, 2002, as supplemented March 13, March 19, and March 21, 2002, Dominion Nuclear Connecticut, Inc. (DNC) submitted Relief Request RR-89-34, Revision 1, for Millstone Power Station, Unit No. 2 (MP2). Your submittal proposed an alternative to certain requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), to support weld repairs of reactor pressure vessel (RPV) head control element drive mechanism (CEDM) penetration nozzles during the spring 2002 refueling outage (i.e., RFO 14). Specifically, you proposed to perform the weld repairs using an ambient temperature temper bead welding technique. The U.S. Nuclear Regulatory Commission (NRC) provided verbal authorization for the proposed alternative on March 22, 2002, and provided written authorization by a Safety Evaluation (SE) that was issued on October 2, 2002. The proposed alternative was authorized pursuant to Section 50.55a(a)(3)(ii) of Title 10 of the *Code of Federal Regulations* (10 CFR) for the third 10-year inservice inspection interval for MP2, which began on April 1, 1999, and ends on March 31, 2009.

By letter dated September 15, 2003, you submitted Revision 2 to Relief Request RR-89-34, to support potential RPV head CEDM penetration nozzle weld repairs during the fall 2003 refueling outage (i.e., RFO 15). The only change proposed by Revision 2 would be to change the repair procedure to remove a requirement for abrasive water jet conditioning of the final surface of the weld. The NRC staff has completed its review of MP2 Relief Request RR-89-34, Revision 2, and has determined that: (1) the proposed change in the repair procedure does not require relief from any ASME Code requirement; (2) the authorization provided by the SE for RR-89-34, Revision 1, is not affected by the proposed repair procedure change since the authorization was not based on water jet conditioning being performed; and (3) the proposed change in the repair procedure can be evaluated by DNC under the provisions of 10 CFR 50.59.

D. Christian

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Based on the preceding considerations, the NRC has terminated its review of RR-89-34, Revision 2, and we are closing TAC No. MC0742. These issues were discussed with Mr. David Dodson of your staff on October 14, 2003.

Sincerely,

/RA/

James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-336

cc w/encl: See next page

D. Christian

- 2 -

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/RA/

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* See previous concurrence

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Millstone Power Station, Unit No. 2

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