

October 1, 2003

Mr. David A. Christian
Sr. Vice President and Chief Nuclear Officer
Virginia Electric and Power Company
Innsbrook Technical Center
5000 Dominion Blvd.
Glen Allen, Virginia 23060-6711

SUBJECT: SURRY UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: DELETION OF
THE MONTHLY ANALOG ROD POSITION TEST (TAC NOS. MB6752 AND
MB6753)

Dear Mr. Christian:

The Commission has issued the enclosed Amendment No. 237 to Renewed Facility Operating License No. DPR-32 and Amendment No. 236 to Renewed Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments change the Technical Specifications (TS) in response to your application transmitted by letter dated November 5, 2002, as supplemented February 14 and June 9, 2003.

These amendments revise the TS to delete the monthly analog rod position test for the control rod bottom bistables.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Christopher Gratton, Sr. Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-280 and 50-281

Enclosures:

1. Amendment No. 237 to DPR-32
2. Amendment No. 236 to DPR-37
3. Safety Evaluation

cc w/encls: See next page

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Operating License No. DPR-32 and Amendment No. 236 to Renewed Facility
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ADAMS ACCESSION NUMBER ML032790301

OFFICE	PDII-1/PM	PDII-2/LA	DE/EEIB	OGC	PDII-1/SC
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DATE	8/26/2003	8/20/2003	7 /28 /2003	9/12/2003	10/1/2003

OFFICIAL RECORD COPY

DATED: October 1, 2003

AMENDMENT NO. 237 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-32 -
SURREY UNIT 1

AMENDMENT NO. 236 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-37 -
SURREY UNIT 2

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VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 237
Renewed License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated November 5, 2002, as supplemented February 14 and June 9, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Renewed Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 237, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

John A. Nakoski, Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 1, 2003

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 236
Renewed License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated November 5, 2002, as supplemented February 14 and June 9, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Renewed Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 236, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

John A. Nakoski, Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 1, 2003

ATTACHMENT TO
LICENSE AMENDMENT NO. 237 TO
RENEWED FACILITY OPERATING LICENSE NO. DPR-32
LICENSE AMENDMENT NO. 236 TO
RENEWED FACILITY OPERATING LICENSE NO. DPR-37
DOCKET NOS. 50-280 AND 50-281

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove Page

TS 4.1-6

Insert Page

TS 4.1-6

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 237 TO
RENEWED FACILITY OPERATING LICENSE NO. DPR-32
AND
AMENDMENT NO. 236 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-37
VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-280 AND 50-281

1.0 INTRODUCTION

By application dated November 5, 2002, Virginia Electric & Power Company (VEPCO, the licensee) requested a change to the Surry Power Station, Units 1 and 2, Technical Specifications (TS) related to the Rod Position Indication (RPI) System Surveillance Requirement (SR). The licensee provided additional information to support the NRC staff's review in letters dated February 14 and June 9, 2003. The supplemental letters dated February 14 and June 9, 2003, provided clarifying information only and did not change the initial proposed no significant hazards consideration determination or expand the scope of the initial application. The licensee's November 5, 2002, February 14, and June 9, 2003, letters can be found in the Nuclear Regulatory Commission's (NRC's) Agencywide Documents Access and Management System (ADAMS) at accession numbers ML023180242, ML030580424, and ML031671465, respectively

The proposed change would eliminate the monthly analog rod position test for the control rod bottom bistables from the TS. Specifically, the proposed change would revise TS Table 4.1-1, "Minimum Frequencies for Check, Calibrations and Test of Instrument Channels." Item No. 9, "Analog Rod Position," is being revised to delete the requirement to perform a monthly channel functional test. In the "Remarks" column of Table 4.1-1, Remark No. 3 for Item No. 9, which notes that the monthly test is for verifying the operability of the rod bottom bistables, is being deleted. Remark No. 4 for Item No. 9 will be renumbered as Remark No. 3.

This Safety Evaluation addresses the acceptability of the proposed TS amendment for this modification.

2.0 REGULATORY EVALUATION

The licensee discussed in its application dated November 5, 2002, that equipment and self-test functions similar to those evaluated herein were evaluated by the NRC staff in the Safety Evaluation Report (SER) by the Office of Nuclear Reactor Regulation (NRR) entitled, "Review

of CE Nuclear Power Topical Report CENPD-396-P for the Common Qualified Platform," dated August 11, 2000 (ADAMS Accession Number ML003740165). The NRC staff accepted the use of Common Q in certain applications (i.e., Post Accident Monitoring System, Core Protection Calculator, Reactor Protection System, Plant Protection System, and Engineered Safety Features Actuation System). The applications accepted by the NRC staff did not include those applications proposed by the licensee. However, the licensee proposed that the acceptability in those applications in the NRC staff's August 11, 2000, SER supports the proposed TS change associated with the similar RPI system equipment at Surry Power Station, Units 1 and 2. The licensee also noted that the proposed change is consistent with NUREG-1431, "Standard Technical Specifications - Westinghouse Plants."

Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36(c)(3) requires that tests, calibration, or inspections be performed to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met. To meet this requirement, rod positions must be known with sufficient accuracy to verify alignment limits are preserved and that the core is operating within the group sequence, overlap, design peaking limits, and eject rod worth, and with minimum shutdown margin. The RPI system provides operators with information to ensure the reactor operates within the bounds of the accident analysis assumptions. Rod bottom bistables are used to energize alarms and indicating lights in the main control room, as well as indicating lights at the individual RPI panels. The monthly test of these bistables verifies that the setpoint that actuates the alarms and lights has not drifted from its intended value. The licensee has replaced the RPI system bistables with a digital system with features that make the monthly setpoint check unnecessary.

The NRC staff applied regulatory requirements and guidelines similar to those discussed in the NRC staff's August 11, 2000, SER to determine the acceptability of the proposed TS changes.

3.0 TECHNICAL EVALUATION

The NRC staff has reviewed the licensee's regulatory and technical analyses in support of its proposed license amendment, which are described in the licensee's submittal. The detailed evaluation described in this section supports the conclusion that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

3.1 TS Table 4.1-1

TS Table 4.1-1 specifies the minimum frequencies for performing channel checks, calibrations, and functional tests of instrument channels. Item No. 9 in this table relates to the Analog Rod Position channels associated with the RPI system. In the "Remarks" section of Item No. 9, the TS states that a monthly channel functional test is required to verify proper rod bottom bistable action. This test is performed to ensure that the rod bottom setpoint has not "drifted" from its calibrated value.

The licensee's justification for the deletion of the monthly rod bottom surveillance is the installation of a new computer-enhanced rod position indication (CERPI) system that will

eliminate the concern for setpoint drift. The licensee states that the CERPI system utilizes diagnostic self-tests that will ensure the setpoint has not been altered.

The CERPI system will use the same detectors and shielded wires as the original system. The detectors are linear transformers mounted outside the rod drive pressure housing. The position of the rod drive shaft within the rod position detector determines the amount of magnetic coupling between the primary and secondary windings. The voltage induced on the secondary windings is proportional to rod position. The output of the secondary windings is transmitted to the RPI system cabinets in the control room via shielded wires. The design change will replace the electronic components in the existing Surry Units 1 and 2 RPI system cabinets. The detectors and shielded wires from the analog RPI equipment to the cabinet are not being replaced.

The rod bottom bistables of the original analog RPI system, which are the subject of the monthly surveillance for each control rod, are Magnetics Micro-Sentry modules located in the RPI cabinets. These modules are analog circuit boards that have potentiometers and other electrical components that provide the rod bottom setpoint to which the rod position signal is compared to generate a rod bottom alarm and light indication for the rod experiencing the rod bottom condition.

3.1.1 System Overview

The existing system is being replaced with a CERPI system. This system uses Common Q equipment, but is not safety-related. The Common Q components used in the CERPI system are:

- Advant AC 160 model RF 616 subrack
- Advant AC 160 model RF 620 extension subrack
- Advant AC 160 model CI 631 communications module
- Advant AC 160 model PM 646A processor module
- Advant S 600 model AI 620 analog input module
- Advant S 600 model DI 620 digital input module
- Advant S 600 model DO 620 digital output module
- Advant AC 160 model RB 601 dummy module
- Westinghouse Common Q Power Supply (configured for redundant 24 VDC outputs)
- Westinghouse Maintenance and Test Panel (MTP)
- Westinghouse Operator Flat Panel Display (OFPD) (configured for a 12-inch display)
- Advant AC 160 model TC 514 Fiber Optic Modem (used for communication with the OFPD)

The components that are used in the CERPI system that are not Common Q components are the Advant S 600 model AO 610 analog output module and the Allied Telesyn model AT-MC 102XL fiber optic modem for Fast Ethernet.

The Advant S 600 model AO 610 analog output module is described in the licensee's response to the NRC staff's request for additional information (RAI) dated February 14, 2003 (ADAMS Accession Number ML030580424), as being similar to the Common Q component Model AO 650 analog output module. The licensee further described the standards used in the design, test, and qualification of these non-Common Q components in the RAI response dated June 9, 2003 (ADAMS Accession Number ML031671460). The analog signal conditioning

equipment was designed to the unique requirements in the CERPI System Requirements Specification and the CERPI Interface Requirements Specification. These specifications were reviewed by the licensee and validation testing was performed that demonstrated the compliance of this equipment to the specifications.

The following table is a comparison between the CERPI component proposed by the licensee and the Common Q component previously approved by the NRC staff.

Characteristic	Plant-Specific Interface	CERPI Component Technical Data: AO 610	Common Q Component Technical Data: AO 650
Number of Outputs	4 or more	16	8
Short Circuit and Open Circuit Protection	Required	Open and short circuit protection	Open and short circuit protection
Output Range	0 to +10 VDC	0 to +10 VDC 0 to +21 mA	0 to +5 VDC 0 to +10 VDC +1 to +5 VDC -10 to +10 VDC -20 mA to +20 mA
Isolation	Not required	500 V RMS for each group of 16	500 V RMS for each channel
Overrange	0 to > +10 VDC	0 to 10.5 VDC	0 to 11.5 VDC (listed for range of interest only)
Analog Output Error	±0.43% budgeted for all programmable logic controller equipment	0.1%	0.1%
Resolution	12 bits	12 bits	12 bits plus sign
Output Transfer Time	≤ 2 seconds budgeted for complete system time response	20 milliseconds	8 milliseconds

The NRC staff reviewed the table and determined that both components meet the licensee's plant-specific requirements.

The Allied Telesyn model AT-MC 102XL fiber optic modem for Fast Ethernet does not have a similar Common Q component. The licensee states that the modem provides electrical isolation by converting the ethernet signal from the MTP to a fiber optic signal to the plant

computer interface equipment. It satisfies the Common Q requirement for electrical isolation of the MTP. Although not required for the non-safety CERPI application, this modem was used to allow the cable run to be fiber optic.

3.1.2 Signal Conditioning

Signal conditioning converts the sensor signals to analog control rod position signals and signals that indicate the resistance of each sensor. In its response to the NRC staff's RAI dated February 14, 2003, the licensee listed several self-tests that check the signal conditioning function. These self-tests are oscillator supervision and control, analog input trouble, rod-to-rod deviation alarm, rod-to-control bank deviation alarm, and rod bottom alarm. The self-tests that are not in the existing RPI system are the analog input trouble test and the oscillator supervision and control functional test.

The analog input trouble functional test will actuate a "CERPI Trouble" alarm if the input signal is out of range by being either high or low. A signal conditioning failure could cause this condition on either the rod position analog input or the sensor resistance analog input.

The signal conditioning contains two redundant sets of oscillators, a main oscillator and a backup oscillator. The main oscillator is used to develop the excitation signal for the primary coil of the sensor. The programmable logic controller (PLC) monitors selected rod position signals and detector-resistance signals to determine whether a main oscillator has failed. When a main oscillator failure is detected, the PLC sets a main oscillator failure alarm flag and automatically transfers to the backup oscillator. If the transfer to the backup oscillator does not clear the alarm condition, the PLC sets an oscillator bus failure flag. The PLC also monitors the backup oscillator while the main oscillator is used for sensor excitation. If the backup oscillator fails, the PLC sets a backup oscillator failure alarm flag. If one or more of the alarm flags are set, the PLC will actuate the CERPI Trouble alarm. The maintenance technician can determine which alarm flag was set by utilizing the MTP.

3.1.3 Programmable Logic Controller

The PLC acquires the analog rod position and sensor resistance signals and calculates control rod positions that are corrected for offset, span, linearity, and temperature effects. The PLC also receives control rod bank position motion demand signals and determines demand positions for each bank of rods used for control. The PLC determines system status and alarm conditions. The PLC provides the following interfaces:

- Alarm contact outputs to the annunciator system.
- Network connection to the OFPD and MTP for signal values, calculated values, and system status.
- Signals to the plant computer for rod positions and for a rod bottom condition. Rod bottom signals are provided to the Emergency Response Facility (ERF) computer.

The licensee states that the monthly rod bottom surveillance will be unnecessary because of the additional diagnostic self-test features of the CERPI system. The majority of the diagnostic

self-tests that the licensee lists test the function of the programmable logic controller and are described below.

3.1.3.1 Processor Module Diagnostics

The CERPI system utilizes the AC 160 PM 646A processor module that was approved by the NRC staff in "Safety Evaluation by the Office of Nuclear Reactor Regulation Related to the Westinghouse Common Q Platform Closeout of Generic Open Items and Approve Changes to Topical Report CENPD-396-P, Rev. 01, Common Qualified Platform," dated February 24, 2003 (ADAMS Accession Number ML030550776). The PM 646A diagnostics include the following:

- Test of Central Processing Unit (CPU) Instruction Set - CPU instructions are executed and then CPU registers and corresponding memory locations are verified against expected results. This diagnostic is performed at processor initialization.
- Test of System and User Flash Programmable Read Only Memory (FEPROM) - This test checks the Cyclical Redundancy Check (CRC) checksum of the system software in the system FEPROM and the application in the user FEPROM. This diagnostic is performed at initialization and on-line.
- Random Access Memory (RAM) Test - A single bit is shifted in a double word and then written to memory. After each shift, the double word is read back from memory and compared with the written value. This procedure is performed for all memory locations. This diagnostic is performed at initialization.
- Domain CRC - The CRC checksums of all read-only domains in RAM are verified. This diagnostic is performed on-line.
- Test of Real Time Clock (RTC) - This test verifies the 1-second interrupt generated by the RTC. This diagnostic is performed on-line.
- RAM Check - The RAM on the processor section of the PM 646A is hardware supervised for corrupted data without any loss of performance. A 'mirrored' RAM is used in conjunction with comparator logic (RAM Checker) to perform this function. This diagnostic is performed on-line.

The PM 646A also performs diagnostics on the high-speed link communications function; however, this function is not used in the CERPI application. The processor module diagnostics will actuate the CERPI Trouble alarm if a failure is detected.

3.1.3.2 Input/Output (I/O) Module Diagnostics

The CERPI system utilizes the Advant S600 I/O product family for I/O. CERPI utilizes the AI 620 analog input module, the DI 620 digital input module, and the DO 620 digital output module. All of these module types are identical to the modules utilized for Common Q. CERPI also utilizes a model AO 610 analog output module that is different than the analog output module utilized for Common Q.

Each S600 I/O module performs self-diagnostics. The module diagnostics determine that the module is in the correct position, the module is of the right type, the module process connector is in place, and the module is not defective.

If the I/O module diagnostics detect a fault, a flag is set indicating an error. The processor module monitors the error flags for each I/O module and actuates the CERPI Trouble alarm if an error is detected.

3.1.3.3 Watchdog Timer and Watchdog Timer Diagnostic

The watchdog timer monitors the real time operation of the PLC. If the PLC halts or slows down, the watchdog timer actuates a relay that will result in the CERPI Trouble alarm. The watchdog timer is a function of the PM 646A module and was approved by the NRC staff in a letter dated February 24, 2003, "Acceptance of the Changes to Topical Report CENPD-396-P, Rev. 01, 'Common Qualified Platform,' and Closeout of Category 2 Open Items."

3.1.4 Operator Flat Panel Display

The OFPD receives rod position information from the PLC over the AF100 network and provides a color graphic, touch screen, and flat panel display interface for the reactor operator. The OFPDs are located in the control room. Fiber optic cable is utilized as the communication media between the PLC and the OFPD. The OFPD displays have a dynamically updated symbol to indicate OFPD health. The OFPD was approved by the NRC staff in a letter dated August 11, 2000, "Review of CE Nuclear Power Topical Report CENPD-396-P for the Common Qualified Platform."

3.1.5 Maintenance and Test Panel

The MTP acquires information from both PLCs over the AF100 network. The MTP provides an interface for the maintenance technician to perform calibration, surveillance, and fault diagnosis. The MTP also provides an interface for transmitting information to the plant computer. The MTP displays the date and time as an indication of MTP health. The MTP was approved by the NRC staff in a letter dated August 11, 2000, "Review of CE Nuclear Power Topical Report CENPD-396-P for the Common Qualified Platform."

3.1.6 Setpoint Drift

The licensee states that the monthly analog rod position test verifying the operation of the rod bottom bistables is being performed to address instrument drift in the rod bottom setpoint. The analog system is currently calibrated once every 18 months. A potentiometer on the circuit board is adjusted accordingly. This test is performed for every detector. Typically, the analog RPI system experiences significant setpoint drift because the signal from the detector is not linearized or temperature compensated. The replacement system will provide signal linearization and temperature compensation.

To verify that setpoint drift will not be an issue in the CERPI system, a detailed evaluation of how the setpoint is stored in memory is necessary. The NRC staff must be assured that the setpoint will not be overwritten, lost, or corrupted.

A key is required to unlock the MTP to allow setpoint alteration. The setpoint is entered on the keyboard of the MTP. Once the setpoint is entered on the MTP, it is stored in flash programmable ROM. The MTP sends the setpoint to the PLCs every half second, where it is stored in the PLC RAM. The MTP and PLC were reviewed in the SE by the Office of NRR, "Review of CE Nuclear Power Topical Report CENPD-396-P for the Common Qualified Platform," contained in the NRC letter dated August 11, 2000. Based on our review of the design and operation of the modified RPI system, the NRC staff concludes that there is reasonable assurance the setpoint will not be changed, lost, or corrupted.

3.2 Technical Evaluation Summary

The rod bottom bistables, which were the component checked for drift during the monthly setpoint check, are among the components replaced with the new CERPI system. The CERPI system, being a digital system, is not subject to drift as the bistables were, and therefore, do not need this type of surveillance. The NRC staff determined the once-per-shift check that verifies rod position (by comparing RPI and group step demand counter positions) and the 18-month channel calibration, in addition to the systems continuous self-test capability, will continue to provide assurance of the operability of the RPI system and, therefore, meets the requirements of 10 CFR 50.36(c)(3). Therefore, the NRC staff concludes that the CERPI system is acceptable for use at the Surry Power Station, Units 1 and 2, and that the monthly TS SR to ensure rod bottom bistable operability, specifically, Remark No. 3 of Item 9 of TS Table 4.1-1, is no longer required.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia State official was notified of the proposed issuance of the amendments. The State official had no comment.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a surveillance requirement with respect to the use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (68 FR 5683). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Tifft, EEIB:DE
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Date: October 1, 2003

Mr. David A. Christian
Virginia Electric and Power Company

Surry Power Station
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