

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>	
3/4.9.4	CONTAINMENT BUILDING PENETRATIONS.....	3/4 9-4
3/4.9.5	(THIS SPECIFICATION NUMBER IS NOT USED).....	3/4 9-5
3/4.9.6	(THIS SPECIFICATION NUMBER IS NOT USED).....	3/4 9-6
3/4.9.7	(THIS SPECIFICATION NUMBER IS NOT USED).....	3/4 9-7
3/4.9.8	RESIDUAL HEAT REMOVAL AND COOLANT CIRCULATION	
	High Water Level	3/4 9-8
	Low Water Level	3/4 9-9
3/4 9.9	CONTAINMENT PURGE AND EXHAUST ISOLATION SYSTEM	3/4 9-10
3/4 9.10	WATER LEVEL - REACTOR VESSEL	3/4 9-11
3/4 9.11	WATER LEVEL - STORAGE POOL	3/4 9-12
3/4 9.12	FUEL STORAGE BUILDING EMERGENCY AIR CLEANING SYSTEM	3/4 9-13
3/4 9.13	SPENT FUEL ASSEMBLY STORAGE	3/4 9-16
FIGURE 3.9-1	FUEL ASSEMBLY BURNUP VS. INITIAL ENRICHMENT FOR SPENT FUEL ASSEMBLY STORAGE	3/4 9-17
3/4.9.14	NEW FUEL ASSEMBLY STORAGE	3/4 9-18
<u>3/4.10</u>	<u>SPECIAL TEST EXCEPTIONS</u>	
3/4.10.1	SHUTDOWN MARGIN	3/4 10-1
3/4.10.2	GROUP HEIGHT, INSERTION, AND POWER DISTRIBUTION LIMITS	3/4 10-2
3/4.10.3	PHYSICS TESTS	3/4 10-3
3/4.10.4	(THIS SPECIFICATION NUMBER IS NOT USED)	3/4 10-4
3/4.10.5	POSITION INDICATION SYSTEM- SHUTDOWN	3/4 10-5
<u>3/4.11</u>	<u>RADIOACTIVE EFFLUENTS</u>	
3/4.11.1	LIQUID EFFLUENTS	
	(THIS SPECIFICATION NUMBER IS NOT USED)	3/4 11-1
	(THIS SPECIFICATION NUMBER IS NOT USED)	3/4 11-2
	(THIS SPECIFICATION NUMBER IS NOT USED)	3/4 11-3
	Liquid Holdup Tanks	3/4 11-4
3/4.11.2	GASEOUS EFFLUENTS	
	(THIS SPECIFICATION NUMBER IS NOT USED)	3/4 11-5
	(THIS SPECIFICATION NUMBER IS NOT USED)	3/4 11-6
	(THIS SPECIFICATION NUMBER IS NOT USED)	3/4 11-7
	(THIS SPECIFICATION NUMBER IS NOT USED)	3/4 11-8
	Explosive Gas Mixture - System	3/4 11-9
3/4.11.3	(THIS SPECIFICATION NUMBER IS NOT USED).....	3/4 11-10
3/4.11.4	(THIS SPECIFICATION NUMBER IS NOT USED)	3/4 11-12
<u>3/4.12</u>	<u>RADIOLOGICAL ENVIRONMENTAL MONITORING</u>	
3/4.12.1	(THIS SPECIFICATION NUMBER IS NOT USED).....	3/4 12-1

INDEX

BASES

<u>SECTION</u>	<u>PAGE</u>
3/4.9.8 RESIDUAL HEAT REMOVAL AND COOLANT CIRCULATION	B 3/4 9-2
3/4.9.9 CONTAINMENT PURGE AND EXHAUST ISOLATION SYSTEM.....	B 3/4 9-2
3/4.9.10 and 3/4.9.11 WATER LEVEL - REACTOR VESSEL and STORAGE POOL	B 3/4 9-2
3/4.9.12 FUEL STORAGE BUILDING EMERGENCY AIR CLEANING SYSTEM	B 3/4 9-2
3/4.9.13 SPENT FUEL ASSEMBLY STORAGE	B 3/4 9-3
3/4.9.14 NEW FUEL ASSEMBLY STORAGE.....	B 3/4 9-3

3/4.10 SPECIAL TEST EXCEPTIONS

3/4.10.1 SHUTDOWN MARGIN	B 3/4 10-1
3/4.10.2 GROUP HEIGHT, INSERTION, AND POWER DISTRIBUTION LIMITS	B 3/4 10-1
3/4.10.3 PHYSICS TESTS	B 3/4 10-1
3/4.10.4 (THIS SPECIFICATION NUMBER IS NOT USED)	B 3/4 10-1
3/4.10.5 POSITION INDICATION SYSTEM - SHUTDOWN.....	B 3/4 10-1
3/4.10.6 REACTOR COOLANT LOOPS	B 3/4 10-1

3/4.11 RADIOACTIVE EFFLUENTS

3/4.11.1 (THIS SPECIFICATION NUMBER IS NOT USED)	B 3/4 11-1
3/4.11.2 GASEOUS EFFLUENTS.....	B 3/4 11-2
3/4.11.3 (THIS SPECIFICATION NUMBER IS NOT USED)	B 3/4 11-5
3/4.11.4 (THIS SPECIFICATION NUMBER IS NOT USED)	B 3/4 11-5

3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

3/4.12.1 (THIS SPECIFICATION NUMBER IS NOT USED)	B 3/4 12-1
3/4.12.2 (THIS SPECIFICATION NUMBER IS NOT USED)	B 3/4 12-1
3/4.12.3 (THIS SPECIFICATION NUMBER IS NOT USED)	B 3/4 12-2

5.0 DESIGN FEATURES

5.1 SITE

5.1.1 EXCLUSION AREA	5-1
5.1.2 LOW POPULATION ZONE	5-1
5.1.3 MAPS DEFINING UNRESTRICTED AREAS AND SITE BOUNDARY FOR RADIOACTIVE GASEOUS AND LIQUID EFFLUENTS	5-1
FIGURE 5.1-1 SITE AND EXCLUSION AREA BOUNDARY	5-3
FIGURE 5.1-2 LOW POPULATION ZONE	5-5
FIGURE 5.1-3 LIQUID EFFLUENT DISCHARGE LOCATION.....	5-7

5.2 CONTAINMENT

5.2.1 CONFIGURATION.....	5-1
5.2.2 DESIGN PRESSURE AND TEMPERATURE.....	5-9

TABLE 2.2-1
REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TOTAL ALLOWANCE (TA)</u>	<u>Z</u>	<u>SENSOR ERROR (S)</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. Manual Reactor Trip	N.A.	N.A.	N.A.	N.A.	N.A.
2. Power Range, Neutron Flux					
a. High Setpoint	7.5	4.56	1.42	≤109% of RTP*	≤111.1% of RTP*
b. Low Setpoint	8.3	4.56	1.42	≤25% of RTP*	≤27.1% of RTP*
3. Power Range, Neutron Flux, High Positive Rate	1.6	0.5	0	≤5% of RTP* with a time constant ≥2 seconds	≤6.3% of RTP* with a time constant ≥2 seconds
4. NOT USED					
5. Intermediate Range, Neutron Flux	17.0	8.41	0	≤25% of RTP*	≤31.1% of RTP*
6. Source Range, Neutron Flux	17.0	10.01	0	≤10 ⁵ cps	≤1.6 x 10 ⁵ cps
7. Overtemperature ΔT	N.A.	N.A.	N.A.	See Note 1	See Note 2
8. Overpower ΔT	N.A.	N.A.	N.A.	See Note 3	See Note 4
9. Pressurizer Pressure - Low	N.A.	N.A.	N.A.	≥1945 psig	≥1,933 psig
10. Pressurizer Pressure - High	N.A.	N.A.	N.A.	≤2385 psig	≤2,397 psig

*RTP = RATED THERMAL POWER

TABLE 3.3-1

REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
1. Manual Reactor Trip	2	1	2	1, 2	1
	2	1	2	3*, 4*, 5*	10
2. Power Range, Neutron Flux					
a. High Setpoint	4	2	3	1, 2	2
b. Low Setpoint	4	2	3	1###, 2	2
3. Power Range, Neutron Flux High Positive Rate	4	2	3	1, 2	2
4. NOT USED					
5. Intermediate Range, Neutron Flux	2	1	2	1###, 2	3
6. Source Range, Neutron Flux					
a. Startup	2	1	2	2##	4
b. Shutdown	2	0	1	3, 4, 5	5
c. Shutdown	2	1	2	3*, 4*, 5*	10
7. Overtemperature ΔT	4	2	3	1, 2	6
8. Overpower ΔT	4	2	3	1, 2	6
9. Pressurizer Pressure—Low	4	2	3	1**	6
10. Pressurizer Pressure—High	4	2	3	1, 2	6
11. Pressurizer Water Level—High	3	2	2	1**	6#

TABLE 4.3-1

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>TRIP ACTUATING DEVICE OPERATIONAL TEST</u>	<u>ACTUATION LOGIC TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
1. Manual Reactor Trip	N.A.	N.A.	N.A.	R(13)	N.A.	1,2,3*,4*,5*
2. Power Range, Neutron Flux						
a. High Setpoint	S	D(2, 4), M(3, 4), Q(4, 6), R(4, 5)	Q	N.A.	N.A.	1, 2
b. Low Setpoint	S	R(4)	S/U(1)	N.A.	N.A.	1***, 2
3. Power Range, Neutron Flux, High Positive Rate	N.A.	R(4)	Q	N.A.	N.A.	1, 2
4. NOT USED						
5. Intermediate Range, Neutron Flux	S	R(4, 5)	S/U(1)	N.A.	N.A.	1***, 2
6. Source Range, Neutron Flux	S	R(4, 5)	S/U(8),Q(9)	N.A.	N.A.	2**, 3, 4, 5
7. Overtemperature ΔT	S	R	Q	N.A.	N.A.	1, 2
8. Overpower ΔT	S	R	Q	N.A.	N.A.	1, 2
9. Pressurizer Pressure--Low	S	R	Q	N.A.	N.A.	1
10. Pressurizer Pressure--High	S	R	Q	N.A.	N.A.	1, 2
11. Pressurizer Water Level--High	S	R	Q	N.A.	N.A.	1
12. Reactor Coolant Flow--Low	S	R	Q	N.A.	N.A.	1

TABLE 4.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>TRIP ACTUATING DEVICE OPERATIONAL TEST</u>	<u>ACTUATION LOGIC TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
13. Steam Generator Water Level—Low-Low	S	R	Q	N.A.	N.A.	1, 2
14. Undervoltage - Reactor Coolant Pumps	N.A.	R	N.A.	Q	N.A.	1
15. Underfrequency - Reactor Coolant Pumps	N.A.	R	N.A.	Q	N.A.	1
16. Turbine Trip						
a. Low Fluid Oil Pressure	N.A.	R	N.A.	S/U(8, 10)	N.A.	1
b. Turbine Stop Valve	N.A.	R	N.A.	S/U(8, 10)	N.A.	1
17. Safety Injection Input from ESF	N.A.	N.A.	N.A.	R	N.A.	1, 2
18. Reactor Trip System Interlocks						
a. Intermediate Range Neutron Flux, P-6	N.A.	R(4)	R	N.A.	N.A.	2**
b. Low Power Reactor Trips Block, P-7	N.A.	R(4)	R	N.A.	N.A.	1
c. Power Range Neutron Flux, P-8	N.A.	R(4)	R	N.A.	N.A.	1
d. Power Range Neutron Flux, P-9	N.A.	R(4)	R	N.A.	N.A.	1

TABLE 4.3-1 (Continued)

TABLE NOTATIONS

***Only if the Reactor Trip System breakers happen to be closed and the Control Rod Drive System is capable of rod withdrawal.**

****Below P-6 (Intermediate Range Neutron Flux Interlock) Setpoint.**

*****Below P-10 (Low Setpoint Power Range Neutron Flux Interlock) Setpoint.**

- (1) If not performed in previous 92 days.**
- (2) Comparison of calorimetric to excore power indication above 15% of RATED THERMAL POWER. Adjust excore channel gains consistent with calorimetric power if absolute difference is greater than 2%. The provisions of Specification 4.0.4 are not applicable to entry into MODE 2 or 1.**
- (3) Single point comparison of Incore to excore AXIAL FLUX DIFFERENCE above 50% of RATED THERMAL POWER. Recalibrate if the absolute difference is greater than or equal to 3%. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1. For the purposes of this surveillance requirement, monthly shall mean at least once per 31 EFPD.**
- (4) Neutron detectors may be excluded from CHANNEL CALIBRATION.**
- (5) Initial plateau curves shall be measured for each detector. Subsequent plateau curves shall be obtained, evaluated and compared to the initial curves. For the Intermediate Range and Power Range Neutron Flux channels the provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.**
- (6) Incore - Excore Calibration, above 75% of RATED THERMAL POWER. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1. For the purposes of this surveillance requirement, quarterly shall mean at least once per 92 EFPD.**
- (7) Each train shall be tested at least every 62 days on a STAGGERED TEST BASIS.**
- (8) If not performed in previous 31 days.**
- (9) Surveillance in MODES 3*, 4*, and 5* shall also include verification that permissives P-6 and P-10 are in their required state for existing plant conditions by observation of the permissive annunciator window.**
- (10) Setpoint verification is not applicable.**
- (11) The TRIP ACTUATING DEVICE OPERATIONAL TEST shall independently verify the OPERABILITY of the undervoltage and shunt trip attachments of the Reactor Trip Breakers.**

REACTOR COOLANT SYSTEM

3/4.4.1 REACTOR COOLANT LOOPS AND COOLANT CIRCULATION

STARTUP AND POWER OPERATION

LIMITING CONDITION FOR OPERATION

3.4.1.1 All reactor coolant loops shall be in operation.

APPLICABILITY: MODES 1 and 2.

ACTION:

With less than the above required reactor coolant loops in operation, be in at least HOT STANDBY within 6 hours.

SURVEILLANCE REQUIREMENTS

4.4.1.1 The above required reactor coolant loops shall be verified in operation and circulating reactor coolant at least once per 12 hours.

SPECIAL TEST EXCEPTIONS

3/4.10.3 PHYSICS TESTS

LIMITING CONDITION FOR OPERATION

3.10.3 The limitations of Specifications 3.1.1.3, 3.1.1.4, 3.1.3.1, 3.1.3.5, and 3.1.3.6 may be suspended during the performance of PHYSICS TESTS provided:

- a. The THERMAL POWER does not exceed 5% of RATED THERMAL POWER,
- b. The Reactor Trip Setpoints on the OPERABLE Intermediate Range and Power Range* channels are set at less than or equal to 25% of RATED THERMAL POWER, and
- c. The Reactor Coolant System lowest operating loop temperature (T_{avg}) is greater than or equal to 541°F.

APPLICABILITY: MODE 2.

ACTION:

- a. With the THERMAL POWER greater than 5% of RATED THERMAL POWER, immediately open the Reactor trip breakers.
- b. With a Reactor Coolant System operating loop temperature (T_{avg}) less than 541°F, restore T_{avg} to within its limit within 15 minutes or be in at least HOT STANDBY within the next 15 minutes.

SURVEILLANCE REQUIREMENTS

4.10.3.1 The THERMAL POWER shall be determined to be less than or equal to 5% of RATED THERMAL POWER at least once per hour during PHYSICS TESTS.

4.10.3.2 Verify each OPERABLE Intermediate Range and Power Range* channel has been subjected to an ANALOG CHANNEL OPERATIONAL TEST per Specification Table 4.3-1 prior to initiating PHYSICS TESTS.

4.10.3.3 The Reactor Coolant System temperature (T_{avg}) shall be determined to be greater than or equal to 541°F at least once per 30 minutes during PHYSICS TESTS.

* Power Range Low Setpoint only.

SPECIAL TEST EXCEPTIONS

3/4.10.4 (THIS SPECIFICATION NUMBER IS NOT USED)

LIMITING SAFETY SYSTEM SETTINGS

BASES

2.2.1 REACTOR TRIP SYSTEM INSTRUMENTATION SETPOINTS (Continued)

The various Reactor trip circuits automatically open the Reactor trip breakers whenever a condition monitored by the Reactor Trip System reaches a preset or calculated level. In addition to redundant channels and trains, the design approach provides a Reactor Trip System which monitors numerous system variables, therefore providing Trip System functional diversity. The functional capability at the specified trip setting is required for those anticipatory or diverse Reactor trips for which no direct credit was assumed in the safety analysis to enhance the overall reliability of the Reactor Trip System. The Reactor Trip System initiates a Turbine trip signal whenever Reactor trip is initiated. This prevents the reactivity insertion that would otherwise result from excessive Reactor Coolant System cooldown and thus avoids unnecessary actuation of the Engineered Safety Features Actuation System.

Manual Reactor Trip

The Reactor Trip System includes manual Reactor trip capability.

Power Range, Neutron Flux

In each of the Power Range Neutron Flux channels there are two independent bistables, each with its own trip setting used for a High and Low Range trip setting. The Low Setpoint trip provides protection during subcritical and low power operations to mitigate the consequences of a power excursion beginning from low power, and the High Setpoint trip provides protection during power operations to mitigate the consequences of a reactivity excursion from all power levels.

The Low Setpoint trip may be manually blocked above P-10 (a power level of approximately 10% of RATED THERMAL POWER) and is automatically reinstated below the P-10 Setpoint.

Power Range, Neutron Flux, High Positive Rate

The Power Range High Positive Rate trip provides protection against rapid flux increases which are characteristic of a rupture of a control rod drive housing. Specifically, this trip complements the Power Range Neutron Flux High and Low trips to ensure that the criteria are met for rod ejection from mid-power.

3/4.10. SPECIAL TEST EXCEPTIONS

BASES

3/4.10.1 SHUTDOWN MARGIN

This special test exception provides that a minimum amount of control rod worth is immediately available for reactivity control when tests are performed for control rod worth measurement. This special test exception is required to permit the periodic verification of the actual versus predicted core reactivity condition occurring as a result of fuel burnup or fuel cycling operations.

3/4.10.2 GROUP HEIGHT, INSERTION, AND POWER DISTRIBUTION LIMITS

This special test exception permits individual control rods to be positioned outside of their normal group heights and insertion limits during the performance of such PHYSICS TESTS as those required to: (1) measure control rod worth and (2) determine the reactor stability index and damping factor under xenon oscillation conditions.

3/4.10.3 PHYSICS TESTS

This special test exception permits PHYSICS TESTS to be performed at less than or equal to 5% of RATED THERMAL POWER with the RCS T_{avg} slightly lower than normally allowed so that the fundamental nuclear characteristics of the core and related instrumentation can be verified. In order for various characteristics to be accurately measured, it is at times necessary to operate outside the normal restrictions of these Technical Specifications. For instance, to measure the moderator temperature coefficient at BOL, it is necessary to position the various control rods at heights which may not normally be allowed by Specification 3.1.3.6 and the RCS T_{avg} may be below the minimum temperature of Specification 3.1.1.4 during the measurement.

3/4.10.4 (THIS SPECIFICATION NUMBER IS NOT USED)

3/4.10.5 POSITION INDICATION SYSTEM - SHUTDOWN

This special test exception permits the Position Indication Systems to be inoperable during rod drop time measurements. The exception is required since the data necessary to determine the rod drop time are derived from the induced voltage in the position indicator coils as the rod is dropped. This induced voltage is small compared to the normal voltage and, therefore, cannot be observed if the Position Indication Systems remain OPERABLE.