



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET SW SUITE 23T85
ATLANTA, GEORGIA 30303-8931**

September 29, 2003

Florida Power and Light Company
ATTN: Mr. J. A. Stall, Senior Vice President
Nuclear and Chief Nuclear Officer
P. O. Box 14000
Juno Beach, FL 33408-0420

**SUBJECT: NOTIFICATION OF ST. LUCIE NUCLEAR PLANT - SAFETY SYSTEM DESIGN
AND PERFORMANCE CAPABILITY INSPECTION - NRC INSPECTION
REPORT 05000335/2003009 AND 05000389/2003009**

Dear Mr. Stall:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a safety system design and performance capability inspection at your St. Lucie Nuclear Plant during the weeks of December 1, 2003, and December 15, 2003. A team of seven inspectors will perform this inspection. The inspection team will be led by Mr. Jim Moorman, a Senior Reactor Inspector from the NRC Region II Office. This biennial inspection will be conducted in accordance with baseline pilot inspection program Attachment 71111.DS, "Safety System Design and Performance Capability."

The inspection will evaluate the capability of installed plant equipment to detect and respond to a steam generator tube rupture event. Procedures which direct the mitigating actions for this event will also be evaluated.

During a telephone conversation on September 26, 2003, Mr. Moorman of my staff, and Mr. George Madden of your staff, confirmed arrangements for an information gathering site visit and the two-week onsite inspection. The schedule is as follows:

- Information gathering visit: Week of October 27, 2003
- Onsite inspection weeks: December 1, 2003, and December 15, 2003.

The purpose of the information gathering visit is to obtain information and documentation outlined in the enclosure needed to support the inspection. Mr. Rudy Bernhard, a Region II Senior Reactor Analyst, may accompany Mr. Moorman during the information gathering visit to review probabilistic risk assessment data and identify risk significant components which will be examined during the inspection. Please contact Mr. Moorman prior to preparing copies of the materials listed in the Enclosure. The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for inspection preparation. During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space; specific documents requested to be made available to the team in their office space; arrangements for site access; and the availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection.

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Moorman at (404) 562-4647 or me at (404) 562-4605.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publically Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Charles R. Ogle, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos.: 50-335, 50-389
License Nos.: DPR-67, NPF-16

Enclosure: Information Request for the Safety System Design and Performance Capability Inspection

cc w/encl:
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(cc w/encl cont'd)

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Distribution w/encl: (See page 4)

FP&L

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SIGNATURE	RA	RA					
NAME	MOORMAN	MUNDAY					
DATE	9/26/2003	9/26/2003	10/ /2003	10/ /2003	10/ /2003	10/ /2003	10/ /2003
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO
PUBLIC DOCUMENT	YES NO						

OFFICIAL RECORD COPY

DOCUMENT NAME: C:\ORPCheckout\FileNET\ML032730918.wpd

**INFORMATION REQUEST FOR THE SAFETY SYSTEM DESIGN AND
PERFORMANCE CAPABILITY INSPECTION
ST. LUCIE STEAM GENERATOR TUBE RUPTURE EVENT**

(The preferred file format is searchable “.pdf” files on CDROM. The CDROM should be indexed and hyperlinked to facilitate ease of use. Please provide 5 copies of each CDROM submitted. Information in “lists” should contain enough information to be easily understood by someone who has a knowledge of pressurized water reactor technology.)

1. Piping and instrumentation drawings for High Pressure Safety Injection System (HPSI), Auxiliary Feedwater System (AFW), Main Steam (MS), Chemical and Volume Control System (CVCS), Component Cooling Water System (CCW), Low Pressure Safety Injection System (LPSI), Reactor Coolant System (RCS) (Paper copies are preferred for these)
2. The normal operating procedures for the systems listed above. Include the procedures which specify the normal valve alignment for these systems.
3. Surveillance procedures used to ensure the operability of equipment required by your Technical Specifications that is used during the mitigation of the SGTR event.
4. Procedures used for the operational testing of check valves in portions of the emergency core cooling systems used during mitigation of the SGTR event.
5. Procedures used to sample the steam generators during a SGTR event.
6. Calibration and functional testing procedures for the radiation monitoring instrumentation used during a SGTR event.
7. Calibration and functional test procedures for instruments used to monitor RCS pressure, pressurizer level and pressure, SG level and pressure, RCS hot and cold leg temperature, RCS subcooling, feedwater flow, steam flow, core exit temperature, high pressure injection flow, LPSI flow, refueling water storage tank level, pressurized heater status, safety relief valve position indicator, AFW flow, condensate storage tank (CST) level, makeup flow, and letdown flow.
8. Test procedures for the primary and secondary system safety relief valves and power operated relief valves.
9. Test procedures for the AFW starting logic.
10. A list of Condition Reports and non-routine work requests initiated since 1998 affecting the systems used to detect and mitigate a SGTR event; including their power supplies.
11. Emergency Operating Procedures (EOPs) and supporting procedures, EOP basis documents, step deviation document, writers guide, and users guide.

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12. Calculations used to support the set points in Emergency Operating Procedures for a SGTR event.
13. A list of engineering calculations applicable to AFW, HPSI, LPSI, RCS, CCW, MS and other systems used in the mitigation of the SGTR event.
14. A list of existing temporary modifications and operator work-arounds.
15. A list of operability determinations performed since 1998.
16. System Health Reports and System Performance Trends for all systems going back 2 years.
17. A copy of the Maintenance Rule program procedure and performance criteria for all plant systems. A list of plant systems currently monitored under 50.65(a)(1) and the performance goals. A list of Maintenance Rule functional failures since 2000.
18. Quality Assurance audits, self-assessments, and third party assessments performed on AFW, HPSI, LPSI, RCS, CCW systems and other related systems in the last 24 months. Quality Assurance audits, self-assessments and third party assessments performed on engineering and maintenance department activities in the last 24 months.
19. Operator training lesson plans, system descriptions, and job performance measures for the EOPs, EOP support procedures, off-normal and operating procedures that would be used to mitigate an SGTR event.
20. Key electrical single line drawings of the intermediate and low voltage alternating current and direct current power systems. (Paper copies are preferred for these)
21. The electrical system load list(s).
22. A brief description of the mitigation strategy for handling the SGTR event, including operator actions and equipment used.
23. A list of Operating Experience Program evaluations of industry, vendor, or NRC generic issues for the past 3 years.
24. A list of equipment and operator actions with a Risk Achievement Worth greater than 1.02.
25. Probabilistic Risk Assessment (PRA) Event Tree for SGTR initiating event. A list of PRA system dependencies and success criteria for AFW, HPSI, MS, CCW, and their support systems.
26. Design Basis Manual for major plant electrical, mechanical, and control systems.
27. Plant Technical Specifications, Bases, and Technical Requirements Manual

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28. A current copy of the Updated Final Safety Analysis Report.
29. A list of 50.59 evaluations and screenings conducted since 2001. The plant procedure which defines the St. Lucie 50.59 process.
30. A list of all permanent plant changes, design changes, setpoint changes, procedure changes, equivalency evaluations, suitability analyses, calculations, and commercial grade dedications since 2001. The plant procedure which defines the design control and modifications process at St. Lucie.
31. Plant procedures that provide the guidelines for the Corrective Action Program and the incorporation of local and industry operating experience into St. Lucie programs and procedures.

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