

September 25, 2003

Dr. Robert C. Mecredy
Vice President, Nuclear Operations
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89 East Avenue
Rochester, NY 14649

SUBJECT: SUMMARY OF THE U.S. NUCLEAR REGULATORY COMMISSION (NRC)
STAFF'S REVIEW OF THE R. E. GINNA NUCLEAR POWER PLANT (GINNA)
STEAM GENERATOR TUBE INSPECTION REPORT DATED JULY 2002
(TAC NO. MB6467)

Dear Dr. Mecredy:

By letter dated July 9, 2002, as supplemented October 17, 2002, and May 29, 2003, Rochester Gas and Electric Corporation (RG&E, the licensee) submitted reports summarizing the steam generator (SG) tube inspections performed during the July 2002 refueling outage for Ginna.

The NRC staff's review of the SG tube inspection report is enclosed. The staff concluded that you have provided the required information, and that at this time, no additional information is required.

Sincerely,

/RA/

Robert L. Clark, Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure: As stated

cc w/encl: See next page

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R.E. Ginna Nuclear Power Plant

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R. E. GINNA NUCLEAR POWER PLANT
ROCHESTER GAS AND ELECTRIC CORPORATION
STAFF ASSESSMENT OF JULY 2002
STEAM GENERATOR TUBE INSPECTION REPORT

By letter dated July 9, 2002, as supplemented October 17, 2002, and May 29, 2003, Rochester Gas and Electric Corporation (RG&E, the licensee) submitted reports summarizing the steam generator (SG) tube inspections performed during the end of cycle (EOC) No. 29 (July 2002) refueling outage for R. E. Ginna Nuclear Power Plant (Ginna). These reports were submitted in accordance with American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI. A summary of the NRC staff's review of the July 2002 refueling outage tube inspection results is provided below.

The Ginna unit has two SGs which are designated A and B. Both of the SGs were inspected during the July 2002 refueling outage. In spring 1996, the original SGs were replaced with Babcock and Wilcox International replacement SGs. The replacement SGs have 4765 total tubes which have an outside diameter of 0.749 inch and a wall thickness of 0.044 inch. The tubes have been hydraulically expanded into the tubesheet and the stainless steel Type 410 tube support plates are lattice grid type. The replacement SGs were placed in operation in June 1996 and have thermally-treated Inconel 690 tubing.

The SG tube inspection summary stated that the licensee inspected 50% of the full length tubes in SGs A and B (tube end cold (TEC) to tube end hot (TEH)) using a bobbin probe. A +Point™ probe was used to inspect 20% of the hot-leg expansion transition region and 20% of the Row 1 and Row 2 tubes in the U-bend region (top hot leg support to top cold leg support) in each SG. Other SG tube examinations included visual examination of repaired welded plugs, +Point™ probe examination of outer radius tube to tube proximity tubes, and diagnostic +Point™ probe examinations. The licensee reported that no degradation mechanisms were identified for SGs A and B.

The licensee discussed degradation models, structural limits, detection techniques, probability of detection, condition monitoring limits, and operational assessment limits for potential degradation mechanisms based on industry experience with thermally-treated Inconel 690 tubing.

The staff did not review the condition monitoring and operational limits in detail, rather the staff focused on the scope and results of the inspection. Based on the staff's review of the information provided by the licensee, the staff concludes that the licensee provided the required information, and that at this time no additional information is required.

Enclosure