September 10, 2003

Mr. D. M. Jamil
Vice President
Catawba Nuclear Station
Duke Energy Corporation
4800 Concord Road
York, South Carolina 29745-9635

SUBJECT: CATAWBA NUCLEAR STATION, UNITS 1 AND 2 RE: ISSUANCE OF AMENDMENTS (TAC NOS. MB3747 AND MB3748)

Dear Mr. Jamil:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 208 to Facility Operating License NPF-35 and Amendment No. 202 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated December 20, 2001, as supplemented by letters dated March 4, 2002, September 12, 2002, November 20, 2002, and August 28, 2003.

The requested changes would revise TS 3.3.2, “Engineered Safety Features Actuation System Instrumentation.”

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Robert E. Martin, Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures:
1. Amendment No. 208 to NPF-35
2. Amendment No. 202 to NPF-52
3. Safety Evaluation

cc w/encls: See next page

September 10, 2003

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DATE	09/09/03	09/09/03	09/02/03	09/02/03	09/09/03

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DUKE ENERGY CORPORATION

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 208
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Energy Corporation, acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc. (licensees), dated December 20, 2001, March 4, 2002, September 12, 2002, November 20, 2002, and August 28, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 208, which are attached hereto, are hereby incorporated into this license. Duke Energy Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance. The implementation of this amendment shall include the relocation of certain technical specification requirements to the Catawba Nuclear Station, Unit Nos. 1 and 2, Selected Licensee Commitments manual as described in the licensee's application dated December 20, 2001, and evaluated in the NRC staff's Safety Evaluation attached to this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

John A. Nakoski, Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: September 10, 2003

DUKE ENERGY CORPORATION

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 202
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Energy Corporation, acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (licensees), dated December 20, 2001, March 4, 2002, September 12, 2002, November 20, 2002, and August 28, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 202, which are attached hereto, are hereby incorporated into this license. Duke Energy Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance. The implementation of this amendment shall include the relocation of certain technical specification requirements to the Catawba Nuclear Station, Unit Nos. 1 and 2, Selected Licensee Commitments manual as described in the licensee's application dated December 20, 2001, and evaluated in the NRC staff's Safety Evaluation attached to this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

John A. Nakoski, Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: September 10, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 208

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

AND LICENSE AMENDMENT NO. 202

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Appendix A Technical Specifications and associated Baes with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
3.3.2-10	3.3.2-10
3.3.2-13	3.3.2-13
3.3.2-14	3.3.2-14
3.3.2-15	3.3.2-15
B3.3.2-19	B3.3.2-19
through	through
B3.3.2-30	B3.3.2-30
B3.3.2-33	B3.3.2-33
B3.3.2-36	B3.3.2-36
through	through
B3.3.2-49	B3.3.2-49

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 208 TO FACILITY OPERATING LICENSE NPF-35

AND AMENDMENT NO. 202 TO FACILITY OPERATING LICENSE NPF-52

DUKE ENERGY CORPORATION, ET AL.

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-413 AND 50-414

1.0 INTRODUCTION

By letter dated December 20, 2001, as supplemented by letters dated March 4, 2002, September 12, 2002, November 20, 2002, and August 28, 2003, Duke Energy Corporation, et al. (the licensee), submitted a request for changes to the Catawba Nuclear Station, Units 1 and 2 Technical Specifications (TS). The requested changes would revise TS 3.3.2, "Engineered Safety Features Actuation System (ESFAS) Instrumentation."

The supplements dated March 4, 2002, September 12, 2002, November 20, 2002, and August 28, 2003, provided clarifying information that did not change the scope of the December 20, 2001, application or the initial proposed no significant hazards consideration determination.

2.0 REGULATORY EVALUATION

Title 10 of *Code of Federal Regulations* (10 CFR), Section 50.36, "Technical Specifications," requires that all operating licenses for nuclear power reactors include TS for the subject plant. The limiting conditions for operation (LCO), including the required completion times, are required for each system included in the TS. The Improved Technical Specifications (ITS) have been implemented for Catawba, Units 1 and 2.

The Nuclear Regulatory Commission (NRC) staff evaluated the licensee's proposed changes using 10 CFR 50.36; 10 CFR 50.55a(h), "Codes and Standards;" the Catawba Updated Final Safety Analysis Report (UFSAR); guidance from NUREG 0800, "Standard Review Plan;" and NUREG 1431, "Standard Technical Specifications for Westinghouse Plants" (STS).

3.0 TECHNICAL EVALUATION

The licensee withdrew portions of its application dated December 20, 2001. The following table lists the affected TS page, the item number and the date of the licensee's letter withdrawing the proposed change. The Functions listed in the following discussions refer to the Functions as described in TS Table 3.3.2-1, "Engineered Safety Feature Actuation System Instrumentation."

These withdrawals are acceptable to the NRC staff since they simply withdraw some of the proposals for changes to the TS that were made in the application dated December 20, 2001.

Initially Proposed Item	Date of Withdrawal Letter
Page 3.3.2-5, modification of Condition K for the main feedwater pump Function 6.e.	November 20, 2002.
Page 3.3.2-5, delete Condition L for doghouse water level function 5.e.	August 28, 2003.
Page 3.3.2-6, modifying Condition M for auxiliary feedwater function 6.f.	November 20, 2002
Page 3.3.2-8, add Condition S for proposed Turbine Trip Reactor Trip function 5.a(4).	November 20, 2002
Page 3.3.2-8, add Condition T for loss of offsite power function 6.d.	November 20, 2002
Page 3.3.2-8, add Condition U for auxiliary feedwater pump function 6.f	November 20, 2002
Page 3.3.2-13, addition of operability exception footnote (f) to Function 5.a(1) and 5.a(2).	November 20, 2002.
Page 3.3.2-13, deletion of Function 5.e, doghouse water level - high high.	August 28, 2003.
Page 3.3.2-13, add Function 5.a(4), Reactor Trip (P-4).	November 20, 2002.
Pages 3.3.2-13 and 3.3.2-14, changes proposed to footnote (e), as applicable to proposed Functions 5.b(1), 5.b(2) and 5.b(4)	March 4, 2002
Pages 3.3.2-14, replace Condition D with a new Condition T for Function 6.d, loss of offsite power.	November 20, 2002.
Page 3.3.2-15, proposed revision to the number of required channels for Function 8.c, ESFAS interlocks.	November 20, 2002
Page 3.3.5-1, Diesel generator start instrumentation	November 20, 2002

As described in the table above all of the changes proposed for pages 3.3.2-5, 3.3.2-6, and 3.3.2-8 were withdrawn. The changes proposed for pages 3.3.2-10, 3.3.2-13, 3.3.2-14 and 3.3.2-15 are discussed below.

Pages 3.3.2-10 and 3.3.2-15, Addition of SR 3.3.2.12 for Nuclear Service Water Suction Transfer, Function 10

The Standby Nuclear Service Water Pond (SNSWP) swap logic aligns the Nuclear Service Water System (RN) to the SNSWP if two out of three service water pit level transmitters detect low pit level. NRC Generic Letter (GL) 96-01, “Testing of Safety-Related Circuits,” states that

licensees should “Compare electrical schematic drawings and logic diagrams for the reactor protection system, EDG load shedding and sequencing, and actuation logic for the engineered safety features systems against plant surveillance test procedures to ensure that all portions of the logic circuitry, including the parallel logic, interlocks, bypasses and inhibit circuits, are adequately covered in the surveillance procedures to fulfill the TS requirements.”

The licensee states that during an NRC inspection of Catawba, it was determined that the SNSWP swap logic was not tested consistent with GL 96-01 guidance, and that this logic testing should be added to TS 3.3.2. The licensee proposed adding SR 3.3.2.12 to the list of SRs on TS page 3.3.2-10, to require the performance of an Actuation Logic Test at a frequency of 18 months. The new SR is also added to TS Table 3.3.2-1 Function 10, Nuclear Service Water Suction Transfer - Low Pit Level on TS page 3.3.2-15. The NRC staff finds that this proposed change corrects an apparent omission in the current Catawba TS, since there is no current requirement to perform an actuation logic test on this instrumentation. The NRC staff finds that the specified frequency for new SR 3.3.2.12 is consistent with that for current TS 3.3.2 SRs 3.3.2.8, 3.3.2.9, 3.3.2.10, and 3.3.2.11, which address similar ESFAS function surveillances. On these bases, the NRC staff concludes that this change is acceptable.

Page 3.3.2-13, Revision of Function 5, Turbine Trip and Feedwater Isolation

This change proposes dividing Function 5, Turbine Trip and Feedwater Isolation, into two categories: Function 5.a, Turbine Trip; and Function 5.b, Feedwater Isolation. The existing Function 5 contains six functions (Functions 5.a through 5.f). The licensee proposed dividing these six functions because some of these functions apply only to the Turbine Trip Function, some of the functions apply only to the Feedwater Isolation Function, and some of the functions apply to both the Turbine Trip and the Feedwater Isolation functions. The licensee stated that, prior to converting to the ITS, the Catawba TS had separate listings for turbine trip and feedwater isolation. This proposed change reverts to the pre-ITS format and clarifies the current TS requirements that apply to these two functions (the Turbine Trip and Feedwater Isolation functions). The licensee concluded that this proposed change will make it easier to clearly implement this part of the TS during plant operations and testing. The following sections describe the proposed changes and the staff’s safety evaluation of the changes.

The existing Table 3.3.2-1, Function 5, Turbine Trip and Feedwater Isolation, lists the following six functions:

a. Automatic Actuation Logic and Actuation Relays
b. Steam Generator (SG) Water Level-High High (P-14)
c. Safety Injection
d. T_{avg}-Low coincident with Reactor Trip, P-4
e. Doghouse Water Level-High High
f. Trip of all main feedwater pumps

The licensee proposed to subdivide these functions as follows:

5.a. Turbine Trip
  (1) Automatic Actuation Logic and Actuation Relays
  (2) SG Water Level- High High (P-14)
  (3) Safety Injection

5.b. Feedwater Isolation
(1) Automatic Actuation Logic and Actuation Relays
(2) SG Water Level-High High (P-14)
(3) Safety Injection
(4) T_{avg}-Low coincident with Reactor Trip, P-4
(5) Doghouse Water Level - High High

Function 5.a.(1), Turbine Trip, Automatic Actuation Logic and Actuation Relays, and Function 5.a.(2), Turbine Trip, SG Water Level-High High (P-14)

The proposed applicable modes or other specified conditions, required channels, conditions, surveillance requirements, allowable value, and nominal trip setpoint for Function 5.a.(1), Turbine Trip, Automatic Actuation Logic and Actuation Relays, and for Function 5.a.(2), Turbine Trip, SG Water Level-High High (P-14), remain the same as in the existing TS. The NRC staff, therefore, finds the proposed renumbering of these Functions from 5.a and 5.b to Functions 5.a(1) and 5.a(2), respectively, does not change the technical basis for these functions, and therefore, finds these changes to be acceptable.

The licensee proposed deleting the reference to Footnote (e) for MODE 2 because this footnote applies only to the Feedwater Isolation functions and, therefore, is not applicable to Function 5.a. This change is appropriate and is acceptable.

Function 5.a(3), Turbine Trip, Safety Injection Applicable Modes and Function 5.b(3), Feedwater Isolation, Safety Injection Applicable Modes

The current Function 5.c in the existing TS states, “Refer to Function 1 (Safety Injection) for all initiation functions and requirements.” This means that the applicable MODES for the existing Function 5.c are, by reference to Function 1, MODES 1, 2, 3, and 4.

For the proposed new Function 5.a(3), Turbine Trip, Safety Injection, the licensee proposed adding the statement, “See Item 5.a.(1) for Applicable Modes.” This would mean that the applicable MODES for the proposed Function 5.a(3) are the same as for Function 5.a(1) and are MODE 1 and MODE 2. Since the main turbine is tripped in MODES 3 and 4 and, therefore, satisfies the requirement for a turbine trip, the staff finds the proposed applicable MODES 1 and 2 for the Turbine Trip, Safety Injection (Function 5.a(3)) to be acceptable.

For the proposed new Function 5.b(3), Feedwater Isolation, Safety Injection, the licensee proposed adding the statement, “See Item 5.b.(1) for Applicable Modes.” This would mean that the applicable MODES for the proposed Function 5.b(3) are the same as for Function 5.b(1) and are MODE 1 and MODES 2 and 3 with the exception provided by footnote (e). Since the main feedwater control valves are not placed into automatic operation until at least MODE 3, the staff finds the proposed applicable MODE 1 and MODES 2 and 3, as modified by footnote (e), to be acceptable.

The proposed required channels, conditions, surveillance requirements, allowable value, and nominal trip setpoint remain the same as in the existing TS. The staff therefore, finds this

proposed renumbering of the Turbine Trip, Safety Injection and Feedwater Isolation, Safety Injection functions to be acceptable.

Function 5.b.(1), Automatic Actuation Logic and Actuation Relays, Mode 3 Applicability, and Function 5.b.(2), SG Water Level-High High (P-14), Mode 3 Applicability

This change proposes adding MODE 3 to the applicable conditions required for the proposed Function 5.b.(2), SG Water Level - High High (P-14). The licensee states that interlock P-14 provides feedwater isolation when the level in a SG exceeds the High-High SG level setpoint.

The licensee states that prior to the ITS conversion the Catawba TS had separate TS requirements for turbine trip, feedwater isolation, and the P-14 interlock. Within those separate requirements, turbine trip and feedwater isolation on High-High SG level were required to be operable in MODES 1 and 2; however, the P-14 interlock was required to be operable in MODES 1, 2, and 3. During the Catawba ITS conversion, the P-14 entry for the ESFAS interlock function was deleted, with the justification that it was captured under the turbine trip and feedwater isolation functions. Consequently, the associated MODE 3 requirement for the P-14 interlock was inadvertently omitted.

The licensee concluded there were no operability concerns since the Catawba Operations staff currently maintains P-14 operable in MODE 3 by procedure. Nevertheless, the licensee determined that there should be a MODE 3 requirement in the TS for the P-14 interlock, since the feedwater system could be placed in service with full automatic control in MODE 3, thus leading to a potential for a SG overfill to occur in MODE 3. Since the automatic actuation logic and actuation relays addressed by Function 5.b.(1) are required in order for the P-14 interlock to be operable, the same change is also proposed for Function 5.b.(1) to be operable in MODE 3. The staff finds the licensee's reasons for proposing that the P-14 interlock be operable in the MODE 3 to be appropriate since the interlock provides protection against SG overfill and therefore, concludes that this change is acceptable.

Function 5.b(1) and 5.b(2), End State of Mode 4

This change proposes to change the Applicable Condition for Function 5.b.(1) from Condition I to Condition H. The Required Action for an inoperable train specified in Condition I has an end state of MODE 3, and the corresponding Condition H has an end state of MODE 4. Since a MODE 3 operability requirement is being added to Function 5.b.(1), the NRC staff finds that the correct end state for Function 5.b.(1) should be MODE 4 and, therefore, concludes that this change is acceptable.

This change proposes to change the Applicable Condition for Function 5.b.(2) from Condition J to Condition D. The Required Action for an inoperable channel specified in Condition J has an end state of MODE 3, and the corresponding Condition D has an end state of MODE 4. Since a MODE 3 operability requirement is being added to this function, the staff finds that the correct end state for Function 5.b.(2) should be MODE 4 and, therefore, concludes that this change is acceptable.

Relocation of Current Function 5.f, Trip of all Main Feedwater Pumps

This change proposes to relocate Function 5.f, Trip of All Main Feedwater Pumps, to the Selected Licensee Commitments (SLC) Manual. The purpose of Function 5.f is to provide a trip of the main turbine when both main feedwater pumps are tripped to limit the loss of SG water level upon a loss of normal feedwater. Function 5.f requires three channels of operable instrumentation per main feedwater pump for this function. Function 5.f refers to Condition K, that states, with one main feedwater pump trip channel inoperable, either the affected channel must be placed in trip within 1-hour or the unit must be in MODE 3 within 7 hours. Since there are no other conditions referenced for this function, in the event that more than one channel of this instrumentation becomes inoperable, TS 3.0.3 would apply.

The licensee states that the function of tripping the main turbine upon loss of both main feedwater pumps is anticipatory in nature only, and is not credited in any accident analysis. The loss of normal feedwater flow transient is discussed in Section 15.2.7 of the Catawba UFSAR. The accident analysis for this transient showed that the loss of feedwater, whether it is caused by main feedwater pump failures or other initiating events, is mitigated by the steam generator low-low level reactor trip function. If a loss of both main feedwater pumps were to occur, the accident analysis showed that a reactor trip occurs on low-low steam generator level, which in turn, causes a turbine trip to occur via the P-4 ESFAS Interlock (TS Table 3.3.2-1, Function 8a). Additionally, this relocation of Function 5.f from TS Table 3.3.2-1 is consistent with the STS, which does not contain this ESFAS function.

On the basis of these considerations, the licensee concluded that this function does not meet the 10 CFR 50.36 criteria for inclusion in plant TS since it: 1) is not used to detect reactor coolant boundary leakage, 2) is not credited in any accident analyses, 3) does not function to mitigate an accident, and 4) is not risk significant. The licensee proposes to relocate this function from TS 3.3.2-1 into the licensee's SLC Manual that is subject to the adminisrative controls and review processes required by 10 CFR 50.59.

On the basis of the above-stated justifications, the criteria in 10 CFR 50.36, and the guidance in the STS, the staff concludes that the relocation of Function 5.f from TS 3.3.2 to the SLC Manual is acceptable.

Page 3.3.2-14, Function 6.e, Trip of All Main Feedwater Pumps

For Function 6.e, the licensee proposed deletion of footnote (a) to the MODE 2 condition that states: "(a) Above the P-11 (Pressurizer Pressure) interlock." The licensee states that since the plant cannot be operated below the P-11 interlock in MODE 2, the footnote has no meaning for Function 6.e. If the plant is in MODE 2 it will be above the P-11 interlock. On this basis the NRC staff finds that the footnote has no meaning and that its deletion is acceptable.

Page 3.3.2-14, Function 6.f, Auxiliary Feedwater Pump Train A and Train B Suction Transfer on Suction Pressure - Low

An NRC safety evaluation dated April 6, 2001, added an asterisked footnote to Condition M of Function 6.f that related to a one-time change, applicable only to Unit 1. The footnote stated:

> If more than one channel of Auxiliary Feedwater Suction Pressure-Low for one train becomes inoperable, immediately enter the applicable Condition(s) and Required Action(s) for the associated AFW train made inoperable by the inoperable channels. This is a one time only change for Unit 1 in support of the activities associated with the replacement of pressure switch 1CAPS5232.

Since this footnote was related to a modification that occurred only one time and it has since been completed, the NRC staff finds the footnote to be extraneous and concludes that its deletion is acceptable.

SUMMARY

Based on the above review and justifications for TS changes, the NRC staff concludes that the licensee's proposed TS changes are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (67 FR 12601). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. E. Waterman
P. Hearn
R. Martin

Date: September 10, 2003

Catawba Nuclear Station

cc:

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