

Exelon Generation Company, LLC
LaSalle County Station
2601 North 21st Road
Marseilles, IL 61341-9757

www.exeloncorp.com

August 12, 2003

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

LaSalle County Station, Units 1 and 2
Facility Operating License Nos. NPF-11 and NPF-18
NRC Docket Nos. 50-373 and 50-374

Subject: 2002 Regulatory Commitment Change Summary Report

Enclosed is the Exelon Generation Company (EGC), LLC, commitment change summary for LaSalle County Station for 2002. Revisions to docketed correspondence were processed using the Nuclear Energy Institute's (NEI) 99-04, Revision 0, "Guidelines for Managing NRC Commitment Changes," dated July 1999.

Should you have any questions concerning this letter, please contact Mr. Glen T. Kaegi, Regulatory Assurance Manager, at (815) 415-2800.

Respectfully,



Susan R. Landahl
Plant Manager
LaSalle County Station

Attachment

cc: Regional Administrator - NRC Region III
NRC Senior Resident Inspector - LaSalle County Station

A001

Attachment
LaSalle County Station
Revised Commitment Summary for 2002

| Commitment Revision Tracking No. | Date of Commitment Revision | Original Document | Original Commitment | Revised Commitment | Basis For Revision |
|----------------------------------|-----------------------------|--------------------------------|--|--|---|
| 02-01 | 9/20/02 | IE Bulletin 96-03 | We shall perform an inspection of the drywell and wetwell each refueling outage. | We shall perform an inspection of the drywell each refueling outage. We shall perform an inspection of the wetwell periodically to coincide with ASME Section XI, Subsection IWE containment inspections. | <p>Current FME practices implemented in response to Bulletin 96-03 have been demonstrated to be effective. By aligning this inspection with the CISI Program, a more efficient utilization of divers would result in a significant reduction in personnel dose.</p> <p>Detailed inspections of the suppression chamber are performed on a periodic basis (3-4-3 years) as part of the CISI Program, which incorporates the requirements of ASME Section XI, Subsection IWE. Because these inspections are done only during refueling outages (2 year cycle), they are performed every refueling outage except two during the 10 year interval. The result of this change is that during two refueling outages every 10 years, an inspection of the suppression chamber would not be required.</p> |
| 02-02 | 8/21/02 | Inspection Report 50-373/80-07 | The responsibility for the Respirator Program shall be vested in one individual, as specified in the Regulatory Position of Reg. Guide 8.15 Section 12.1 and NUREG-0041 Section 3.1.5. | The commitment has been deleted. | Corporate procedure RP-AA-440 incorporates appropriate administrative controls, and references Reg. Guide 8.15 and NUREG-0041 requirements. |
| 02-03 | 11/5/02 | Generic Letter 88-01 | Inspections of IGSCC susceptible welds will be done in accordance with Generic Letter 88-01, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping." | Inspections of IGSCC susceptible welds will be done in accordance with BWRVIP-75 guidance for normal water chemistry, and Risk-Informed Inservice Inspections. BWRVIP-75 is "BWR Vessel and Internals Project Technical Basis for Revisions to Generic Letter 88-01 Inspection Schedules." | The BWRVIP-75 and Risk-Informed Inservice Inspections guidance allows the inspection of fewer welds, resulting in less radiation exposure. |

Attachment
LaSalle County Station
Revised Commitment Summary for 2002

| Commitment Revision Tracking No. | Date of Commitment Revision | Original Document | Original Commitment | Revised Commitment | Basis For Revision |
|----------------------------------|-----------------------------|--|--|---|--|
| 02-04 | 1/7/03 | NUREG-0519, Safety Evaluation Report Related to the Safe Operation of LaSalle County Station Units 1 and 2 | NUREG-0519 Section 3.9.6 "Inservice Testing of Pumps and Valves" states that the applicant's proposal to leak test pressure isolation valves at each periodic test interval and not each time the valve is disturbed for those systems which are rated at a lower pressure than the reactor coolant system is acceptable for the following reasons: (1) full closure of these valves is verified in the control room by direct monitoring position indicators, ... | <p>The control room position indication is to be removed from the Reactor Core Isolation Cooling (RCIC) System Injection Line Inboard Check Valves for Units 1 and 2. These valves are pressure isolation valves.</p> <p>These valves have position indication that does not always reset to indicate that the valve closed after it has been opened. Experience has shown that the cam that operates the valve limits switch will not always rotate to the closed position after the disc is seated due to problems in the design.</p> <p>The Inboard position indication is being removed because the unreliable indication can cause operational challenges and require drywell entries, causing unnecessary personnel dose.</p> | The change has been evaluated under 10CFR50.59 and has been determined to not require NRC approval. The change does not increase the probability of accidents or increase the consequences of accidents or malfunctions of equipment already analyzed, or introduce new failure modes or accidents. The position indication is not required by the plant's Technical Specifications. |