November 4, 2003

Mr. John L. Skolds, President
and Chief Nuclear Officer
Exelon Nuclear
Exelon Generation Company, LLC
200 Exelon Way, KSA 3-E
Kennett Square, PA 19348

SUBJECT: LIMERICK GENERATING STATION, UNITS 1 AND 2 - ISSUANCE OF AMENDMENT RE: REVISION TO THE REACTOR PRESSURE VESSEL MATERIAL SURVEILLANCE PROGRAM (TAC NOS. MB7003 AND MB7004)

Dear Mr. Skolds:

The Commission has issued the enclosed Amendment No. 167 to Facility Operating License No. NPF-39 and Amendment No. 130 to Facility Operating License No. NPF-85 for the Limerick Generating Station, Units 1 and 2 (LGS-1 and 2). These amendments are in response to your application dated December 20, 2002, as supplemented on May 30, 2003.

The amendments remove the current facility reactor material specimen surveillance schedule from the Technical Specifications for LGS-1 and 2. The licensee will also revise the Updated Final Safety Analysis Report for LGS-1 and 2 to reflect implementation of the Boiling Water Reactor Vessel and Internals Project reactor pressure vessel integrated surveillance program as the basis for demonstrating the compliance with the requirements of Appendix H, "Reactor Vessel Material Surveillance Program Requirements," to Title 10 of the *Code of Federal Regulations*, Part 50.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Scott P. Wall, Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-352 and 50-353

Enclosures: 1. Amendment No. 167 to License No. NPF-39
2. Amendment No. 130 to License No. NPF-85
3. Safety Evaluation

cc w/encls: See next page

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Dr. Judith Johnsrud
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Sierra Club
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The amendments remove the current facility reactor material specimen surveillance schedule from the Technical Specifications for LGS-1 and 2. The licensee will also revise the Updated Final Safety Analysis Report for LGS-1 and 2 to reflect implementation of the Boiling Water Reactor Vessel and Internals Project reactor pressure vessel integrated surveillance program as the basis for demonstrating the compliance with the requirements of Appendix H, "Reactor Vessel Material Surveillance Program Requirements," to Title 10 of the *Code of Federal Regulations*, Part 50.

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DISTRIBUTION:

PUBLIC	PDI-2 R/F	JClifford	SWall	MO'Brien (2)	OGC
SCoffin	JUhle	JHoncharik	LLois	GHill (4)	ACRS
CBixler, RGN-I					

ADAMS Accession Numbers: Package: ML033080147,
Amendment: ML032310540; TS(s): ML
* SE input provided - no major changes made. ** See previous concurrence

OFFICE	PDI-2/PM	PDI-2/LA	EMCB/SC*	OGC**	PDI-2/SC
NAME	SWall	MO'Brien	SCoffin	LZaccari	JBoska for JClifford
DATE	11/03/03	11/03/03	6/30/03	8/29/03	11/03/03

OFFICIAL RECORD COPY

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-352

LIMERICK GENERATING STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 167
License No. NPF-39

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated December 20, 2002, as supplemented by letter dated May 30, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. The license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-39 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 167, are hereby incorporated in the license. Exelon Generation Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days. The program description set out in the licensee's application dated December 20, 2002, as supplemented by letter dated May 30, 2003, and evaluated in the safety evaluation enclosed with this amendment shall be incorporated into the UFSAR. The licensee shall submit the changes authorized by this amendment with the next update of the UFSAR in accordance with 10 CFR 50.71(e).

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by JBoska for/

James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications

Date of Issuance: November 4, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 167

FACILITY OPERATING LICENSE NO. NPF-39

DOCKET NO. 50-352

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
xi	xi
3/4 4-19	3/4 4-19
3/4 4-21	3/4 4-21
B 3/4 4-5	B 3/4 4-5
B 3/4 10-2	B 3/4 10-2

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-353

LIMERICK GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 130
License No. NPF-85

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated December 20, 2002, as supplemented by letter dated May 30, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. The license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-85 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 130, are hereby incorporated in the license. Exelon Generation Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days. The program description set out in the licensee's application dated December 20, 2002, as supplemented by letter dated May 30, 2003, and evaluated in the safety evaluation enclosed with this amendment shall be incorporated into the UFSAR. The licensee shall submit the changes authorized by this amendment with the next update of the UFSAR in accordance with 10 CFR 50.71(e).

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by JBoska for/

James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications

Date of Issuance: November 4, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 130

FACILITY OPERATING LICENSE NO. NPF-85

DOCKET NO. 50-353

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
xi	xi
3/4 4-19	3/4 4-19
3/4 4-21	3/4 4-21
B 3/4 4-5	B 3/4 4-5
B 3/4 10-2	B 3/4 10-2

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 167 AND 130 TO FACILITY OPERATING

LICENSE NOS. NPF-39 AND NPF-85

EXELON GENERATION COMPANY, LLC

LIMERICK GENERATING STATION, UNITS 1 AND 2

DOCKET NOS. 50-352 AND 50-353

1.0 INTRODUCTION

By application dated December 20, 2002 (Reference 1), as supplemented on May 30, 2003 (Reference 2), Exelon Generation Company, LLC (Exelon or the licensee), requested amendments to remove the current facility reactor material specimen surveillance schedule from the Technical Specifications (TSs) for Limerick Generating Station, Units 1 and 2 (LGS-1 and 2). The licensee will also revise the Updated Final Safety Analysis Report (UFSAR) for LGS-1 and 2, to reflect implementation of the Boiling Water Reactor Vessel and Internals Project (BWRVIP) reactor pressure vessel (RPV) integrated surveillance program (ISP) as the basis for demonstrating the compliance with the requirements of Appendix H, "Reactor Vessel Material Surveillance Program Requirements," to Title 10 of the *Code of Federal Regulations* (10 CFR), Part 50. The supplement dated May 30, 2003, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on February 4, 2003 (68 FR 5669).

The BWRVIP RPV ISP was submitted for NRC staff review and approval in Topical Reports BWRVIP-78, "BWR Vessel and Internals Project, BWR Integrated Surveillance Program Plan," and BWRVIP-86, "BWR Vessel and Internals Project, BWR Integrated Surveillance Program Implementation Plan," (References 3 and 4). Additional information necessary to establish the technical basis for, and proposed implementation of, the BWRVIP ISP was provided in letters from the BWRVIP to the NRC dated December 15, 2000, and May 30, 2001, (References 5 and 6). The NRC staff approved the proposed BWRVIP ISP in a safety evaluation (SE) which was provided to the BWRVIP by letter dated February 1, 2002, (Reference 7). However, the NRC staff's SE specified that plant-specific information be provided by BWR licensees who wish to implement the BWRVIP ISP for their facilities. The licensee's December 20, 2002, and May 30, 2003, submittals addressed the plant-specific information specified in the NRC staff's February 1, 2002, BWRVIP ISP SE.

2.0 REGULATORY REQUIREMENTS

The NRC staff determined that Exelon, in its December 20, 2002, and May 30, 2003, submittals, identified the applicable regulatory requirements. The regulatory requirements for which the NRC staff based its acceptance are described below.

Pursuant to 10 CFR 50.36, TSs are required to include items in the following five specific categories related to station operation: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements (SRs); (4) design features; and (5) administrative controls. Section 50.36(c)(3) states "Surveillance requirements are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met." As a result, SRs that do not satisfy the criteria of 10 CFR 50.36(c)(3) may be deleted from the TSs or relocated to other licensee-controlled documents.

Appendix H to 10 CFR Part 50 requires nuclear power plant licensees to implement RPV surveillance programs to "monitor changes in the fracture toughness properties of ferritic materials in the reactor vessel beltline region...which result from exposure of these materials to neutron irradiation and the thermal environment." Two specific alternatives are provided with regard to the design of a facility's RPV surveillance program which may be used to address the requirements of Appendix H to 10 CFR Part 50.

The first alternative is the implementation of a plant-specific RPV surveillance program consistent with the requirements of American Society for Testing and Materials (ASTM) Standard Practice E 185, "Standard Practice for Conduction Surveillance Tests for Light-Water Cooled Nuclear Power Reactor Vessels." In the design of a plant-specific RPV surveillance program, a licensee may use the edition of ASTM Standard Practice E 185 which was current on the issue date of the American Society of Mechanical Engineers Code to which the reactor vessel was purchased, or later editions through the 1982 Edition.

The second alternative provided in Appendix H to 10 CFR Part 50 is the implementation of an ISP. An ISP is defined in Appendix H to 10 CFR Part 50 as occurring when, "the representative materials chosen for surveillance for a reactor are irradiated in one or more other reactors that have similar design and operating features." Five specific criteria are stated in Appendix H to 10 CFR Part 50 which must be met to support approval of an ISP:

a. The reactor in which the materials will be irradiated and the reactor for which the materials are being irradiated must have sufficiently similar design and operating features to permit accurate comparisons of the predicted amount of radiation damage.

b. Each reactor must have an adequate dosimetry program.

c. There must be adequate arrangement for data sharing between plants.

d. There must be a contingency plan to assure that the surveillance program for each reactor will not be jeopardized by operation at reduced power level or by an extended outage of another reactor from which data are expected.

e. There must be substantial advantages to be gained, such as reduced power outages or reduced personnel exposure to radiation, as a direct result of not requiring surveillance capsules in all reactors in the set.

As noted above in Section 1.0, the NRC staff approved the proposed BWRVIP ISP in an SE issued to the BWRVIP by letter dated February 1, 2002, (Reference 7). In Reference 7, all of the criteria cited above for approval of an ISP were addressed either completely or partially. For those criteria which could not be fully addressed in Reference 7, plant-specific information was required. The NRC staff identified in Reference 7 the specific information which would be needed from licensees who wished to implement the BWRVIP for their facilities. As stated in Reference 7:

> "[L]icensees who wish to participate in the BWR ISP must provide, for NRC staff review and approval, information which defines how they will determine RPV and/or surveillance capsule fluences based on the dosimetry data which will be available for their facilities. This information must be submitted concurrently with each licensee's submittal to replace their existing plant-specific surveillance program with the BWR ISP as part of their facility's licensing basis. The information submitted must be sufficient for the staff to determine that:
>
> (1) RPV and surveillance capsule fluences will be established as based on the use of an NRC-approved fluence methodology that will provide acceptable results based on the available dosimetry data; and
>
> (2) If one methodology is used to determine the neutron fluence values for a licensee's RPV and one or more different methodologies are used to establish the neutron fluence values for the ISP surveillance capsules which "represent" that RPV in the ISP, the results of these differing methodologies are compatible (i.e., within acceptable levels of uncertainty for each calculation)."

This plant-specific information was specified by the NRC staff to ensure that Criterion III.C.1.b of Appendix H to 10 CFR Part 50 for an ISP could be met by each facility and to confirm that data which would be shared as part of the BWRVIP ISP could be effectively utilized by each licensee for the monitoring of RPV embrittlement for their facility.

Regulatory Guide (RG) 1.190, "Calculational and Dosimetry Methods For Determining Pressure Vessel Neutron Fluence," describes methods and assumptions acceptable to the NRC staff for determining the pressure vessel neutron fluence. The guide is intended to ensure the accuracy and reliability of the fluence determination required by General Design Criteria 14, 30, and 31 of Appendix A, "General Design Criteria for Nuclear Power Plants," to 10 CFR Part 50.

3.0 TECHNICAL EVALUATION

In its application dated December 20, 2002, as supplemented on May 30, 2003, Exelon submitted information for LGS-1 and 2 which addressed the plant-specific information requested in the NRC staff's SE approving the BWRVIP ISP (Reference 7). Exelon proposed to delete SRs 4.4.6.1.3, 4.4.6.1.4, and Table 4.4.6.1.3-1 from LGS-1 and 2 TSs, and revise Sections 4.1.4.5 and 4.3.2.8 of the LGS-1 and 2 UFSAR.

SRs 4.4.6.1.3 and 4.4.6.1.4 currently require that RPV material surveillance specimens and the reactor flux wire specimens located within the surveillance capsules be removed and examined in accordance with the schedule in Table 4.4.6.1.3-1. The results of these examinations are used to update the pressure/temperature (P/T) curves of Figure 3.4.6.1-1. The limits defined by Figure 3.4.6.1-1 provide an acceptable range of operating temperatures and pressures for heatup, cooldown, criticality, and inservice leak and hydrostatic testing. A program for RPV material surveillance ensures that data is available to update the operating P/T limits.

Section II.B.3 of Appendix H to 10 CFR Part 50 requires the submittal to, and approval by, the NRC of a proposed withdrawal schedule for material specimens before implementation. The control of changes to this schedule by way of a license amendment to the TSs duplicates the requirements of Appendix H to 10 CFR Part 50. In Generic Letter 91-01, "Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimens from Technical Specification," the NRC staff concluded that this duplication is unnecessary. The P/T curves provide the limits necessary to assure that facility operation will be within safety limits. The schedule for the withdrawal of RPV material surveillance specimens are not, therefore, required to assure that facility operations will be within safety limits because changes to this schedule are controlled by the requirements of Appendix H to 10 CFR Part 50. The NRC staff finds the deletion of SRs 4.4.6.1.3, 4.4.6.1.4, and Table 4.4.6.1.3-1, as proposed in References 1 and 2, from the LGS-1 and 2 TSs, is acceptable since SRs 4.4.6.1.3, 4.4.6.1.4, and Table 4.4.6.1.3-1 do not satisfy the criteria of 10 CFR 50.36(c)(3). The licensee made associated changes to the TS Bases. The NRC staff has no objection to the TS Bases changes.

The proposed revisions to Sections 4.1.4.5 and 4.3.2.8 of the LGS-1 and 2 UFSAR stated in part:

> "LGS RPV fluence has been evaluated using a method in accordance with the recommendations of RG 1.190. Future evaluations of RPV fluence will be completed using a method in accordance with the recommendations of RG 1.190."

RG 1.190 was developed to provide state-of-the-art calculations and measurement procedures that are acceptable to the NRC staff for determining pressure vessel fluence. Although the NRC staff would note that the guidance in RG 1.190 is not a regulatory requirement of the NRC, the staff has concluded that the inclusion of this statement in the LGS-1 and 2 UFSAR is sufficient to address both items (1) and (2) from Reference 7. Regarding item (1), the licensee's use of a methodology for determining LGS-1 and 2 RPV neutron fluence values which is consistent with the attributes of RG 1.190 and has been approved by the NRC staff will provide acceptable results based upon the available dosimetry data. Regarding item (2), RPV surveillance capsules tested under the BWRVIP ISP will have their fluences determined by the use of a methodology which is consistent with the attributes of RG 1.190 and has been

approved by the NRC staff. The NRC staff has concluded that any two (or more) different fluence methodologies will provide "compatible" (as defined in Reference 7) results provided that the best estimate fluence values are within each other's uncertainty bounds.

The licensee proposed a revision to Section 5.3.1.6.1 of the LGS-1 and 2 UFSAR which documented the licensee's incorporation of the BWRVIP ISP into the LGS-1 and 2 licensing basis. Relevant excerpts from the LGS-1 and 2 UFSAR section are provided below:

> "In 2003, the NRC approved LGS participation in the BWR Vessel and Internals Project (BWRVIP) Integrated Surveillance Program (ISP) as described in BWRVIP-78 and BWRVIP-86 (Reference 5.3-11) [BWRVIP-86-A: "BWR Vessel and Internals Project, Updated BWR Integrated Surveillance Program (ISP)," Final Report, October 2002]. The NRC approved the ISP for the industry in Reference 5.3-11 and approved LGS participation...The ISP meets the requirements of 10 CFR 50 Appendix H and provides several advantages over the original program...The current withdrawal schedule for both units is based on the latest NRC-approved revision of BWRVIP-86 (Reference 5.3-11). Based on this schedule, LGS is not scheduled to withdraw any additional material specimens."

The NRC staff has concluded that the information provided in the proposed revision to the LGS-1 and 2 UFSAR is adequate to document the licensee's intent to appropriately implement the BWRVIP ISP as the method for demonstrating the compliance of LGS-1 and 2 with the requirements of Appendix H to 10 CFR Part 50.

Based on the above, the NRC staff has concluded that the information provided by the licensee was sufficient to conclude that the BWRVIP ISP, as approved in Reference 7, can be implemented for LGS-1 and 2 as the basis for demonstrating continued compliance with the requirements of Appendix H to 10 CFR Part 50. As part of the implementation and documentation of the licensee's intent to utilize the BWRVIP ISP for this purpose, the licensee shall modify the LGS-1 and 2 UFSAR as stated in its December 20, 2002, and May 30, 2003, submittals. This document is controlled in accordance with the requirements of 10 CFR 50.59, "Changes, tests, and experiments."

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (68 FR 5669). Accordingly, the amendments meet the eligibility

criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

7.0 REFERENCES

1. M. P. Gallagher, Exelon, to U.S. NRC Document Control Desk, "Request for License Amendment Regarding Reactor Vessel Specimen Removal Schedule," December 20, 2002.

2. K. R. Jury, Exelon, to U.S. NRC Document Control Desk, "Additional Information Supporting the Request for License Amendment Regarding Reactor Vessel Specimen Removal Schedule," May 30, 2003.

3. C. Terry, BWRVIP, to U.S. NRC Document Control Desk, "Project No. 704 - BWR Vessel and Internals Project, BWR Integrated Surveillance Program Plan (BWRVIP-78)," December 22, 1999.

4. C. Terry, BWRVIP, to U.S. NRC Document Control Desk, "Project No. 704 - BWRVIP-86: BWR Vessel and Internals Project, BWR Integrated Surveillance Program Implementation Plan," [Electric Power Research Institute] EPRI Technical Report 1000888, December 22, 2000.

5. C. Terry, BWRVIP, to U.S. NRC Document Control Desk, "Project No. 704 - BWRVIP Response to NRC Request for Additional Information Regarding BWRVIP-78," December 15, 2000.

6. C. Terry, BWRVIP, to U.S. NRC Document Control Desk, "Project No. 704 - BWRVIP Response to Second NRC Request for Additional Information on the BWR Integrated Surveillance Program," May 30, 2001.

7. W. H. Bateman, NRC, to C. Terry, BWRVIP, "Safety Evaluation Regarding EPRI Proprietary Reports BWR Vessel and Internals Project, BWR Integrated Surveillance Program Plan (BWRVIP-78)" and "BWRVIP-86: BWR Vessel and Internals Project, BWR Integrated Surveillance Program Implementation Plan," February 1, 2002.

Principal Contributors: J. Honcharik
L. Lois
S. Wall

Date: November 4, 2003