

August 22, 2003

MEMORANDUM TO: Laura Dudes, Section Chief  
New Reactors Section  
New, Research and Test Reactors Program  
Division of Regulatory Improvement Programs, NRR

FROM: Joseph Colaccino, Senior Project Manager */RA/*  
New Reactors Section  
New, Research and Test Reactors Program  
Division of Regulatory Improvement Programs, NRR

SUBJECT: JUNE 26, 2003, AP1000 TELEPHONE CONFERENCE CALL SUMMARY

On Thursday, June 26, 2003, a telephone conference call was held with Westinghouse Electric Company (Westinghouse) representatives and Nuclear Regulatory Commission (NRC) staff to discuss AP1000 draft safety evaluation report open items (DSER OIs) 2.3.4-1, 14.3.4-1, 15.3-1, 15.3-2, and 15.3.6-1. The call participants are listed in Attachment 1. A summary of the conference call is included below.

OI 2.3.4-1:

This OI is related to NRC Regulatory Guide (RG) 1.194, "Atmospheric Relative Concentrations for Control Room Radiological Habitability Assessments at Nuclear Power Plants," which was issued in June 2003 (ADAMS Accession Number ML031530505). Westinghouse stated that the calculations related to this OI were performed in 1997 for the AP600. The NRC staff stated that Westinghouse should compare the inputs and assumptions used in its calculations of the X/Q values used in the control room dose analysis with those in RG 1.194. Westinghouse agreed to address this OI by reviewing their calculations and addressing and justifying the differences between their calculation methodology and the methodology discussed in RG 1.194.

Westinghouse and the NRC staff also discussed Table 1 in the Westinghouse revision 1 response to NRC request for additional information number 451.006.

OI 14.3.4-1:

Westinghouse stated that the control room values were not a Tier 1 site parameter in any other advanced light water reactor certification and there was no need to deviate for the AP1000. The NRC staff stated that offsite  $\chi/Q$ s are Tier 1. The NRC staff plans to evaluate this issue upon resolution of OI 2.3.4-1.

OI 15.3-1:

The NRC staff had not completed their independent evaluation of the use of the AP600 aerosol removal coefficients at the time of the phone call.

OI 15.3-2:

The NRC staff pointed out that this OI, which involves an evaluation of the hypothetical reference control room atmospheric dispersion factors, is dependent on the successful resolution of OIs 2.3.4-1 and 6.4-1. The NRC staff noted that preliminary analyses are acceptable. However, control room dose results for the steam generator tube rupture and main steam line break accidents may be impacted if the control room  $\chi/Qs$  were to increase.

OI 15.3.6-1:

The NRC staff had not completed their independent evaluation of the loss-of-coolant accident radiological consequences analysis at the time of the phone call.

Docket No. 52-006

Attachment: As stated

cc w/atts: See next page

L. Dudes

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OI 15.3-2:

The NRC staff pointed out that this OI, which involves an evaluation of the hypothetical reference control room atmospheric dispersion factors, is dependent on the successful resolution of OIs 2.3.4-1 and 6.4-1. The NRC staff noted that preliminary analyses are acceptable. However, control room dose results for the steam generator tube rupture and main steam line break accidents may be impacted if the control room  $\chi$ /Qs were to increase.

OI 15.3.6-1:

The NRC staff had not completed their independent evaluation of the loss-of-coolant accident radiological consequences analysis at the time of the phone call.

Docket No. 52-006

Attachment: As stated

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NAME	JColaccino	RDennig	LDudes-JMS3 for:
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JUNE 26, 2003  
TELEPHONE CONFERENCE CALLS SUMMARY  
LIST OF PARTICIPANTS

Nuclear Regulatory Commission

M. Hart  
L. Brown  
J. Colaccino

Westinghouse

M. Corletti  
D. Huchings  
T. Meneely

AP 1000

cc:

Mr. W. Edward Cummins  
AP600 and AP1000 Projects  
Westinghouse Electric Company  
P.O. Box 355  
Pittsburgh, PA 15230-0355

Mr. H. A. Sepp  
Westinghouse Electric Company  
P.O. Box 355  
Pittsburgh, PA 15230

Lynn Connor  
Doc-Search Associates  
2211 SW 1<sup>ST</sup> Ave - #1502  
Portland, OR 97201

Barton Z. Cowan, Esq.  
Eckert Seamans Cherin & Mellott, LLC  
600 Grant Street 44<sup>th</sup> Floor  
Pittsburgh, PA 15219

Mr. Ed Rodwell, Manager  
Advanced Nuclear Plants' Systems  
Electric Power Research Institute  
3412 Hillview Avenue  
Palo Alto, CA 94304-1395

Charles Brinkman, Director  
Washington Operations  
Westinghouse Electric Company  
12300 Twinbrook Parkway, Suite 330  
Rockville, MD 20852

Mr. R. Simard  
Nuclear Energy Institute  
1776 I Street NW  
Suite 400  
Washington, DC 20006

Mr. Thomas P. Miller  
U.S. Department of Energy  
Headquarters - Germantown  
19901 Germantown Road  
Germantown, MD 20874-1290

Mr. David Lochbaum  
Nuclear Safety Engineer  
Union of Concerned Scientists  
1707 H Street NW, Suite 600  
Washington, DC 20006-3919

Mr. Paul Gunter  
Nuclear Information & Resource Service  
1424 16th Street, NW., Suite 404  
Washington, DC 20036

Mr. Tom Clements  
6703 Guide Avenue  
Takoma Park, MD 20912

Mr. James Riccio  
Greenpeace  
702 H Street, NW, Suite 300  
Washington, DC 20001

Mr. James F. Mallay, Director  
Regulatory Affairs  
FRAMATOME, ANP  
3315 Old Forest Road  
Lynchburg, VA 24501

Mr. Ed Wallace, General Manager  
Projects  
PBMR Pty LTD  
PO Box 9396  
Centurion 0046  
Republic of South Africa

Mr. Vince Langman  
Licensing Manager  
Atomic Energy of Canada Limited  
2251 Speakman Drive  
Mississauga, Ontario  
Canada L5K 1B2

Mr. Gary Wright, Manager  
Office of Nuclear Facility Safety  
Illinois Department of Nuclear Safety  
1035 Outer Park Drive  
Springfield, IL 62704

Dr. Gail H. Marcus  
U.S. Department of Energy  
Room 5A-143  
1000 Independence Ave., SW  
Washington, DC 20585

Mr. Steven Dolly  
Nuclear Control Institute  
1000 Connecticut Avenue, NW  
Suite 410  
Washington, DC 20036

Mr. Jack W. Roe  
SCIENTECH, INC.  
910 Clopper Road  
Gaithersburg, MD 20878

Patricia Campbell  
Winston & Strawn  
1400 L Street, NW  
Washington, DC 20005

Mr. David Ritter  
Research Associate on Nuclear Energy  
Public Citizens Critical Mass Energy  
and Environmental Program  
215 Pennsylvania Avenue, SE  
Washington, DC 20003

Mr. Michael M. Corletti  
Passive Plant Projects & Development  
AP600 & AP1000 Projects  
Westinghouse Electric Company  
P. O. Box 355  
Pittsburgh, PA 15230-0355