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Michael R. Kansler
President

June 24, 2003
NL-03-111

U.S. Nuclear Regulatory Commission
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Mail Stop O-P1-17
Washington, DC 20555-0001

SUBJECT: Indian Point Nuclear Generating Unit No. 2
Docket No. 50-247
Indian Point Nuclear Generating Unit No. 3
Docket No. 50-286
**Annual 10 CFR 50.46 Report for Year 2002-
Emergency Core Cooling System Evaluation Changes**

REFERENCES: (1) Westinghouse letter (IPP-03-26), S. Ira to J. DeRoy, "10 CFR 50.46 Annual Notification and Reporting for 2002", dated March 10, 2003.
(2) Entergy letter (NL-02-057) F. Dacimo to NRC "Annual 10 CFR 50.46 Report for Indian Point Unit 2", dated April 24, 2002.
(3) Westinghouse letter (INT-03-8), S. Ira to J. DeRoy, "10 CFR 50.46 Annual Notification and Reporting for 2002 ", dated March 10, 2003.
(4) Entergy letter (IPN-02-051) M. R. Kansler to NRC "Annual 10 CFR 50.46 Report for 2002- Emergency Core Cooling System Evaluation Changes", dated June 26, 2002.

Dear Sir:

Entergy Nuclear Operations, Inc. has reviewed the 2002 small and large break loss of coolant accident (LOCA) peak clad temperature (PCT) changes described in References (1) and (3) for Indian Point Unit 2 and Indian Point Unit 3, respectively. There are no additional changes to the small or large break PCTs since the submittal of References (2) and (4) for Indian Point Unit 2 and Indian Point Unit 3, respectively. Therefore, the Indian Point Unit 2 and Indian Point Unit 3 PCTs remain below the maximum fuel cladding temperature of 2200°F and Indian Point Unit 2 and Indian Point Unit 3 continue to comply with 10 CFR 50.46. This letter satisfies the reporting requirements of 10 CFR 50.46(a)(3)(ii).

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There are no new commitments made by this letter. If you have any questions, please contact Ms. Charlene Faison at 914-272-3378.

Very truly yours,


Michael R. Kansler
President
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cc: Regional Administrator
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