

**JUN 16 2003**

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U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555-001

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**2002 SUMMARY OF REVISED REGULATORY COMMITMENTS  
SALEM GENERATING STATION  
SALEM UNIT NOS. 1 AND 2  
DOCKET NOS. 50-272 AND 50-311**

Gentlemen:

This correspondence is being transmitted in accordance with the Nuclear Energy Institute (NEI) process for managing Nuclear Regulatory Commission (NRC) commitments. It provides a summary of changes to NRC commitments that have been made but not reported by other means. The following information is provided regarding each of the changed commitments and their source documents.

**1. NRC IE Bulletin 79-13, *Cracking in Feedwater System Piping*, and NRC Information Notice 93-20, *Thermal Fatigue Cracking of Feedwater Piping to Steam Generators*.**

The above NRC Bulletin and Information Notice identified instances of feedwater pipe cracking in a number of nuclear utilities and required that the inspection program described in the bulletin be completed for all pressurized water reactor facilities.

PSEG Nuclear (PSEG) responses to the bulletin resulted in commitments of the Salem Unit 1 and 2 Inservice Inspection (ISI) Programs (Long Term Plan) to perform required volumetric exams of Steam Generator Feedwater (SGF) piping nozzles and transition pieces at a frequency greater than required by ASME Section XI. This was due to industry concerns pertaining to thermal fatigue cracking assisted by corrosion and loading stresses.

Subsequently, Salem Units 1 and 2 SGF nozzle transition pieces have been redesigned with material that is less susceptible to thermal fatigue. The design modification to the steam generator nozzle transition pieces replaced the existing SGF reducers and

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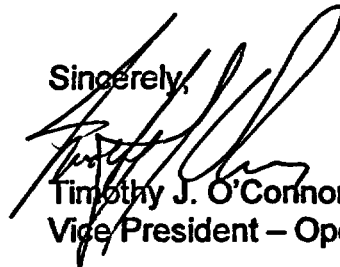
adjacent elbows with a forged single piece tuning fork fabricated from SA508, Grade 2, Class 1 low alloy steel material.

The above changes provide protection against erosion/corrosion thermal sleeve degradation and reduce the susceptibility of the welds to cracking while, maintaining the ability to safely operate the steam generators and feedwater system. These new transition piece forgings eliminate the need for increased examination frequencies.

Current and future examinations will be conducted in accordance with the requirements of ASME Section XI, as required by Technical Specifications 4.0.5 and 10CFR50.55a.

Should there be any questions please contact Howard Berrick at 856-339-1862.

Sincerely,



Timothy J. O'Connor  
Vice President - Operations

/hgb

C Mr. H. J. Miller  
Regional Administrator USNRC, Region 1  
475 Allendale Road  
King of Prussia, PA 19046

U. S. Nuclear Regulatory Commission  
ATTN: Mr. Robert Fretz,  
Licensing Project Manager - Salem  
Mail Stop 08B2  
Washington DC 20555-001

USNRC Senior Resident Inspector - Salem (X24)

Mr. K. Tosch, Manager, IV  
Bureau of Nuclear Engineering  
PO Box 415  
Trenton, NJ 08625