

June 25, 2003

ORGANIZATION: ATOMIC ENERGY OF CANADA LIMITED (AECL)

SUBJECT: SUMMARY OF MEETING ON MAY 27, 2003, WITH CANADIAN
NUCLEAR SAFETY COMMISSION AND ATOMIC ENERGY OF
CANADA LIMITED TO DISCUSS ACR-700 LICENSING APPROACH

The Nuclear Regulatory Commission (NRC) hosted a public meeting with the Canadian Nuclear Safety Commission (CNSC) and Atomic Energy of Canada Limited (AECL) on May 27, 2003, at the NRC headquarters to present AECL's planned licensing approach of the Advanced CANDU Reactor (ACR) in the United States (U.S.) and Canada. By letter dated June 19, 2002, AECL requested that the NRC begin a pre-application review of the ACR design. Additionally, AECL has requested that a similar review be conducted by the CNSC. The AECL presentation was requested by both regulatory agencies in order to facilitate the coordination between CNSC and NRC on the ACR design reviews. A list of meeting attendees is provided as Enclosure 1.

AECL provided an overview of the ACR deployment plan and overall licensing approach in North America. The U.S. licensing requirement is more prescriptive for traditional light-water reactors but the Canadian licensing approach is non-prescriptive with emphasis on safety goals that require more restrictive acceptance criteria for more frequent accident events and encouraging innovations in design to meet the operational reliability targets.

AECL stated that the objective of the CNSC licensing review is to obtain a positive statement on the licensibility of the ACR in Canada while its objective of NRC review is to obtain a standard design certification (SDC) for the ACR. Although the objectives and schedules for the reviews are somewhat different, AECL expects that the parallel licensing reviews of the ACR would provide an excellent opportunity for regulatory cooperation between CNSC and NRC.

The staff raised a concern that the integration of parallel technical reviews may not be feasible due to different licensing review objectives; licensibility from CNSC vs. SDC from NRC. The staff further explained that NRC's certification review is a plant-specific process in accordance with 10 CFR Part 50, which is far different than obtaining the licensibility in Canada. AECL responded that there is no design certification requirement by the CNSC in Canada but the common technical information has been submitted to both CNSC and NRC to promote an integrated review. AECL also stated that the safety evaluation report (SER) and probabilistic risk assessment (PRA) will be adapted to U.S. format and content for design certification application submission.

The next ACR pre-application review activity is scheduled for June 4-5, 2003, at the Whiteshell Laboratories in Winnipeg, Manitoba. The NRC staff plans on attending the technical presentations on description, scaling criteria, and experiment results of the RD-14M thermal hydraulics facility followed by a tour of the on-site test facilities, including RD-14M.

For additional details on the material covered in these presentations please refer to the Agencywide Documents Access and Management System (ADAMS). This system provides text and image files of NRC's public documents. The presentations mentioned above may be accessed through the ADAMS system under Accession No. ML031500303. If you do not have access to ADAMS or if there are problems in accessing the handouts located in ADAMS, contact the NRC Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by e-mail to pdr@nrc.gov.

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Office of Nuclear Reactor Regulation

Project No. 722

Enclosures: As stated

cc w/encls: See next page

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ADAMS ACCESSION NUMBER: ML031570330-Pkg.

ML031570340-Meeting Summary

ML031500303-Meeting Handouts

OFFICE	PM:NRLPO	PM:NRLPO	(A)DD:NRLPO
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Distribution for Meeting Summary dated June 25, 2003

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ACR-700 Licensing Reviews with CNSC & USNRC
May 27, 2003 T3B45 9:00am - Noon

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ACR-700 Licensing Reviews with CNSC & USNRC
May 27, 2003 T3B45 9:00am - Noon

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ACR-700

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