



Tennessee Valley Authority, Post Office Box 2000, Soddy-Daisy, Tennessee 37384-2000

May 19, 2003

10 CFR 50.90

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Gentlemen:

In the Matter of) Docket No. 50-328
Tennessee Valley Authority)

SEQUOYAH NUCLEAR PLANT (SQN) - REVISED INFORMATION REGARDING
TECHNICAL SPECIFICATION (TS) CHANGE 02-07, "ONE-TIME
FREQUENCY EXTENSION FOR TYPE A TEST (CONTAINMENT INTEGRATED
LEAK RATE TEST [CILRT])," TAC NOS. MB6987 AND MB6988

- References:
1. TVA letter to NRC dated October 4, 2002, "Sequoyah Nuclear Plant (SQN) - Units 1 and 2 Technical Specification (TS) Change No. 02-07, "One-Time Frequency Extension for Type A Test (Containment Integrated Leak Rate Test [CILRT])"
 2. TVA letter to NRC dated February 19, 2003, "Response to Request for Additional Information (RAI) Regarding Technical Specification (TS) Change No. 02-07, 'One-Time Frequency Extension for Type A Test (Containment Integrated Leak Rate Test [CILRT])' (TAC Nos. MB6987 and MB6988)"
 3. TVA letter to NRC dated April 14, 2003, "Sequoyah Nuclear Plant (SQN) - Supplemental Information Regarding Technical Specification (TS) Change No. 02-07, "One-Time Frequency Extension for Type A Test (Containment Integrated Leak Rate Test [CILRT])," TAC Nos. MB6987 and MB6988

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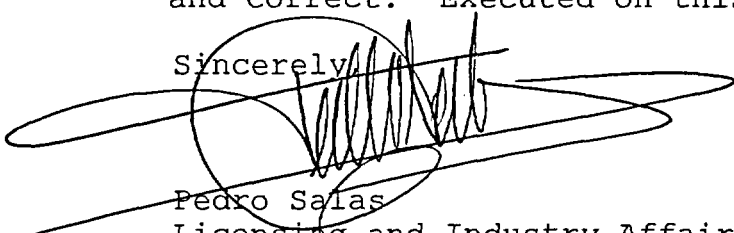
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This letter provides a revised proposed TS page (enclosed) for Unit 2. This change, as NRC suggested, modifies a previously proposed revision (TVA's referenced letters) to the TS language for Unit 2 regarding the frequency extension for performing a Type A test. NRC explained that the revision will eliminate possible ambiguity as a result of a one-cycle extension for Unit 2 that NRC approved on May 7, 2002. This revision is editorial in nature and does not change the "No Significant Hazard Determination" contained in the referenced letters.

This letter is being sent in accordance with NRC RIS 2001-05. There are no commitments contained in this submittal. Please direct questions concerning this issue to me at (423) 843-7170 or J. D. Smith at (423) 843-6672.

I declare under penalty of perjury that the foregoing is true and correct. Executed on this 19 day of May, 2003

Sincerely,



Pedro Salas

Licensing and Industry Affairs Manager

Enclosure

cc (Enclosure):

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ENCLOSURE

REVISED INFORMATION
FOR TECHNICAL SPECIFICATION (TS)
CHANGE NO. 02-07, ONE-TIME FREQUENCY EXTENSION
FOR TYPE A TEST CONTAINMENT INTEGRATED LEAK RATE TEST

ADMINISTRATIVE CONTROLS

6.8.4 f. Radioactive Effluent Controls Program (Cont.)

of radioactivity when the projected doses in a 31-day period would exceed 2 percent of the guidelines for the annual dose or dose commitment conforming to Appendix I to 10 CFR Part 50,

- 7) Limitations on the dose rate resulting from radioactive material released in gaseous effluents from the site to areas at or beyond the SITE BOUNDARY SHALL BE LIMITED to the following:
 1. For noble gases: Less than or equal to a dose rate of 500 mrem/yr to the total body and less than or equal to a dose rate of 3000 mrem/yr to the skin, and
 2. For Iodine-131, Iodine-133, tritium, and for all radionuclides in particulate form with half-lives greater than 8 days: Less than or equal to a dose rate of 1500 mrem/year to any organ.
- 8) Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50,
- 9) Limitations on the annual and quarterly doses to a MEMBER OF THE PUBLIC from Iodine-131, Iodine-133, tritium, and all radio-nuclides in particulate form with half-lives greater than 8 days in gaseous effluents released from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50, and
- 10) Limitations on the annual dose or dose commitment to any MEMBER OF THE PUBLIC due to releases of radioactivity and to radiation from uranium fuel cycle sources conforming to 40 CFR Part 190.

g. Radiological Environmental Monitoring Program (DELETED)

h. Containment Leakage Rate Testing Program

fall 2003

A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50 Appendix J, Option B, as modified by approved exemptions. Visual examination and testing, including test intervals and extensions, shall be in accordance with Regulatory Guide (RG) 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995 with exceptions provided in the site implementing instructions. Performance of the ~~spring 2002~~ containment integrated leakage rate (Type A) test may be deferred ~~up to one cycle but no later than fall 2003.~~

4 years

spring 2007

The peak calculated containment internal pressure for the design basis loss of coolant accident, P_a , is 12.0 psig.

The maximum allowable containment leakage rate, L_a , at P_a , is 0.25% of the primary containment air weight per day.

Leakage rate acceptance criteria are:

- a. Containment overall leakage rate acceptance criteria is $\leq 1.0 L_a$. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are $\leq 0.60 L_a$ for the combined Type B and Type C tests, and $\leq 0.75 L_a$ for Type A tests;