

BROWNS FERRY UNIT 1
RESPONSE TO GENERIC COMMUNICATIONS ISSUED SINCE 1985

File Name	Date	Description	Response
Bulletins			
BL-02-02	08/09/2002	Reactor Pressure Vessel Head and Vessel Head Penetration Nozzle Inspection Programs	To holders of operating licenses for boiling-water reactors for information.
BL-02-01	03/18/2002	Reactor Pressure Vessel Head Degradation and Reactor Coolant Pressure Boundary Integrity	To holders of operating licenses for boiling-water reactors for information.
BL-01-01	08/03/2001	Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles	To holders of operating licenses for pressurized water nuclear power reactors
BL-97-02	09/23/1997	Puncture Testing of Shipping Packages under 10 CFR Part 71	For Action - All holders of Nuclear Regulatory Commission Certificates of Compliance for shipping packages.
BL-97-01	04/30/1997	Potential for Erroneous Calibration, Dose Rate, or Radiation Exposure Measurements With Certain Victoreen Model 530 and 530SI Electrometer/Dose-Meters	For Action - Nonreactor licensees that possess one of the Victoreen Electrometer/Dosimeters specified in this bulletin.
BL-96-04	07/05/1996	Chemical, Galvanic, or Other Reactions in Spent Fuel Storage and Transportation Casks	Action is only requested from those addressees who are licensees with independent spent fuel storage installations, vendors of spent fuel storage or transportation casks, and holders of certificates of compliance for spent fuel storage or transportation casks (action addressees).
BL-96-03	05/06/1996	Potential Plugging of Emergency Core Cooling Suction Strainers by Debris in Boiling-Water Reactors	
BL-96-02	04/11/1996	Movement of Heavy Loads over Spent Fuel, over Fuel in the Reactor Core, or over Safety-Related Equipment	TVA letter dated May 13, 1996 - No commitments made.
BL-96-01	03/08/1996	Control Rod Insertion Problems	Sent to all holders of pressurized-water reactor (PWR) operating licenses.
BL-95-02	10/17/1995	Unexpected Clogging of a Residual Heat Removal	
BL-95-01	01/13/1995	Quality Assurance Program for Transportation of Radioactive Material	For Action - All radiography licensees.
BL-94-02	11/14/1994	Corrosion Problems in Certain Stainless Steel Packagings Used to Transport Uranium Hexafluoride	Registered users of Model Nos. NCI-21PF-1 and GE-21PF-1 uranium hexafluoride transportation packages.
BL-94-01	04/14/1994	Potential Fuel Pool Draindown Caused by Inadequate Maintenance Practices at Dresden Unit 1	For Action: All holders of licenses for nuclear power reactors that are permanently shut down with spent fuel in the spent fuel pool (except Shoreham).
BL-93-02s1	02/18/1994	Debris Plugging of Emergency Core Cooling Suction Strainers	
BL-93-03	05/28/1993	Resolution of Issues Related to Reactor Vessel Water Level Instrumentation in BWRs	
BL-93-02	05/11/1993	Debris Plugging of Emergency Core Cooling Suction Strainers	
BL-93-01	04/20/1993	Release of Patients After Brachytherapy Treatment With Remote Afterloading Devices	Addressees - Brachytherapy Licensees Authorized to Use Afterloading Brachytherapy Unit(s) Capable of Delivering Dose Rates Greater than 500 RADS (centigray) per Hour at 1 Centimeter.
BL-92-03	12/08/1992	Release of Patients After Brachytherapy	For Action - Brachytherapy Licensees Authorized to use the Omnitron Model 2000 High Dose Rate (HDR) Afterloading Brachytherapy Unit.
BL-92-01s1	08/28/1992	Failure of Thermo-Lag 330 Fire Barrier System to Perform Its Specified Fire Endurance Function	TVA's April 14, 1993 letter commits to complete removal of thermo-lag. Completion letter submitted October 29, 1996.
BL-92-02	08/24/1992	Safety Concerns Related to "End of Life" of Aging Theratronics Teletherapy Units	For Action -All Teletherapy Licensees.

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BL-92-01	06/24/1992	Failure of Thermo-Lag 330 Fire Barrier System ... Cabling in Wide Cable Trays & Small Conduits Free From Fire Damage	TVA's April 14, 1993 letter commits to complete removal of thermo-lag. Completion letter submitted October 29, 1996.
BL-91-01s1	07/27/1993	Reporting Loss of Criticality Safety Controls	For Action - All fuel fabrication facilities.
BL-91-01	10/18/1991	Reporting Loss of Criticality Safety Controls	Addressees - All fuel cycle and uranium fuel research and development licensees.
BL-90-01s1	12/22/1992	Loss of Fill-Oil in Transmitters Manufactured by Rosemount	
BL-90-02	03/20/1990	Loss of Thermal Margin Caused by Channel Box Bow	TVA's April 24, 1990 letter informed NRC that BFN does not use channel boxes for a second bundle lifetime.
BL-90-01	03/09/1990	Loss of Fill-Oil in Transmitters	
BL-89-01s2	06/28/1991	Failure of Westinghouse Steam Generator Tube Mechanical Plugs	Addressees: All holders of operating licenses or construction permits for pressurized-water reactors (PWRs).
BL-89-01s1	11/14/1990	Failure of Westinghouse Steam Generator Tube Mechanical Plugs	Addressees: All holders of operating licenses or construction permits for pressurized-water reactors (PWRs).
BL-89-03	11/21/1989	Potential Loss of Required Shutdown Margin During Refueling Operations	Addressees: All holders of operating licenses or construction permits for pressurized water reactors.
BL-89-02	07/19/1989	Stress Corrosion Cracking of High-Hardness Type 410 .. Bolting .. Anchor Darling S350W .. Swing Check Valves .. Similar	TVA's January 17, 1990 letter informed NRC that BFN does not have any check valves in its safety-related systems within the scope of NRCB 89-02.
BL-89-01	05/15/1989	Failure of Westinghouse Steam Generator Tube Mechanical Plugs	Addressees: All holders of operating licenses or construction permits for pressurized-water reactors (PWRs).
BL-88-10s1	08/03/1989	Nonconforming Molded-Case Circuit Breakers	
BL-88-07s1	12/30/1988	Power Oscillations in Boiling Water Reactors (BWRs)	
BL-88-11	12/20/1988	Pressurizer Surge Line Thermal Stratification	Addressees: All holders of operating licenses or construction permits for pressurized water reactors (PWRs).
BL-88-10	11/22/1988	Nonconforming Molded-Case Circuit Breakers	
BL-88-09	07/26/1988	Thimble Tube Thinning in Westinghouse Reactors	Addressees: All holders of operating licenses or construction permits for Westinghouse (W)-designed nuclear power reactors that utilize bottom mounted instrumentation.
BL-88-08s2	08/04/1988	Thermal Stresses in Piping Connected to Reactor Coolant Systems	TVA response provided October 24, 1988. TVA concluded that there were no unisolable sections of piping connected to the reactor coolant system (RCS) which could be subjected to the thermal stresses described by Bulletin 88-08.
BL-88-05s2	08/03/1988	Nonconforming Materials Supplied by Piping Supplies, Inc. ... and West Jersey Manufacturing Company ...	TVA response provided August 25, 1988. Closed in Inspection Report 90-33.
BL-88-08s1	06/24/1988	Thermal Stresses in Piping Connected to Reactor Coolant Systems	TVA response provided October 24, 1988. TVA concluded that there were no unisolable sections of piping connected to the reactor coolant system (RCS) which could be subjected to the thermal stresses described by Bulletin 88-08.
BL-88-08	06/22/1988	Thermal Stresses in Piping Connected to Reactor Cooling Systems	TVA response provided October 24, 1988. TVA concluded that there were no unisolable sections of piping connected to the reactor coolant system (RCS) which could be subjected to the thermal stresses described by Bulletin 88-08.
BL-88-07	06/15/1988	Power Oscillations in Boiling Water Reactors (BWRs)	

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File Name	Date	Description	Response
BL-88-06	06/14/1988	Actions To Be Taken for the Transportation of Model No. SPEC 2-T Radiographic Exposure Device	Addressees: All NRC licensees authorized to manufacture, distribute, or operate radiographic exposure devices or source changers.
BL-88-05s1	06/15/1988	Nonconforming Materials Supplied by Piping Supplies, Inc. ... and West Jersey Manufacturing Company ...	TVA response provided August 25, 1988. Closed in Inspection Report 90-33.
BL-88-05	05/06/1988	Nonconforming Materials Supplied by Piping Supplies, Inc. ... and West Jersey Manufacturing Company ...	TVA response provided August 25, 1988. Closed in Inspection Report 90-33.
BL-88-04	05/05/1988	Potential Safety-Related Pump Loss	
BL-88-08s3	04/11/1989	Thermal Stresses in Piping Connected to Reactor Coolant Systems	TVA response provided October 24, 1988. there are no unisolable sections of piping connected to the RCS which can be subjected to stresses from temperature stratification or temperature oscillations which could induce unacceptable cycling stress fatigue.
BL-88-03	03/10/1988	Inadequate Latch Engagement in HFA Type Latching Relays Manufactured by General Electric (GE) Company	
BL-88-02	02/05/1988	Rapidly Propagating Fatigue Cracks in Steam Generator Tubes	For Action - All holders of operating licenses or construction permits for Westinghouse (W)-designed nuclear power reactors with steam generators having carbon steel support plates.
BL-88-01	02/05/1988	Defects in Westinghouse Circuit Breakers	TVA response, dated April 13, 1988, stated the Bulletin was not applicable to BFN.
BL-87-02s2	06/10/1988	Fastener Testing to Determine Conformance with Applicable Material Specifications	TVA responded by letters dated March 16, 1988, April 15, 1988, July 6, 1988, September 12, 1988, and January 27, 1989. Closed by NRC letter dated August 18, 1989.
BL-87-02s1	04/22/1988	Fastener Testing to Determine Conformance with Applicable Material Specifications	TVA responded by letters dated March 16, 1988, April 15, 1988, July 6, 1988, September 12, 1988, and January 27, 1989. Closed by NRC letter dated August 18, 1989.
BL-87-02	11/06/1987	Fastener Testing to Determine Conformance with Applicable Material Specifications	TVA responded by letters dated March 16, 1988, April 15, 1988, July 6, 1988, September 12, 1988, and January 27, 1989. Closed by NRC letter dated August 18, 1989.
BL-87-01	07/09/1987	Thinning of Pipe Walls in Nuclear	The purpose of this bulletin is to request that licensees submit information concerning their programs for monitoring the thickness of pipe walls in high-energy single-phase and two-phase carbon steel piping systems. TVA responded by letter dated September 18, 1987. SER dated August 31, 1988.
BL-86-04	10/29/1986	Defective Teletherapy Timer That May Not Terminate Treatment Dose	Addressees: All NRC licensees authorized to use cobalt-60 teletherapy units.
BL-86-03	10/08/1986	Potential Failure of Multiple ECCS Pumps Due to Single Failure of Air-Operated Valve in Minimum Flow Recirculation Line	TVA responded by letter dated November 14, 1986. The single failure vulnerability discussed in this bulletin does not exist at any of the our nuclear facilities.
BL-86-02	07/18/1986	Static "O" Ring Differential Pressure Switches	
BL-86-01	05/23/1986	Minimum Flow Logic Problems That Could Disable RHR Pumps	TVA responded by letter dated June 2, 1986. BFN's as-constructed configuration precludes the single failure vulnerability regarding the minimum flow valves for the RHR system.

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File Name	Date	Description	Response
BL-85-03s1	04/27/1988	Motor-Operated Valve Common Mode Failures During Plant Transients due to Improper Switch Settings	TVA responded by letters dated May 13, 1986, September 30, 1986, May 1, 1987, and July 15, 1988. Outstanding programmatic commitments to be implemented under program that addresses Generic Letter 89-10.
BL-85-03	11/15/1985	Motor-Operated Valve Common Mode Failures During Plant Transients Due to Improper Switch Settings	TVA responded by letters dated May 13, 1986, September 30, 1986, May 1, 1987, and July 15, 1988. Outstanding programmatic commitments to be implemented under program that addresses Generic Letter 89-10.
BL-85-02	11/05/1985	Undervoltage Trip Attachments of Westinghouse DB-50 Type Reactor Trip Breakers	TVA responded by letter dated December 3, 1985. Brownsr Ferry does not utilize Westinghouse DR-S0 reactor trip breakers.
BL-85-01	10/29/1985	Steam Binding of Auxilliary Feedwater Pumps	Certain PWR licensees and all PWR CP holders are requested to take further action to prevent similar events from occurring at their facilities and to document those actions taken or planned.

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Generic Letters			
gl99002er	08/23/1999	NRC Generic Letter 99-02 (Errata): Laboratory Testing of Nuclear-Grade Activated Charcoal	November 18, 1999 TVA letter closed for all three units. Acceptable testing standards references in BFN Technical Specifications.
gl99002	06/03/1999	NRC Generic Letter 99-02: Laboratory Testing of Nuclear-Grade Activated Charcoal	November 18, 1999 TVA letter closed for all three units. Acceptable testing standards references in BFN Technical Specifications.
gl99001	05/03/1999	NRC Generic Letter 99-01: Recent Nuclear Material Safety and Safeguards Decision on Bundling Exempt Quantities	Addressees: All materials licensees.
gl98005	11/10/1998	NRC Generic Letter 98-05: Boiling Water Reactor Licensees Use of the BWRVIP-05 Report to Request Relief from Augmented Examination Requirements on Reactor Pressure Vessel Circumferential Shell Welds	This generic letter requires no specific action or written response.
gl98004	07/14/1998	NRC Generic Letter 98-04: Potential for Degradation of the Emergency Core Cooling System and the Containment Spray System after a Loss-of-Coolant Accident Because of Construction and Protective Coating Deficiencies and Foreign Material in Containment	
gl98003	06/22/1998	NRC Generic Letter 98-03: NMSS Licensees' and Certificate Holders' Year 2000 Readiness Programs	For Action: All licensees or certificate holders for uranium hexafluoride production plants, uranium enrichment plants, and uranium fuel fabrication plants, except those that have permanently ceased operations.
gl98002	05/28/1998	NRC Generic Letter No. 98-02: Loss of Reactor Coolant Inventory and Associated Potential for Loss of Emergency Mitigation Functions While in a Shutdown Condition	Addressees: All holders of operating licenses for pressurized-water reactors (PWRs), except those who have permanently ceased operations, and have certified that fuel has been permanently removed from the reactor vessel.
gl98001	05/11/1998	NRC Generic Letter No. 98-01: Year 2000 Readiness of Computer Systems at Nuclear Power Plants	
gl98001s1	01/14/1999	NRC Generic Letter No. 98-01 Supplement 1: Year 2000 Readiness of Computer Systems at Nuclear Power Plants	
gl97006	12/30/1997	NRC Generic Letter 97-06: Degradation of Steam Generator Internals	Addressees: All holders of operating licenses for pressurized-water reactors (PWRs), except those who have permanently ceased operations and have certified that fuel has been permanently removed from the reactor vessel.
gl97005	12/17/1997	NRC Generic Letter 97-05: Steam Generator Tube Inspection Techniques	Addressees: All holders of operating licenses for pressurized-water reactors (PWRs), except those who have permanently ceased operations and have certified that fuel has been permanently removed from the reactor vessel.
gl97004	10/07/1997	NRC Generic Letter 97-04: Assurance of Sufficient Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal Pumps	
gl97003	07/09/1997	Annual Financial Update of Surety Requirements For Uranium Recovery Licensees	Addressees: Uranium Recovery Licensees and State Officials.
gl97002	05/15/1997	Revised Contents of the Monthly Operating Report	This generic letter requires no specific action or written response.
gl97001	04/01/1997	Degradation of Control Rod Drive Mechanism Nozzle and Other Vessel Closure Head Penetrations	Addressees: All holders of operating licenses for pressurized water reactors (PWRs).

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File Name	Date	Description	Response
gl96007	12/05/1996	Interim Guidance on Transportation of Steam Generators	Addressees: All holders of operating licenses and decommissioning facilities with possession-only licenses for pressurized-water nuclear power reactors.
gl96006s1	11/13/1997	NRC Generic Letter 96-06, Supplement 1: Assurance of Equipment Operability and Containment Integrity During Design-Basis Accident Conditions	
gl96006	09/30/1996	Assurance of Equipment Operability And Containment Integrity During Design-Basis Accident Conditions	
gl96005	09/18/1996	Periodic Verification of Design-Basis Capability of Safety-Related Power-Operated Valves	
gl96004	06/26/1996	Boraflex Degradation in Spent Fuel Pool Storage Racks	TVA's October 22, 1996 letter completed requested actions. No boraflex used.
gl96002	02/13/1996	Reconsideration of Nuclear Power Plant Security Requirements Associated With an Internal Threat	This generic letter requires no specific action or written response.
gl96003	01/31/1996	NRC Generic Letter 96-03: Relocation of The Pressure Temperature Limit Curves And Low Temperature Overpressure Protection System Limits	Licensees and applicants who choose to adopt this line item improvement are encouraged to propose changes that are consistent with the attached guidance.
gl96001	01/10/1996	Testing of Safety-Related Circuits	
gl950010	12/15/1995	Relocation of Selected Technical Specifications Requirements Related to Instrumentation	Licensees that do not wish to amend technical specifications are not expected to submit any response to this generic letter.
gl95009	11/03/1995	Monitoring and Training of Shippers and Carriers of Radioactive Materials	No specific action nor written response is required.
gl95008	10/31/1995	10 CFR 50.54(p) Process for Changes to Security Plans Without Prior NRC Approval	No specific actions or written response is required.
gl95007	08/17/1995	Pressure Locking and Thermal Binding of Safety-Related Power-Operated Gate Valves	
gl95006	08/15/1995	Changes in the Operator Licensing Program	This generic letter requires no specific action or written response.
gl95005	08/03/1995	Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes Affected by Outside Diameter Stress Corrosion Cracking	Addressees: All holders of operating licenses or construction permits for pressurized-water reactors (PWRs).
gl95003	04/28/1995	Circumferential Cracking of Steam Generator Tubes	Addressees: All holders of operating licenses or construction permits for pressurized water reactors (PWRs).
gl95004	04/28/1995	Final Disposition of the Systematic Evaluation Program Lessons-Learned Issues	No specific action or written response is required.
gl95002	04/26/1995	Use of Numarc/epri Report TR-102348, "Guideline on Licensing Digital Upgrades," in Determining the Acceptability of Performing Analog-to-Digital Replacements under 10 CFR 50.59	No specific action or written response is required.
gl95009s1	04/05/1996	Monitoring and Training of Shippers and Carriers of Radioactive Materials	No specific action or written response is required.
gl95006s1	01/31/1997	Changes in Operator Licensing Program	This generic letter requires no specific action or written response.
gl95001	01/26/1995	NRC Staff Technical Position on Fire Protection for Fuel Cycle Facilities	Addressees: All current licensees and applicants for uranium conversion and fuel fabrication facilities.
gl94004	09/02/1994	Voluntary Reporting of Additional Occupational Radiation Exposure Data	This generic letter only requests voluntary submittal of information.
gl94003	07/25/1994	Intergranular Stress Corrosion Cracking of Core Shrouds in Boiling Water Reactors	
gl94002	07/11/1994	Long-Term Solutions and Upgrade of Interim Operating Recommendations for Thermal-Hydraulic Instabilities in BWRs	

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gl94001	05/31/1994	Removal of Accelerated Testing and Special Reporting Requirements for Emergency Diesel Generators	Licensee action to propose TS changes or relief from a docketed commitment under the guidance of this generic letter is voluntary.
gl93008	12/29/1993	Relocation of Technical Specification Tables Of Instrument Response Time Limits	Licensee action to propose TS changes under the guidance of this generic letter is voluntary.
gl93007	12/28/1993	Modification of the Technical Specification Administrative Control Requirements For Emergency and Security Plans	Licensee action to propose TS changes under the guidance of this generic letter is voluntary.
gl93006	10/25/1993	Research Results on Generic Safety Issue 106, "Piping and the Use of Highly Combustible Gases In Vital Areas"	In this generic letter, the NRC is only communicating information on results of government-sponsored research to resolve a generic safety issue and is not recommending that licensees or applicants take particular courses of action or requesting that licensees communicate information back to the NRC on this matter.
gl93003	10/20/1995	Verification of Plant Records	This generic letter requires no specific action or written response.
gl93005	09/27/1993	Line-Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing During Power Operation	Licensee action to propose TS changes under the guidance of this generic letter is voluntary.
gl93004	08/21/1993	Rod Control System Failure and Withdrawal of Rod Control Cluster Assemblies 10 CFR 50.54(f)	FOR ACTION - ALL HOLDERS OF OPERATING LICENSES OR CONSTRUCTION PERMITS FOR WESTINGHOUSE (W)-DESIGNED NUCLEAR POWER REACTORS EXCEPT HADDAM NECK
gl93002	03/23/1993	NRC Public Workshop on Commercial Grade Procurement and Dedication	This generic letter contains no information collection requirements.
gl93001	03/03/1993	Emergency Response Data System Test Program	In this generic letter, the staff recommends but does not require (or seek to impose) an ERDS testing schedule and procedure to implement the testing required by the rule.
gl92009	12/31/1992	Limited Participation By NRC in the IAEA International Nuclear Event Scale	By this generic letter, the NRC staff does not request any licensee or applicant to submit information to the NRC, does not recommend any new regulatory action, and does not modify any existing regulatory position of the NRC staff.
gl92008	12/17/1992	Thermo-Lag 330-1 Fire Barriers	TVA's April 14, 1993 letter commits to complete removal of thermo-lag. Completion letter submitted October 29, 1996.
gl92007	10/20/1992	Office of Nuclear Reactor Regulation Reorganization	This generic letter contains no information collection requirements.
gl92006	09/16/1992	Operator Licensing National Examination Schedule	Your updated schedules, to be voluntarily submitted, should be sent to
gl92005	09/04/1992	NRC Workshop on the Systematic Assessment of Licensee Performance (SALP) Program	This generic letter contains no information collection requirements.
gl92004	08/19/1992	Resolution of the Issues Related to Reactor Vessel Water Level Instrumentation in BWRs Pursuant to 10 CFR 50.54(f)	
gl92001rs.html	05/19/1995	NRC Generic Letter 92-01, Revision 1, Supplement 1: Reactor Vessel Structural Integrity	
gl92001rs	05/19/1995	NRC Generic Letter 92-01, Revision 1, Supplement 1: Reactor Vessel Structural Integrity	
gl92003	03/19/1992	Compilation of the Current Licensing Basis: Request for Voluntary Participation in Pilot Program	In issuing this letter, the NRC is imposing no new requirements or staff positions.
gl92002	03/06/1992	Resolution of Generic Issue 79, "Unanalyzed Reactor Vessel (PWR) Thermal Stress During Natural Convection Cooledown"	In issuing this letter, the NRC is imposing no new requirements or staff positions.

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gl92001r1		Reactor Vessel Structural Integrity	
gl92001	02/28/1992	Reactor Vessel Structural Integrity	
gl91019	12/19/1991	Information to Addressees Regarding New Telephone Numbers for NRC Offices Located in One White Flint North	No specific action or written response is required.
gl91018	11/07/1991	Information ... Regarding Two NRC ... [IPs] On Resolution of Degraded and Nonconforming Conditions and On Operability	No specific licensee actions are required.
gl91017	10/17/1991	Generic Safety Issue 29, "Bolting Degradation or Failure in Nuclear Power Plants"	However, the suggestions contained in this letter do not constitute NRC requirements; therefore no specific action or written response is required.
gl91018r1	10/08/1997	NRC Generic Letter 91-18, Revision 1: Information to Licensees Regarding NRC Inspection Manual Section on Resolution of Degraded and Nonconforming Conditions	This generic letter requires no specific action or response.
gl91016	10/03/1991	Licensed Operators' and Other Nuclear Facility Personnel Fitness For Duty	This generic letter requires no specific action or written response.
gl91014	09/23/1991	Emergency Telecommunications	Since this letter initiates no information gathering and requires no reply, no OMB clearance number is required.
gl91015	09/23/1991	Operating Experience Feedback Report: Solenoid-Operated Valve Problems at U.S. Reactors	No specific action or written response is required by this generic letter.
gl91013	09/19/1991	Request for Info Related to the Resolution of GI 130, "Essential Service Water System Failures at Multi-Unit Sites"	TO: LICENSEES AND APPLICANTS OF THE FOLLOWING PRESSURIZED WATER REACTOR NUCLEAR POWER PLANTS:
gl91010	08/27/1991	Explosives Searches at Protected Area Portals	TO ALL FUEL CYCLE FACILITY LICENSEES WHO POSSESS, USE, IMPORT, OR EXPORT FORMULA QUANTITIES OF STRATEGIC SPECIAL NUCLEAR MATERIAL
gl91012	07/18/1991	Operator Licensing National Examination Schedule	Your updated schedules, to be voluntarily submitted, should be returned to
gl91011	07/08/1991	Resolution of GIs 48, "LCOs ... 1E Vital Inst. Buses," and 49, "Interlocks ... Tie Breakers," Pursuant to 10 CFR 50.54	TVA's April 30, 1992 completed for all three units. BFN has completed its evaluation and has concluded that the administrative controls in place satisfy the recommendations of the GI.
gl91009	06/27/1991	Modification of Surveillance Interval ... Electrical Protection Assemblies in Power Supplies ... Reactor Protection System	This letter does not require any licensee to propose changes to its plant TS.
gl91008	05/06/1991	Removal of Component Lists From Technical Specifications	Action taken in response to the guidance provided in this generic letter is voluntary and is not a backfit under 10 CFR 50.109.
gl91007	05/02/1991	GI-23, "Reactor Coolant Pump Seal Failures" and Its Possible Effect on Station Blackout	This generic letter consists of information only and does not require specific action or written response.
gl91006		Resolution of Generic Issue A-30, "Adequacy of Safety-Related DC Power Supplies," Pursuant to 10 CFR 50.54(f)	May 27, 1992 documented TVA completion of commitments for all three units.
gl91005	04/09/1991	Licensee Commercial-Grade Procurement and Dedication Programs	No response to this letter is required.
gl91004	04/02/1991	Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle	Any response to the NRC suggestion for TS changes is voluntary.
gl91003	03/06/1992	Reporting of Safeguards Events	The actions in this generic letter are voluntary.
gl91002		Reporting Mishaps Involving LLW Forms Prepared for Disposal	Licensees are also reminded of their obligation to report defects and deviations in accordance with NRC requirements in 10 CFR Part 21.

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gl91001	01/04/1991	Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimens From Technical Specifications	Action taken in response to the guidance provided in this generic letter is voluntary and is not a backfit under 10 CFR 50.109.
gl90007	12/11/1990	Operator Licensing National Examination Schedule	Your updated schedules, to be voluntarily submitted in the enclosed suggested formats.
gl90006	08/10/1990	Resolution of Generic Issues 70, "PORV and Block Valve Reliability," and 94, "Additional LTOP Protection for PWRs"	TO: ALL PRESSURIZED WATER REACTOR LICENSEES AND CONSTRUCTION PERMIT HOLDERS
gl90005	08/10/1990	Guidance for Performing Temporary Non-Code Repair of ASME Code Class 1, 2, and 3 Piping	This generic letter provides guidance that will be considered by the NRC staff in evaluating relief requests submitted by licensees for temporary non-code repairs of code Class 3 piping. Compliance with the staff guidance is not required.
gl90008	07/31/1992	Simulation Facility Exemptions	This Generic Letter is being issued to remind the addressees of the need to meet the schedule requirements for the use of simulation facilities in the administration of operating tests established in Section 55.45(b) of Title 10 of the Code of Federal Regulations (CFR). This Generic Letter also describes the review of exemption requests and the potential consequences for failure to meet the requirements of Section 55.45(b).
gl90004	06/25/1990	Request for Information on the Status of Licensee Implementation of GSIs Resolved With Imposition of Requirements or CAs	The requested status of FSI's was provided on May 30, 1990. There were no commitments made in the submittal.
gl90003s1	06/15/1990	Relaxation of Staff Position In Generic Letter 83-28, Item 2.2 Part 2, "Vendor Interface for Safety-Related Components"	This letter is for information purposes only and no specific action or written response is required.
gl90009	05/14/1990	Alternative Requirements for Shorter Visual Inspection Intervals and Corrective Actions	Compliance with the alternative TS requirements for visual inspection intervals is voluntary.
gl90003	04/25/1990	Relaxation of Staff Position in Generic Letter 83-28, Item 2.2 Part 2 "Vendor Interface for Safety-Related Components"	TVA responded on August 24, 1990. TVA notified NRC that all commitments had been completed by letter dated October 15, 1990.
gl90002s1	03/20/1990	Alternative Requirements for Fuel Assemblies in the Design Features Section of Technical Specifications	No response is required to this letter.
gl90002	02/01/1990	Alternative Requirements for Fuel Assemblies in the Design Features Section of Technical Specifications	This enclosure provides guidance for the preparation of a license amendment request to modify the Technical Specifications (TS) requirements for "Fuel Assemblies" in the Section 5, "Design Features," of TS. No response was required.
gl90001	01/18/1990	Request for Voluntary Participation	VOLUNTARY PARTICIPATION IN NRC REGULATORY IMPACT SURVEY
gl89001s1	11/14/1990	NUREG-1301 and 2 - "Offsite Dose Calculation Manual Guidance: Standard Radiological Effluent Controls for PWRs and BWRs"	Enclosed, for your information, is a copy of the subject NUREG report. This NUREG report does not impose any new requirements or staff positions; its use is therefore totally voluntary.
gl89010s3	10/25/1990	Consideration of the Results of NRC-Sponsored Tests of Motor-Operated Valves	
gl89023	10/23/1989	NRC Staff Responses To Questions Pertaining To Implementation Of 10 CFR Part 26	The attached NUREG does not constitute any new requirements beyond 10 CFR Part 26. Furthermore, transmittal of the NUREG does not impose any new regulatory requirements.
gl89022	10/19/1989	Potential For Increased Roof Loads Due To Recent Change In Probable Maximum Precipitation Criteria...	This information is provided for your review and determination of appropriate action, if any. No submittal is required.

BROWNS FERRY UNIT 1
RESPONSE TO GENERIC COMMUNICATIONS ISSUED SINCE 1985

File Name	Date	Description	Response
gl89021	10/19/1989	Request For Information Concerning Status Of Implementation of Unresolved Safety Issue (USI) Requirements	TVA provided the requested status information by letter dated November 15, 1989. There were no commitments made in the letter.
gl89020	09/26/1989	Protected Area Long-Term Housekeeping	TO ALL FUEL CYCLE FACILITY LICENSEES WHO POSSESS, USE, OR PROCESS FORMULA QUANTITIES OF STRATEGIC SPECIAL NUCLEAR MATERIAL
gl89019	09/20/1989	Request For Actions Related To Resolution Of Unresolved Safety Issue A-47, "...," Pursuant To 10 CFR 50.54(f)	TVA responded by letter dated August 29, 1994. Closure documented by NRC letter dated September 21, 1994.
gl89017	09/06/1989	Planned Administrative Changes To The NRC Operator Licensing Written Examination Process	No written response to the Generic Letter was requested.
gl89018	09/06/1989	Resolution Of Unresolved Safety Issues A-17, "Systems Interactions In Nuclear Power Plants"	No specific action or written response is required by this letter.
gl89016	09/01/1989	Installation of a Hardened Wetwell Vent	
gl89014	08/21/1989	Line-Item Improvements In Technical Specifications - Removal Of 3.25 Limit On Extending Surveillance Intervals	Licensees and applicants are encouraged to propose changes to plant TS that are consistent with the guidance provided in the enclosure. No response was required.
gl89015	08/21/1989	Emergency Response Data System	The purpose of this generic letter is to solicit the participation of those utilities which have not yet volunteered* in the ERDS program.
gl89007s1	08/21/1989	Power Reactor Safeguards Contingency Planning For Surface Vehicle Bombs	No additional burden or response is required by this supplement, which is intended only as clarification of the original request.
gl89010s2	08/03/1990	Availability of Program Descriptions	
gl89011	06/30/1989	Resolution Of Generic Issue 101 "Boiling Water Reactor Water Level Redundancy"	No written response or specific action is required by this letter.
gl89012	07/06/1989	Operator Licensing Examination	Your schedules, to be voluntarily submitted on the enclosed suggested formats, should be returned to ...
gl89013	07/18/1989	Service Water System Problems Affecting Safety-Related Equipment	
gl89010s5	06/28/1993	Inaccuracy of Motor-Operated Valve Diagnostic Equipment	
gl89010	06/28/1989	Safety-Related Motor-Operated Valve Testing And Surveillance	
gl89010s1	06/13/1990	Results of the Public Workshops	
gl89009	05/08/1989	ASME Section III Component Replacements	No written response to this Generic Letter was required.
gl89008	05/02/1989	Erosion/Corrosion-Induced Pipe Wall Thinning	
gl89007	04/28/1989	Power Reactor Safeguards Contingency Planning For Surface Vehicle Bombs	Response was security related. Compliance documented in Inspection Report 96-11.
gl89006	04/12/1989	Task Action Item I.D.2 - Safety Parameter Display System - 10 CFR 50.54 (f)	
gl89013s1	04/04/1990	Service Water System Problems Affecting Safety-Related Equipment	This supplement contains the questions and answers read into the transcripts during the workshops. No response was required.
gl89005	04/04/1989	Pilot Testing Of The Fundamentals Examination	Candidates interested in participating in this program are requested to submit the enclosed application to ...
gl89004s1	04/04/1989	Guidance on Developing Acceptable Inservice Testing Programs	This generic letter only requests applicable information under the provisions of 10 CFR 50.54(f) from addressees who voluntarily choose to use the guidance in NUREG-1482 to make changes to their inservice testing programs.
gl89004	04/03/1989	Guidance On Developing Acceptable Inservice Testing Programs	No written response to this Generic Letter was required.
gl89003	03/24/1989	Operator Licensing Examination Schedule	No written response to this Generic Letter was required.

BROWNS FERRY UNIT 1
RESPONSE TO GENERIC COMMUNICATIONS ISSUED SINCE 1985

File Name	Date	Description	Response
gl89002	03/21/1989	Actions To Improve The Detection Of Counterfeit And Fraudulently Marketed Products	No response to this letter is required.
gl89010s6	03/08/1994	Information on Schedule And Grouping, And Staff Responses to Additional Public Questions	
gl89010s4	02/12/1992	Consideration of Valve Mispositioning in Boiling Water Reactors	
gl89001	01/31/1989	Implementation Of Programmatic And Procedural Controls for Radiological Effluent Technical Specifications	Licensees are encouraged to propose changes to TS that are consistent with the guidance provided in the enclosures. However, no response was required.
gl89010s7	01/24/1996	Consideration of Valve Mispositioning in Pressurized-water Reactors	
gl88020	11/23/1988	Individual Plant Examination For Severe Accident Vulnerabilities	On September 1, 1992, TVA responded to Generic Letter 88-20 for all three units. TVA completed and submitted the single unit BFN PRA.
gl88019	10/28/1988	Use Of Deadly Force By Licensee Guards To Prevent Theft Of Special Nuclear Material	No specific actions or written responses are required by this generic letter.
gl88018	10/20/1988	Plant Record Storage On Optical Disks	The purpose of this generic letter is to inform addressees that the staff approves the use of this method of record keeping when appropriate quality assurance controls are applied. No written response was required.
gl88017	10/17/1988	Loss of Decay Heat Removal	TO ALL HOLDERS OF OPERATING LICENSES OR CONSTRUCTION PERMITS FOR PRESSURIZED WATER REACTORS (PWRs)
gl88016	10/03/1988	Removal Of Cycle-Specific Parameter Limits From Technical Specifications	Licensees and applicants are encouraged to propose changes to TS that are consistent with the guidance provided in the enclosure. However, no written response was required.
gl88015	09/12/1988	Electric Power Systems - Inadequate Control Over Design Process	No specific action or written response is required by this letter.
gl88020s5	09/08/1995	Individual Plant Examination of External Events (IPEEE) for Severe Accident Vulnerabilities - 10 CFR 50.54(f)	Within 60 days of the date of this generic letter, all addressees who voluntarily choose to perform seismic IPEEEs using the modified procedures described above are required to submit a response to the information requested above.
gl88020s1	08/29/1989	Initiation Of The Individual Plant Examination For Severe Accident Vulnerabilities - 10 CFR 50.54	This supplement does not change the scope of Generic Letter 88-20.
gl88014	08/08/1988	Instrument Air Supply System Problems Affecting Safety-Related Equipment	
gl88013	08/08/1988	Operator Licensing Examinations	No written response was required for this Generic Letter.
gl88012	08/02/1988	Removal of Fire Protection Requirements From Technical Specification	Implementation of the recommended Fire Protection Technical Specifications was voluntary.
gl88011	07/12/1988	NRC Position on Radiation Embrittlement Of Reactor Vessel Materials And Its Impact On Plant Operations	
gl88020s3	07/06/1990	Completion of Containment Performance Improvement Program and Forwarding of Insights for Use in the IPE for SAV	This generic letter provides information that may assist licensees in performing their Individual Plant Examination pursuant to Generic Letter 88-20. It does not contain any new requirements and no reply to this generic letter is required.
gl88010	07/01/1988	Purchase of GSA Approved Security Containers	This letter is for information only and does not require a response.
gl88020s4	06/28/1991	Individual Plant Examination of External Events (IPEEE) for Severe Accident Vulnerabilities	

BROWNS FERRY UNIT 1
RESPONSE TO GENERIC COMMUNICATIONS ISSUED SINCE 1985

File Name	Date	Description	Response
gl88009	05/17/1988	Pilot Testing of Fundamentals Examination	No written response was required. However, Licensees interested in participating in this program are requested to submit the enclosed application ...
gl88008	05/03/1988	Mail Sent or Delivered To The Office Of Nuclear Reactor Regulation	This generic letter provides guidance on how mail is to be sent or delivered to the Office of Nuclear Reactor Regulation.
gl88007	04/07/1988	Modified Enforcement Policy ... 10 CFR 50.49, "Environmental Qualification Of Electrical Equipment Important To Safety"	This letter does not require any response and therefore does not need approval of the Office of Management and Budget
gl88020s2	04/04/1990	Accident Management Strategies for Consideration in the Individual Plant Examination Process	This generic letter supplement does not establish any requirements for licensees to take the specific accident management strategies into account as a part of the IPE or to implement any of the strategies. Adoption on the part of licensee of any accident management strategies in response to this supplement is voluntary.
gl88006	03/22/1988	Removal Of Organization Charts From Technical Specification Administrative Control Requirements	Licensees and applicants are encouraged to propose changes to their TS that are consistent with the guidance provided in Enclosure 1. However, compliance was not required.
gl88005	03/17/1988	Boric Acid Corrosion of Carbon Steel Reactor Pressure Boundary Components In PWR plants	TO ALL LICENSEES OF OPERATING PWRs AND HOLDERS OF CONSTRUCTION PERMITS FOR PWRs
gl88004	02/23/1988	Distribution of Gems Irradiated In Research Reactors	This letter is for information only and does not require a response.
gl88003	02/17/1988	Resolution of Generic Safety Issue 93, "Steam Binding of Auxiliary Feedwater Pumps"	TO ALL LICENSEES, APPLICANTS FOR OPERATING LICENSES, AND HOLDERS OF CONSTRUCTION PERMITS FOR PRESSURIZED WATER REACTORS
gl88001s1	02/04/1992	NRC Position on Intergranular Stress Corrosion Cracking (IGSCC) in BWR Austenitic Stainless Steel Piping	No written response is required beyond that contained in GL 88-01.
gl88001	01/25/1988	NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping	
gl88002	01/20/1988	Integrated Safety Assessment Program II (ISAP II)	Participation in the Integrated Safety Assessment Program was voluntary.
gl87016	11/12/1987	Transmittal of NUREG-1262, "Answers To Questions On Implementation Of 10 CFR 55 On Operators' Licenses"	The views expressed in the report are intended as guidance, and are meant to reflect the rule and its statement of considerations. The views are not intended to interpret the rule or any of its provisions, nor does the document increase or decrease any requirements of the rule.
gl87015	11/04/1987	Policy Statement On Deferred Plants	The Policy Statement does not include any new regulatory requirements.
gl87014	08/14/1987	Operator Licensing Examinations	Your schedules, in the enclosed suggested formats, should be returned to ...
gl87013	07/10/1987	Integrity of Regualification Examinations At Non-Power Reactors	TO ALL NON-POWER REACTOR LICENSEES
gl87012	07/09/1987	Loss of Residual Heat Removal While The Reactor Coolant System is Partially Filled	ALL LICENSEES OF OPERATING PWRs AND HOLDERS OF CONSTRUCTION PERMITS FOR PWRs
gl87011	06/19/1987	Relaxation in Arbitrary Intermediate Pipe Rupture Requirements	No written response was required.
gl87010	06/12/1987	Implementation of 10 CFR 73.57, Requirements For FBI Criminal History Checks	No written response was required.
gl87009	06/04/1987	Sections 3.0 And 4.0 of Standard Tech Specs on Limiting Conditions For Operation And Surveillance Requirements	Licensees and applicants are encouraged to propose changes to their TS that are consistent with the guidance provided in the enclosures; however, these changes are voluntary for all licensees and current OL applicants.

BROWNS FERRY UNIT 1
RESPONSE TO GENERIC COMMUNICATIONS ISSUED SINCE 1985

File Name	Date	Description	Response
gl87008	05/11/1987	Implementation of 10 CFR 73.55 Miscellaneous Amendments and Search Requirements	No written response was required.
gl87007	03/19/1987	Information Transmittal of Final Rulemaking For Revisions To Operator Licensing - 10 CFR 55 And Confirming Amendments	No written response was required.
gl87006	03/13/1987	Periodic Verification Of Leak Tight Integrity Of Pressure Isolation Valves	TVA responded by letter dated March 13, 1987. NRC closed by letter dated July 6, 1988.
gl87005		Rqst Add'l Info Assessment ... Degradation Of Mark I Drywells	TVA responded by letter dated May 29, 1987.
gl87004	03/08/1987	Temporary Exemption From Provisions Of The FBI Criminal History Rule For Temporary Workers	No written response was required.
gl87003	02/27/1987	Verification of Seismic Adequacy of Mechanical and Electrical Equipment In Operating Reactors (USI A-46)	The information developed in resolving USI A-46 is described in the enclosures to this letter and is being provided to you for information only.
gl87002	02/19/1987	Verification of Seismic Adequacy of Mechanical and Electrical Equipment In Operating Reactors (USI A-46)	
gl87002s1	02/14/1992	Transmits SSER No. 2 on SQUG Generic Implementation Procedure, Revision 2, as Corrected February 14, 1992 (GIP-2)	
gl87001	01/08/1987	Public Availability Of The NRC	No written response was required.
gl86016	10/22/1986	Westinghouse EGCS Evaluation Models	TO: All Pressurized Water Reactor Applicants and Licensees
gl86017	10/17/1986	Availability of NUREG-1169, "Technical Findings Related to Generic Issue C-8 BWR MSIC Leakage And Treatment Methods"	No action is being requested of applicants or licensees.
gl86015	09/22/1986	Info ... Compliance W 10CFR50.49 "EQ of Electric Equipment Important to Safety For Nuclear Power Plants"	This letter does not require any response.
gl86014	08/20/1986	Operator Licensing Examinations	Your schedules, in the enclosed suggested formats, should be returned to
gl86013	07/23/1986	Potential Inconsistency Between Plant Safety Analyses and Technical Specifications	TO: ALL POWER REACTOR LICENSEES AND APPLICANTS WITH COMBUSTION ENGINEERING AND BABCOCK AND WILCOX PRESSURIZED WATER REACTORS
gl86012	07/03/1986	Criteria for Unique Purpose Exemption From Conversion From The Use of Heu Fuel	TO: ALL NON-POWER REACTOR LICENSEES AUTHORIZED TO USE HEU FUEL
gl86011	06/25/1986	Distribution of Products Irradiated in Research Reactors	TO ALL NON-POWER REACTOR LICENSEES
gl86006	05/29/1986	Implementation Of TMI Action Item II.K.3.5; "Automatic Trip of Reactor Coolant Pumps"	TO ALL APPLICANTS AND LICENSEES WITH COMBUSTION ENGINEERING (CE) DESIGNED NUCLEAR STEAM SUPPLY SYSTEMS (NSSSs) (EXCEPT MAINE YANKEE)
gl86005	05/29/1986	Implementation Of TMI Action Item II.K.3.5; "Automatic Trip Of Reactor Coolant Pumps"	TO ALL APPLICANTS AND LICENSEES WITH BABCOCK AND WILCOX (B&W) DESIGNED NUCLEAR STEAM SUPPLY SYSTEMS (NSSSs)
gl86010	04/24/1986	Implementation of Fire Protection Requirements	No formal response to this Generic Letter was required.
gl86009	03/31/1986	Technical Resolution of Generic Issue B-59, (n-1) Loop Operation in BWRs and PWRs	This Generic Letter does not involve any reporting requirements so that no OMB clearance is necessary.
gl86010s1	03/25/1994	Fire Endurance Test Acceptance Criteria for Fire Barrier Systems Used to Separate Redundant Safe Shutdown Trains Within the Same Fire Area	There were no actions requested or reporting requirements associated with this generic communication.

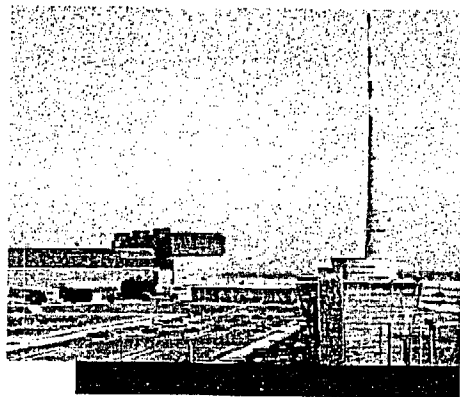
BROWNS FERRY UNIT 1
RESPONSE TO GENERIC COMMUNICATIONS ISSUED SINCE 1985

File Name	Date	Description	Response
gl86008	03/25/1986	Availability of Supplement 4 to NUREG-0933, "A Prioritization of Generic Safety Issues"	This letter does not contain any new requirements or guidance for licensees of operating reactors, applicants for operating licenses, and holders of construction permits. No reply is required.
gl86007	03/20/1986	Transmittal of NUREG-1190 Regarding The San Onofre Unit 1 Loss of Power And Water Hammer Event	This generic letter is provided for information only, and does not involve any reporting requirements.
gl86004	02/13/1986	Policy Statement On Engineering Expertise On Shift	TVA provided the requested information by letter dated May 29, 1986. There were no commitments contained in TVA's letter.
gl86003	02/10/1986	Applications For License Amendments	No specific response to this letter is required.
gl86002	01/23/1986	Technical Resolution of Generic Issue B-19 - Thermal Hydraulic Stability	This generic letter does not include any reporting requirements so that no OMB clearance is necessary.
gl86001	01/03/1986	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	No OMB clearance is required since no information is requested. This information is being provided to BWR applicants and licensees with Mark III Containments for informational purposes only.
gl85022	12/03/1985	Potential For Loss Of Post-LOCA Recirculation Capability Due To Insulation Debris Blockage	No written response or specific action is required by this letter.
gl85021		This Generic Communication was not issued	
gl85020	11/08/1985	Resolution Of Generic Issue 69: High Pressure Injection/Make-Up Nozzle Cracking In Babcock And Wilcox Plants	This generic letter does not impose any new regulatory requirements or any reporting requirements.
gl85019	09/27/1985	Reporting Requirements On Primary Coolant Iodine Spikes	No written response or specific action is required by this letter.
gl85018	09/27/1985	Operator Licensing Examinations	Your schedules, in the enclosed suggested formats, should be returned to
gl85017	08/23/1985	Availability Of Supplements 2 and 3 To NUREG-0933 "A Prioritization Of Generic Safety Issues"	This letter does not contain any new requirements or guidance for licensees of operating reactors, applicants for operating licenses, and holders of construction permits. No reply is required.
gl85016	08/23/1985	High Boron Concentrations	No written response or specific action is required by this letter.
gl85015	08/06/1985	Information On Deadlines For 10CFR50.49, "EQ Of Electric Equipment Important To Safety At Npps"	This letter does not require any response.
gl85014	08/01/1985	Commercial Storage At Power Reactor Sites Of Low Level Radioactive Waste Not Generated By The Utility	No written response or specific action is required by this letter.
gl85013	08/05/1985	Transmittal Of NUREG-1154 Regarding The Davis-Besse Loss Of Main And Auxiliary Feedwater Event	This generic letter is provided for information only, and does not involve any reporting requirements.
gl85012	06/28/1985	Implementation Of TMI Action Item II.K.3.5, "Automatic Trip Of Reactor Coolant Pumps"	TO ALL APPLICANTS AND LICENSEES WITH WESTINGHOUSE (W) DESIGNED NUCLEAR STEAM SUPPLY SYSTEMS (NSSSs)
gl85011	06/28/1985	Completion of Phase II of "Control Of Heavy Loads At Nuclear Power Plants" NUREG-0612	No written response or specific action is required by this letter.
gl85010	05/23/1985	Technical Specification For Generic Letter 83-28, Items 4.3 and 4.4	No written response or specific action is required by this letter.
gl85009	05/23/1985	Technical Specifications For Generic Letter 83-28, Item 4.3	No written response or specific action is required by this letter.
gl85008	05/23/1985	10 CFR 20.408 Termination Reports - Format	No written response or specific action is required by this letter.
gl85007	05/02/1985	Implementation of Integrated Schedules for Plant Modifications	No written response or specific action is required by this letter.
gl85002	04/17/1985	Staff Recommended Actions Regarding Steam Generator Tube Integrity	TO ALL PWR LICENSEES OF OPERATING REACTORS, APPLICANTS FOR OPERATING LICENSES, AND HOLDERS OF CONSTRUCTION PERMITS, AND FT. ST. VRAIN

BROWNS FERRY UNIT 1
RESPONSE TO GENERIC COMMUNICATIONS ISSUED SINCE 1985

File Name	Date	Description	Response
gl85006	01/16/1985	Quality Assurance Guidance for ATWS Equipment That Is Not Safety-Related	Reporting requirements relating to the ATWS rule are presented in 10 CFR 50.62. There are no additional reporting requirements associated with this letter.
gl85005	01/31/1985	Inadvertent Boron Dilution Events	TO ALL PRESSURIZED WATER REACTOR LICENSEES:
gl85004	01/28/1985	Operating Licensing Examinations	Your schedules, in the enclosed suggested format, should be returned to
gl85003	01/28/1985	Clarification of Equivalent Control Capacity for Standby Liquid Control Systems	There are no reporting requirements associated with this generic letter.
gl85001	01/09/1985	Fire Protection Policy Steering Committee Report	Generic Letter (GL) 85-01 was issued only as a DRAFT for comment at public meetings which were held in 1984. However, GL 85-01 was never issued as a final and therefore is not available.
Legend:			
	Listed as open in the framework letter.		
	Not applicable or no response required.		

TENNESSEE VALLEY AUTHORITY BROWNS FERRY NUCLEAR PLANT UNIT 1 REGULATORY FRAMEWORK



NRR - Rockville, MD

April 24, 2003

Attachment 3





Meeting Objective

To facilitate NRC review and approval of the regulatory framework for the restart of Browns Ferry Unit 1.



Agenda

- Background
- Proposed Regulatory Framework for Unit 1
- Licensing and Project Schedule
- Summary and Conclusion



Background

- All three BFN Units are essentially identical GE BWR4, Mark I Containment reactors
- Designed and constructed by TVA
- Units 1, 2, and 3 licensed in 1973, 1974, and 1976 respectively
- All three BFN units voluntarily shutdown by TVA in March 1985, because of regulatory and management issues
 - NRC issued a “show cause” letter for all TVA nuclear plants in September 1985, under 10 CFR 50.54 (f), and requested TVA to specify corrective actions
 - TVA submitted the Nuclear Performance Plan, Volume 3, in August 1986. It outlined the steps needed to recover the BFN units and was specifically directed to Unit 2
 - TVA committed to obtain NRC approval prior to restart of any BFN unit



Background (Cont.)

- NRC considers BFN management concerns successfully resolved on July 31, 1990
- TVA executed Unit 2 restart plan, obtained NRC approval, and restarted Unit 2 on May 24, 1991
- TVA proposed regulatory framework for restart of Units 1 and 3 in July 1991, outlining improvements to the Unit 2 restart plan
- NRC approved the regulatory framework in April 1992
- TVA executed the Unit 3 restart plan, obtained NRC approval, and restarted Unit 3 on November 19, 1995
- TVA requested Unit 1 be removed from the “Watch List” on April 16, 1996
 - Major management and cultural changes implemented at TVA



Background (Cont.)

- NRC removed Unit 1 from the “Watch List” on June 21, 1996 based on TVA commitments
 - Implement the same special programs employed for Unit 3 recovery
 - Not to restart Unit 1 without Commission approval
- TVA Board of Directors decided on May 16, 2002, to restart Unit 1 after detailed study and favorable Supplemental Environmental Impact Statement
- Unit 1 Restart Principles
 - BFN units will be operationally the same upon completion of Unit 1 restart
 - Unit 1 restart activities will use plant processes and procedures
- TVA submitted a proposed update to the regulatory framework for Unit 1 on December 13, 2002
 - Addresses regulatory requirements, special programs, commitments and Technical Specification changes implemented for restart of Units 2 and 3

Proposed Regulatory Framework for Unit 1



- TVA needed to update the Unit 1 regulatory framework
 - Reflect the current list of Technical Specifications and generic communications
 - Incorporate lessons-learned from Unit 3 restart
 - Make framework consistent with changes in regulatory requirements and processes since the Unit 1 framework was developed in 1991
 - Propose methodology for satisfying Unit 1 License Condition
- Supplement submitted on February 28, 2003
 - Bulletin 79-12 inadvertently omitted from list of open generic communications
- NRC's March 11, 2003 letter states that Manual Chapter 0350 applies to plants with performance problems, which is not the case for Browns Ferry Unit 1

Proposed Regulatory Framework for Unit 1 (Cont.)



- Review of the proposed Unit 1 regulatory framework
 - Enclosure 1
 - Generic Communications
 - Special Programs
 - Commitments
 - Operational Readiness
 - Communications
 - Technical Specifications

Proposed Regulatory Framework for Unit 1 (Cont.)



- Generic Communication (Three categories)
 - Commitments defined and tracked.
(See Table 1, Page E1-7*)
 - Commitment not explicitly defined. Submittal required.
(See Table 1, top half of Page E1-8)
 - Commitments defined and tracked. Originating generic communication required notification of completion within a specific timeframe.
(See Table 1, bottom half of Page E1-8)

* - Note that Bulletin 79-12 was inadvertently omitted from the table. Correction was submitted on February 28, 2003.

Proposed Regulatory Framework for Unit 1 (Cont.)



- Generic Communication (Completion)
 - TVA will formally notify NRC when these issues are completed
 - Issues will be completed prior to fuel load unless completion physically tied to work that can only be completed after fuel load
 - TVA will strive to meet notification timeframes requested in the generic communication, but these timeframes are no longer as significant
 - New generic communications will be resolved for Unit 1 in accordance with their schedule requirements

Proposed Regulatory Framework for Unit 1 (Cont.)



- Special Programs
 - Definition of a “Special Program”
 - A corrective action plan to resolve major issues or address an area of specific concern
 - Eighteen Special Programs (Thirty one subprograms) listed in the framework letter
 - Five closed for all three units
 - Twenty six open for Unit 1 (Listed in Table 3, Page E1-10)
 - For Unit 2 restart, NRC issued Safety Evaluation Reports for both design criteria and program implementation methodology
 - For Unit 3 restart, NRC issued Safety Evaluation Reports for all changes to design criteria and most changes in program implementation

Proposed Regulatory Framework for Unit 1 (Cont.)



- Special Programs (Cont.)
 - Lessons learned from Unit 3 Restart
 - TVA intends to implement the programs in accordance with the Unit 3 implementation and criteria precedents
 - Implementation can be more efficient in certain areas (e.g., drywell steel)
 - Changes to design criteria and program implementation need to be addressed using today's regulatory requirements and processes (i.e., 50.59 and 50.90)
 - Commitment closure packages for each special program will be prepared by the line organization, reviewed by Licensing, and closure documented in accordance with TVA's commitment management process
 - TVA will formally notify NRC upon completion of each of these special programs

Proposed Regulatory Framework for Unit 1 (Cont.)



- Commitments
 - TVA's commitment management program is consistent with NEI 99-04, "Guidelines for Managing NRC Commitments", which was endorsed by Regulatory Issue Summary 00-17
 - TVA does not intend to formally notify NRC of the completion of each commitment
 - TVA's processes ensure that all restart commitments will be completed
 - In the Framework letter, TVA proposes that previous "commitments", that are not commitments under today's regulatory guidance, be dispositioned or implemented using current plant processes.

Proposed Regulatory Framework for Unit 1 (Cont.)



- Operational Readiness
 - Self Assessments
 - Implementation of the special programs
 - Additional resources required for three unit operation
 - Completion of restart prerequisites
 - Independent Reviews
 - Nuclear Assurance
 - Nuclear Safety Review Board
 - INPO

Proposed Regulatory Framework for Unit 1 (Cont.)



- Communications
 - Periodic project status meetings
 - Issue-specific meetings as needed

Proposed Regulatory Framework for Unit 1 (Cont.)



- Validation of Current Unit 1 Technical Specifications
 - NRC approved the Unit 1 Improved Technical Specification (ITS) conversion with the following License Condition (2.C.4):
 - The licensee shall review the Technical Specification (TS) changes made by License Amendment No. 234 and any subsequent TS changes, verify that the required analyses and modifications needed to support the changes are complete, and submit them for NRC review and approval prior to entering the mode for which the TS applies. This amendment is effective immediately and shall be implemented prior to entering the mode for which the TS applies.
 - TVA proposal to satisfy the License Condition
 - TVA will formally notify NRC that Unit 1 specific analyses have been completed to support Amendment 234 (ITS conversion) and TS 405 (new source term) for which the Unit 1 analyses were not complete
 - If any Technical Specification changes are required as a result of these analyses, TVA will request an amendment

Proposed Regulatory Framework for Unit 1 (Cont.)



- Validation of Current Unit 1 Technical Specifications (Background)
 - TVA converted to the ITS format for BFN Units 1-3 while Unit 1 was in its extended outage
 - Required analyses for Unit 1 identified and documented in TVA's application for conversion to the ITS format
 - ITS Table 3.3.1.1-1 - Calibration Frequencies for High Reactor Pressure, High Drywell Pressure, and Reactor Low Water Level
 - ITS Table 3.3.5.1-1 - Calibration Frequencies for Reactor Low Water Level, High Drywell Pressure, Reactor Low Pressure, and Reactor High Water Level
 - ITS Table 3.3.5.2-1, Calibration Frequencies for Reactor Low Water Level and Reactor High Water Level
 - ITS Table 3.3.6.1-1 – Reactor Water Cleanup (RWCU), Reactor Core Isolation Cooling (RCIC) and High Pressure Coolant Injection (HPCI) Torus and Pump Room temperature functions and calibration frequencies. Reactor Low Water Level and High Drywell Pressure calibration frequencies. HPCI/RCIC steam supply low pressure and exhaust diaphragm high pressure instrument checks.

Proposed Regulatory Framework for Unit 1 (Cont.)



- Validation of Current Unit 1 Technical Specifications (Background - Cont.)
 - Required analyses for Unit 1 identified and documented in TVA's application for conversion to the Improved Technical Specification format (Cont.)
 - ITS Table 3.3.6.2-1 – Reactor Low Water Level and High Drywell pressure calibration frequencies
 - ITS Table 3.3.7.1-1 – High Drywell pressure calibration frequency
 - Unit 1 analyses completed for all amendments implemented since NRC approved the Unit 1 ITS conversion (TS 234)

Proposed Regulatory Framework for Unit 1 (Cont.)



- Validation of Current Unit 1 Technical Specifications (Background - Cont.)
 - Proposed use of the alternate source term (TS 405) specifically stated that Unit 1 LOCA, Main Steam Line Break and Control Rod Drop accidents analyses would be completed and submitted to NRC prior to restart in accordance with License Condition 2.C.4
 - Times or setpoints revised in LCO 3.1.7, SR 3.1.7.3, new Section 3.9.9 and Program/Manuals 5.5.7
 - Other ITS sections affected include Tables 3.3.6.2-1 and 3.3.7.1-1, LCOs 3.6.4.1, 3.6.4.2, 3.6.4.3, and 3.7.3
 - TVA will formally notify NRC that Unit 1 specific analyses have been completed to support Amendment 234 and TS 405 for which the Unit 1 analyses were not complete
 - If any Technical Specification changes are required as a result of these analyses, TVA will request an amendment

Proposed Regulatory Framework for Unit 1 (Cont.)



- Technical Specifications needed for Unit 1 restart
 - All but three of the Technical Specifications listed in Table 2 (Page E1-9) were previously approved for Units 2 and 3
 - Reduction in the number of ECCS systems needed to respond to a LOCA (applies to all three units)
 - Deletion of LPCI MG sets
 - Miscellaneous I&C changes



Unit 1 Project Overview and Schedule

- Licensing Schedule
 - Forecast submittal dates routinely communicated by use of the Unit 1 Licensing Project Plan
 - Regulatory Framework submitted on December 13, 2002
 - License renewal to be submitted in December 2003
 - Majority of Technical Specifications to be submitted in 2004
 - Extended Power Uprate to be submitted in June 2005
 - Unit 1 restart currently scheduled for May 2007



Summary and Conclusion

TVA PROPOSAL	JUSTIFICATION
Generic Communications	
List of open generic communications is as shown in Table 1.	Extensive reviews performed.
Compliance with timeframe for completion notification contained in the original generic communication is no longer considered a requirement.	Timeframes are no longer as significant.
Special Programs	
TVA may modify implementation precedent for Special Programs at its discretion.	No current requirement or need for regulatory review of implementation methodology.
Changes in BFN design criteria will be evaluated under 10 CFR 50.59. Annual summary of changes to be submitted to NRC. Changes requiring NRC approval will be submitted as a License amendment.	Need and process for obtaining NRC approval determined using current regulatory requirements (10 CFR 50.59 and 10 CFR 50.90). Notification exceeds regulatory requirements (24 months).



Summary and Conclusion (Cont.)

TVA PROPOSAL	JUSTIFICATION
TVA Commitments	
Previous “commitments”, that are not commitments under today’s regulatory guidance, will be dispositioned or implemented using current plant processes.	Consistent with NEI 99-04, “Guidelines for Managing NRC Commitments, which was endorsed by Regulatory Issue Summary 00-17.
TVA will not notify NRC of the completion of each commitment. TVA will notify NRC of the completion of Special Programs and generic communications as noted in Table 1.	Consistent with NEI 99-04, “Guidelines for Managing NRC Commitments, which was endorsed by Regulatory Issue Summary 00-17.
Operational Readiness Review	
Will focus on changes associated with operating a three unit site.	TVA has demonstrated excellence in operating Units 2 and 3.
Will consist of self-assessments and independent reviews.	Provides sufficient internal oversight.



Summary and Conclusion (Cont.)

TVA PROPOSAL	JUSTIFICATION
Communications Protocol	
Periodic project status meetings.	Maximizes TVA / NRC resource utilization.
Issue specific meetings, as requested.	Maximizes TVA / NRC resource utilization.
Technical Specifications	
TVA will satisfy License Condition 2.C.4 by submitting a list of Unit 1 TS changes that were made without the supporting analyses and verifying that the required analyses have been completed.	Maximizes TVA / NRC resource utilization. Satisfies License Condition wording and intent.