May 16, 2003

LICENSEE: Rochester Gas & Electric (RG&E)

FACILITY: R. E. Ginna Nuclear Power Plant (Ginna)

SUBJECT: SUMMARY OF TELECOMMUNICATION WITH ROCHESTER GAS & ELECTRIC CORPORATION (RG&E) TO DISCUSS THE RESPONSE TO THE LICENSE RENEWAL APPLICATION (LRA) RAIs 4.2.1.-1 and 4.2.2.-1 - R. E. GINNA NUCLEAR POWER PLANT (Ginna)

On April 23, 2003, the NRC staff (the staff) and representatives from RG&E held a telecommunication (telecon) to discuss the draft response from RG&E dated April 11, 2003, to the NRC staff request for additional information (RAI). The telecon related to the use of Regulatory Guide 1.190 to perform the neutron fluence calculation in reply to the staff RAI 4.2.2-1. The staff requested clarification of the data contained in the vendor report WCAP-15885, Revision 0, "R. E. Ginna Heatup and Cooldown Limit Curves for Normal Operation," July 2002, describing the radiation analysis and neutron dosimetry used in Ginna's reactor vessel. A list of telephone participants is enclosed.

Staff Question:

The staff requested clarification for the calculated maximum pressure vessel exposures (fluence) shown in Tables 6 and 7 of WCAP-15885, as a function of cumulative operating time (Effective Full Power Years, (EFPY)). Specifically, the staff was concerned about the calculated maximum fluence of the intermediate to nozzle shell circumferential weld and the nozzle shell course clad/base metal interface values as shown in Table 7 at 54 EFPY. These were shown to be in the order of magnitude of E+18. This value seemed low when compared with corresponding values for clad/base metal interface fluence values shown in Table 6, which were approximately at E+19. The staff thought the value of the fluence calculated in Table 7 seemed rather low, as they could not readily corroborate a physical location in the shell courses that corresponded to this fluence. The staff felt the calculational location may not be lying in the vessel belt line.

Telecon Discussion:

The licensee informed the staff that the circumferential weld was located 10-inches above the upper active fuel region. The licensee proposed that they will resolve the issue either by performing additional three dimensional computations by including additional shell regions beyond the current extent, and verify the accuracy and the validity of reported fluence values in Table 7. This alternative will involve additional effort and cost. Alternatively, the licensee indicated they could perform a gross over estimation (by a factor of two) of the fluence at the weld and the results of such overestimate would help identify whether the assumptions are conservative (as indicated by a predicted low value for pressurized thermal shock (PTS)), or if there was indeed a need for a detailed evaluation. The staff agreed with the licensee's later approach for resolving the issue. The staff indicated that when the applicant completes the PTS evaluation at 54 EFPY, it will resolve the staff concerns conclusively.

Staff Question:

The staff had a concern regarding Tables 10 thru 14 of WCAP-15885, on the measured sensor specific activities and reaction rate computations pertaining to reactor vessel Capsules V, R, T and S. The issue was the lack of details for the radial adjustments assumptions that the licensee made.

Telecon Discussions:

During the discussion, the licensee explained that the radial adjustment was necessary to account for the distance factor from the center of the capsule (axial level at 158.11 radius) relative to the dosimeter location. The staff was satisfied with the licensee's explanations and there were no additional request on this issue.

RG&E has reviewed and did not have any comments on this telecon summary.

/***RA***

Ram Subbaratnam, Project Manager
License Renewal Section
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket No.: 50-244

Enclosure: As stated

cc w/encl: See next page

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Ram Subbaratnam, Project Manager
License Renewal Section
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket No.: 50-244

Enclosure: As stated

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DATE	05/14 /2003	05/14 /2003	05/14 /2003	05/16/2003

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DISTRIBUTION: Summary of Telecon with OPPD Re: Ginna RAIs, Dated: May 16, 2003

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Enclosure 1

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