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Fred Dacimo
Vice President, Operations

May 15, 2003
NL-03-082

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station O-P1-17
Washington, DC 20555-0001

Subject: Indian Point Unit No. 2
Docket No. 50-247
License No. DPR-26
Monthly Operating Report for April 2003

Dear Sir:

Enclosed is the Monthly Operating Report for Indian Point 2 for the month of April 2003 that is being submitted in accordance with Technical Specification 6.9.1.7.

Entergy is making no commitments in this letter. Should you have any questions regarding this submittal, please contact Mr. John McCann, Manager, Licensing, Indian Point Energy Center at (914) 734-5074.

Sincerely yours,

A handwritten signature in black ink, appearing to be "FD", with a horizontal line underneath.

Fred R. Dacimo
Vice President, Operations
Indian Point Energy Center

cc: see next page

IE24

Attachment

cc: Mr. Hubert J. Miller
Regional Administrator – Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406-1498

Resident Inspector
U.S. Nuclear Regulatory Commission
Indian Point 2
P.O. Box 38
Buchanan, NY 10511

INPO Records Center
700 Galleria Parkway
Atlanta, Georgia 30339-5957

Mr. Paul Eddy
State of New York Department of Public Service
3 Empire Plaza
Albany, NY 12223

OPERATING DATA REPORT

DOCKET NO. 50-247
DATE May 6, 2003
COMPLETED BY M. Walther
TELEPHONE (914)734-5728

OPERATING STATUS

1. Unit Name :	INDIAN POINT UNIT No. 2	Notes
2. Reporting Period :	April-2003	
3. Licensed Thermal Power (MWt) :	3071.4	
4. Nameplate Rating (Gross Mwe) :	1008	
5. Design Electrical Rating (Net Mwe) :	986	
6. Maximum Dependable Capacity (Gross Mwe) :	965*	
7. Maximum Dependable Capacity (Net Mwe) :	931*	

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report , Give Reasons :

* The above changes reflect summer ratings effective April 6, 2003 .

9. Power Level To Which Restricted , If Any (Net Mwe) : _____

10. Reasons For Restrictions , If Any : _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	719	2,879	252,744
12. Number Of Hours Reactor Was Critical	685.68	2,845.68	177,347.30
13. Reactor Reserve Shutdown Hours	0	0	4,566.64
14. Hours Generator On-Line	673.78	2,833.78	173,406.08
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	2,050,270	8,658,399	489,101,518
17. Gross Electrical Energy Generated (MWH)	674,651	2,854,572	152,932,928
18. Net Electrical Energy Generated (MWH)	651,080	2,761,489	146,540,961
19. Unit Service Factor	93.7	98.4	68.6
20. Unit Availability Factor	93.7	98.4	68.6
21. Unit Capacity Factor (Using MDC Net)	96.9	101.3	64.9
22. Unit Capacity Factor (Using DER Net)	91.8	97.3	62.7
23. Unit Forced Outage Rate	6.3	1.6	13.7

24. Shutdowns Scheduled Over Next 6 Months (Type , Date , and Duration Of Each) :

25. If Shut Down At End Of Report Period , Estimated Date Of Startup :

26. Units In Test Status (Prior to Commercial Operation) :	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
COMMERCIAL OPERATION	N/A	N/A

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-247

UNIT I.P. Unit #2

DATE May 6, 2003

COMPLETED BY M. Walther

TELEPHONE (914)734-5728

MONTH April-2003

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>980</u>
2	<u>981</u>
3	<u>978</u>
4	<u>979</u>
5	<u>979</u>
6	<u>979</u>
7	<u>980</u>
8	<u>979</u>
9	<u>980</u>
10	<u>980</u>
11	<u>979</u>
12	<u>979</u>
13	<u>980</u>
14	<u>980</u>
15	<u>979</u>
16	<u>978</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>979</u>
18	<u>979</u>
19	<u>978</u>
20	<u>980</u>
21	<u>978</u>
22	<u>978</u>
23	<u>980</u>
24	<u>979</u>
25	<u>978</u>
26	<u>981</u>
27	<u>982</u>
28	<u>681</u>
29	<u>0</u>
30	<u>75</u>
31	<u></u>

INSTRUCTIONS

On this format , list the average daily unit power level in Mwe-Net for each day in the reporting month. Compute to the nearest whole megawatt .

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-247
UNIT I.P. Unit #2
DATE May 6, 2003
COMPLETED BY M. Walther
TELEPHONE (914)734-5728

REPORT MONTH April-2003

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	030428	F	45.22	A	3	2003-003	EA	(RELAYX) X	Reactor trip due to turbine-generator trip as a result of a main generator over-frequency trip caused by loss of offsite load.

¹
F : Forced
S : Scheduled

²
Reason :
A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training & License Examination
F - Administrative
G - Operational Error (Explain)
H - Other (Explain)

³
Method :
1 - Manual
2 - Manual Scram
3 - Automatic Scram
4 - Other (Explain)

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

⁵
Exhibit I - Same Source

Summary Of Operating Experience

April 2003

Indian Point Unit No. 2 was synchronized to the bus for a total of 673.78 hours, producing a gross electrical generation of 674,651 MWH.

On April 28, 2003, at 1645 hours, an automatic reactor scram occurred due to a turbine-generator trip as a result of a loss of offsite load. The plant trip was initiated by a main generator over-frequency trip. All required safety systems functioned as designed. Fast bus transfer from supply by the unit auxiliary transformer to supply by the station auxiliary transformer was blocked per design by the over-frequency transfer interrupt circuit. The blocked power transfer de-energized four of six 6.9 Kv buses (1, 2, 3, and 4) and two of four associated 480 volt buses (2A and 3A). All four-reactor coolant pumps and 4 out of 6 circulating water pumps were de-energized due to the loss of the 6.9 Kv buses. Per plant design all three Emergency Diesel Generators (EDG) started but did not automatically load their assigned buses. EDG 22 was manually loaded to energize Buses 2A and 3A. 6.9 Kv buses 5 and 6 remained energized throughout the event. In response to the transient, Pressurizer PORV 455C lifted for a short duration and reseated with no leakage. Initially motor driven auxiliary feed water pump 21 did not start due to the de-energization of its assigned bus 3A. The plant was stabilized and in natural circulation. After a post trip assessment, the reactor was brought critical at 0204 hours on April 30, 2003, with unit synchronization at 1358 hours. Power ascension commenced with full power being achieved on May 1, 2003, at 1000 hours.