OMB No.: 3150-0011 Expiration Date: 9/30/86 IEB 85-01

# UNITED STATES NUCLEAR REGULATORY COMMISSION OFFICE OF INSPECTION AND ENFORCEMENT WASHINGTON, D.C. 20555

October 29, 1985

IE BULLETIN 85-01: STEAM BINDING OF AUXILIARY FEEDWATER PUMPS

#### Addressees:

For Action - Those nuclear power reactor facility licensees and construction permit (CP) holders listed in Attachment 1.

For Information - All other nuclear power reactor facilities.

#### Purpose:

The purpose of this bulletin is to inform licensees and CP holders of a potentially serious safety problem that has occurred at certain operating facilities involving the inoperability of auxiliary feedwater (AFW) pumps as a result of steam binding. Certain PWR licensees and all PWR CP holders are requested to take further action to prevent similar events from occurring at their facilities and to document those actions taken or planned.

### Description of Circumstances:

Numerous events have been reported where hot water has leaked into AFW systems and flashed to steam, disaling the AFW pumps. Events at Robinson 2 in 1981 through 1983, Crystal Riv., 3 in 1982 and 1983, and D. C. Cook 2 in 1981 were summarized in IE Information Notice (IN) 84-06, issued in January 1984. Also in January 1984, the Institute of Nuclear Power Operations (INPO) issued Significant Event Report (SER) 5-84 detailing events at Robinson 2 and Farley. In April 1984, INPO issued Significant Operating Experience Report (SOER) 84-3 that discussed another event at Surry 2 in 1983.

The NRC's Office for Analysis and Evaluation of Operational Data (AEOD) issued a case study report entitled "Steam Binding of Auxiliary Feedwater Pumps" in July 1984. This study identified 22 events since 1981; 13 of these occurring in 1983. Based on operating experience, it appears that backleakage into AFW could occur in any PWR. In a number of plants, the two motor-driven pumps feed into a single pipe which feeds into the steam generator; therefore, a leaking valve in that pipe increases the probability of steam binding in both trains of AFW. Also, multiple AFW pumps often take suction from a common manifold; therefore, if one pump becomes steam bound because of leaking check valves, the steam can heat the common suction and cause other pumps to become steam bound.

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AFW capability is needed for normal shutdown and transient conditions as well as for accident mitigation. The AEOD case study examined the effects of steam binding on a sequence in which there was a loss of the power conversion (steam generation) system after a transient other than loss-of-offsite power. A probabilistic risk analysis had previously shown this sequence to be a dominant contributor to the core-melt risk for a sample plant (Sequoyah). The case study indicated that unavailability of the AFW system as a result of steam binding contributes significantly to the risk of core melt in PWRs. Monitoring AFW to detect backleakage and to promptly correct the situation if backleakage occurs would reduce the probability of steam binding.

Since the AEOD study was issued, a series of events involving backflow of hot water into the AFW system occurred at McGuire 2 over a period of 7 days in August 1984, before effective corrective action was taken. One of these events involved overpressurization of the suction line and damage to instruments. In November 1984, Catawba 1 experienced backflow of hot water into AFW resulting in fumes from insulation and blistering of paint. In December 1984, the NRC's Office of Nuclear Reactor Regulation (NRR) determined that steam binding of AFW was a generic issue and assigned it a high priority (Generic Issue 93, "Steam Binding of Auxiliary Feedwater Pumps").

To determine the extent of the safety issue and the need for short-term corrective actions, the NRC's regional offices conducted a survey in April and May of 1985. Of the 58 operating reactors surveyed, 39 had temperature monitoring of AFW piping at least once per shift. Of the remaining 19, 17 had normally closed gate or globe valves in the pump discharge path in addition to check valves, or some unique feature such as complete separation of trains that made serious safety problems unlikely. The remaining 2 licensees have subsequently decided to monitor AFW piping temperature.

Although some degree of action has been taken at all units, many have not incorporated these actions into procedures to detect or correct steam binding. Without these provisions, there is little assurance that effective actions will continue. For this reason, the addressees are requested to take the following actions:

# Action for Addressees Listed in Attachment 1

1. Develop procedures for monitoring fluid conditions within the AFW system on a regular basis during times when the system is required to be operable. This monitoring should ensure that fluid temperature at the AFW pump discharge is maintained at about ambient temperature. Monitoring of fluid conditions, if used as the primary basis for precluding steam binding, is recommended each shift.

This item is not intended to require elaborate instrumentation. A simple means of monitoring temperature, such as touching the pipe, is a satisfactory approach.

2. Develop procedures for recognizing steam binding and for restoring the AFW system to operable status, should steam binding occur.

 Procedural controls should remain in effect until completion of hardware modification to substantially reduce the likelihood of steam binding or until superseded by action implemented as a result of resolution of Generic Issue 93.

Schedule: For operating plants, develop and implement procedures within 90 days of the date of this bulletin. For plants under construction, develop and implement procedures within 90 days after receiving an operating license or provide an appropriate response and commitment within 1 year of the date of this bulletin, whichever comes first.

Reporting Requirements: Prepare and submit a report describing the methods used to accomplish these actions. Include the date(s) that procedures and training were implemented or scheduled to be implemented. State the frequency of monitoring of the temperature. For operating plants, submit this report within 120 days of the date of this bulletin. For plants under construction, submit the report within 120 days after receiving an operating license or within 1 year of the date of this bulletin, whichever comes first. It is not necessary to submit the procedures for review.

The written report, requested above, shall be submitted to the appropriate Regional Administrator under oath or affirmation under provisions of Section 182a, Atomic Energy Act of 1954, as amended. In addition, the original copy of the cover letter and a copy of the attached report should be transmitted to the U. S. Nuclear Regulatory Commission, Document Control Desk, Washington, DC 20555 for reproduction and distribution.

This request for information was approved by the Office of Management and Budget under blanket clearance number 3150-0011. Comment on burden and duplication should be directed to the Office of Management and Budget, Reports Management, Room 3208. New Executive Office Building, Washington, DC 20503.

Although no specific request or requirement is intended, the following information would be helpful to the NRC in evaluating the cost of this bulletin:

- 1. staff time to perform requested review and testing
- 2. staff time to prepare requested documentation

IEB 85-01 October 29, 1985 Page 4 of 4

If there are any questions regarding this matter, please contact the Regional Administrator of the appropriate NRC regional office or this office.

James M. Taylor, Birector

Office of Inspection and Enforcement

Technical Contacts: Mary S. Wegner, IE (301) 492-4511

C. Vernon Hodge, IE (301) 492-7275

Attachments:

1. Addressees for Action

2. List of Recently Issued IE Bulletins

1EB 85-01 October 29, 1985 Page 4 of 4

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151

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\*See previous concurrences

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\*DEPER:IE WFisher 8/5/85 If there are any questions regarding this matter, please contact the Regional Administrator of the appropriate NRC regional office or this office.

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IEB 85-XX October , 1985 Page 4 of 4

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Attachment 1 IEB 85-01 October 29, 1985 Page 1 of 1

# ADDRESSEES FOR ACTION:

The following PWRs having an OL:

BYRON 1 CRYSTAL RIVER 3 DAVIS-BESSE DIABLO CANYON 1 GINNA INDIAN POINT 3 KEWAUNEE' MAINE YANKEE MILLSTONE 2 NORTH ANNA 1 NORTH ANNA 2 PALISADES POINT BEACH 1 POINT BEACH 2 PRAIRIE ISLAND 1 PRAIRIE ISLAND 2 RANCHO SECO SAN ONOFRE 1 ST. LUCIE 1 ST. LUCIE 2 SUMMER TMI 1 **TROJAN** TURKEY POINT 3 TURKEY POINT 4 WATERFORD 3 WOLF CREEK 1 YANKEE-ROWE

All PWRs holding a CP

## LIST OF RECENTLY ISSUED IE BULLETINS

Bulletin No.	Subject	Date of Issue	Issued to
84-03	Refueling Cavity Water Seal	8/24/84	All power reactor facilities holding an OL or CP except Fort St. Vrain
84-02	Failures Of General Electric Type HFA Relays In Use In Class 1E Safety System	3/12/84	All power reactor facilities holding an OL or CP
84-01	Cracks In Boiling Water Reactor Mark I Containment Vent Headers	2/3/84	All BWR facilities with Mark I contain-ment and currently in cold shutdown with an OL for Action and All other BWRs with an OL or CP for information
83-08	Electrical Circuit Breakers With An Undervoltage Trip Feature In Use In Safety- Related Applications Other Than The Reactor Trip System	12/28/83	All power reactor facilities holding an OL or CP
83-07 Sup. 2	Apparently Fraudulent Products Sold By Ray Miller, Inc.	12/09/83	Same as IEB 83-07
83-07 Sup. 1	Apparently Fraudulent Products Sold By Ray Miller, Inc.	10/26/83	Same as IEB 83-07
83-07	Apparently Fraudulent Products Sold by Ray Miller, Inc.	7/22/83	All power reactor facilities holding an OL or CP; Other fuel cycle facilities and Category B, Priority I (processor and distributors) material licensees

OL = Operating License CP = Construction Permit