



## 5. TECHNICAL ISSUES

### 5.1 Aesthetic and Scenic Resources

#### 05.01 (001) Aesthetic and Scenic Resources

##### COMMENT

The commentor states that the EIS ignores the presence of unusually aesthetically p particularly the buttes, on and adjacent to the Idaho National Engineering Laborato notes that the Middle Butte and other sites on the Idaho National Engineering Labor the Shoshone-Bannock Tribes. The commentor also states that visual impacts should based on what could be seen from the Idaho National Engineering Laboratory boundary the EIS should also analyze visual impacts for tribal members who have been granted access to the site.

##### RESPONSE

Volume 2, section 4.2 identifies that portion of the Idaho National Engineering Lab the Big Butte Resource Area, which is administered by the Bureau of Land Management section 4.5 has been revised to acknowledge that features of the natural landscape to the Shoshone-Bannock Tribes.

Volume 2, section 5.5 discusses the impacts of proposed projects on aesthetic and s for the various EIS alternatives. Most of the proposed projects would be confined areas and be in size and shape to adjacent structures. The locations of some new f determined for the Ten-Year Plan alternative; however, such facilities probably wou existing facilities and at least 1/2 mile from public roads. Although no final sit expected to occur on or near the buttes, the final siting determination will consid aesthetically pleasing landforms.

Volume 2, section 5.4 has been revised to state that the Shoshone-Bannock Tribes wo before any project is developed that could impact resources of importance to the Tr

#### 05.01 (002) Aesthetic and Scenic Resources

##### COMMENT

The commentor states that impacts to visibility and enjoyment of view at the Fort H as effects on tourism, are not considered in the EIS.

##### RESPONSE

The Fort Hall Reservation is approximately 27 miles southeast of the southern bound Although a specific analysis was not performed for the Fort Hall Reservation, the a the EIS concluded that the potential for impairment of the visual resource at Crate Monument, which is approximately 12 miles west of INEL's western boundary, could no The analysis used very conservative methods, including assumptions that many of th sources of emissions would not incorporate emissions controls, and that pristine co at Craters of the Moon. However, DOE would not be able to obtain an air permit for sources unless it could be shown to the satisfaction of the Idaho Division of Envir there would be no perceptible impacts on visibility at the Craters of the Moon Nati is the nearest Class I area to INEL. The control measures that would be required t Craters of the Moon would also serve to prevent impairment of visibility or enjoyme Fort Hall Reservation.

In addition, the Fort Hall Reservation lies outside the path of prevailing winds fl site. As noted in Volume 1, Appendix B, section 4.7, the mountain ranges bordering channel the prevailing westerlies into a southwest wind, away from the reservation.

#### 05.01 (003) Aesthetic and Scenic Resources

COMMENT

Commentors urge that the beauty of Idaho be preserved.

RESPONSE

DOE agrees. In developing the alternatives for management of spent nuclear fuel (S restoration and waste management at INEL, DOE was sensitive to the impacts that could disturbance of the natural landscape. Thus, for new facilities, DOE would use land disturbed or land that is adjacent to developed land. The amount of land required also be minimized. Even for the case in which all SNF would be shipped to INEL, on percent of the site land area) would be devoted to new facilities. In developing efforts to prevent degradation of views and prevent environmental damage that might natural flora and fauna.

## 5.2 Air Quality

### 05.02 (001) Air Quality

COMMENT

The commentor wonders about the effects on air quality of releases of polluting chemicals materials to the air.

RESPONSE

DOE's policy is to comply with all applicable Federal, state, and local regulations protect human health and the environment. Where possible, potential concentrations the various alternatives have been estimated, considering appropriate local meteorological each site. DOE employs pollution reduction techniques to minimize air releases when constructing, and operating facilities.

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environmental impacts including impacts to air quality, for all the alternatives considered in this EIS. Impacts for all alternatives would be small.

### 05.02 (003) Air Quality

COMMENT

The commentor states that the radiological risks of the various alternatives in the and are consistent with other studies that have concluded that the risks of handling defense high-level waste or commercial fuel are not large.

RESPONSE

The comment is consistent with the EIS, which shows that the radiological risks for alternatives would be low, including the risks of interim storage of high-level waste. The risks of handling commercial SNF, with the exception of certain special-case fuels are beyond the scope of this EIS.

### 05.02 (004) Air Quality

COMMENT

The commentor questions the appropriateness of the units of measure (picocuries per Volume 1, Appendix C, Table 4-18 to describe tritium activity in air moisture.

RESPONSE

The title of Table 4-18 has been revised to "Tritium measured in air at the Savanna more clearly reflect that a volume of air rather than water (or precipitation) was

### 05.02 (005) Air Quality

COMMENT

The commentor indicates that the Hanford Site is in a noncompliant area for particulate

**RESPONSE**

The commentor is correct. According to Volume 1, Appendix A, Table 4.7-2, the maximum average particulate concentration exceeds State of Washington standards. The EIS has reflected this fact.

**II 05.02 (006) Air Quality****COMMENT**

The commentor states that a definition of 95 percent meteorology should be provided in 5.14 or Appendix F-5. The commentor also notes that the definition given in Volume section 5.3 is incorrect and should be replaced.

**RESPONSE**

The commentor is correct. The following definition of 95 percent meteorology has been added to 5.14 and has replaced the incorrect definition in Volume 2, Appendix F: meteorology is defined as stable weather conditions, unfavorable to atmospheric contaminants, which are not exceeded more than 5 percent of the time."

**II 05.02 (007) Air Quality****COMMENT**

The commentor cannot tell from the EIS analysis if susceptible populations, such as homes, have been considered, or whether pollutant deposition on local food crops has been

**RESPONSE**

DOE can determine no cases where susceptible subgroups, such as nursing home occupants, require specific evaluation. The basis for this statement is (1) air quality impacts at all sites are well below health-based standards for all pollutants considered, and (2) the impacts are based on dose-response data, which have already accounted for susceptible subgroups. Pollutant deposition on local food crops has been directly assessed in the case of criteria pollutants and indirectly assessed in the case of criteria pollutants. In the latter case, all of the pollutants are below the secondary air quality standards, which have been established to protect effects to vegetation, property, or other elements of the environment.

DOE has added a better explanation of source terms and a description of the direct and secondary pathways that were evaluated and included in the EIS. (See Volumes 1

**II 05.02 (008) Air Quality****COMMENT**

The commentor considers the EIS presumptuous to claim that levels of all nonradioactive pollutants, with the possible exception of hydrochloric acid, which results from the incineration of mixed low-level waste, are below applicable standards. The commentor states that to confirm this, and it is impossible to be so positive about any proposed incinerator, asks if this incinerator is being evaluated under the Environmental Protection Agency Strategy."

**RESPONSE**

With respect to hydrochloric acid, the incinerator in question is the Waste Experimental Facility. This facility is included in Volume 2 for the Ten-Year Plan and Maximum and Disposal alternatives for processing low-level and mixed low-level waste. However, it is not a "proposed incinerator," but rather an existing facility that has had several years of low-level waste and limited amounts of mixed wastes. Thus, a considerable amount of operating experience exists. The Waste Experimental Reduction Facility has air quality specific limitations for various pollutants. The facility can continue to operate and the reviewing agency will evaluate all data under applicable standards and guidelines, Environmental Protection Agency's (EPA's) new "Combustion Strategy," and will apply restrictions and emissions standards designed to ensure compliance.

Other incinerators proposed under these alternatives (e.g., the Idaho Waste Process Low-Level Waste Treatment Facility, and the private-sector Alpha-Mixed Low-level Waste Facility) are early in the conceptual design stage of development, and the projects are uncertain. Annual average increment levels, exclusive of baseline levels, should be

promulgated State of Idaho standards for noncarcinogenic toxics, including hydrochl analyses presented in the EIS used maximum 8-hour concentrations in accordance with Idaho guidelines. Due to the conservative approach used in these analyses, and the conditions that will be applied by the State of Idaho Division of Environmental Qua review function, DOE can state with confidence that all pollutant levels would be w standards.

## II 05.02 (009) Air Quality

### COMMENT

The commentor objects to any promise of adding combustion controls to mitigate impa cites the case in which DOE received a permit for nitrogen oxide emissions from the Processing Plant in 1989, and although the permit contained a requirement to instal for those emissions, the equipment has yet to be installed.

### RESPONSE

The activity in question was the Fuel Processing Restoration (FPR) Project. The pe operation of the FPR project and was not independently applicable. The FPR project the increases in nitrogen oxide emissions did not materialize. With regard to this promise to add combustion controls to mitigate impacts. Rather, each new project w determine whether controls are required or warranted. In some cases, combustion co required by the State of Idaho Division of Environmental Quality before a facility construction permit.

## II 05.02 (010) Air Quality

### COMMENT

The commentor states that Idaho air quality rules should be specified as "Rules for Pollution in Idaho," and references to the Air Quality Bureau should be updated.

### RESPONSE

The commentor is correct. References to Idaho air quality rules and the Air Qualit updated in Volume 2.

## II 05.02 (011) Air Quality

### COMMENT

The commentor states that ambient air concentrations at the Idaho National Engineer be modeled at the inner boundary of the grazing area on the site, because the publi that area.

### RESPONSE

As defined in Rules for the Control of Air Pollution in Idaho, "ambient air" refers atmosphere to which the general public has access. This is not the case with grazi site. Access to these areas is controlled and is restricted to certain individuals does not have access. DOE's position is that these grazing areas do not meet the contain "ambient air." Therefore, ambient air quality standards do not apply, and required for these areas.

## II 05.02 (012) Air Quality

### COMMENT

The commentor asked DOE to explain why the latest version of the SCREEN air quality SCREEN2) was not used.

### RESPONSE

The EIS used air quality baseline data for some toxic air pollutants that had been modeling efforts, which used the SCREEN model. Rather than repeat these analyses u approach was taken whereby: (a) for any screening level, baseline toxic results th were run to



determine if there were significant differences in the results obtained using SCREEN versus SCREEN2. For the manner in which the SCRE applied, test runs indicated that no difference would be obtained by reassessing th previously been performed. There is no requirement in Rules for the Control of Air perform the analyses that were done using SCREEN. The analyses to determine compli increment standards were performed using ISCST-2.

## II 05.02 (013) Air Quality

### COMMENT

The commentor disagrees with the statement about krypton-85 being "by far, the radi highest emission rate." The commentor also states that since reprocessing has been the radionuclide with the highest emission rate.

### RESPONSE

The statement cited by the commentor is from Volume 1, Appendix B. Volume 2 makes krypton-85 has historically been the radionuclide with the highest emission rate, b the activity primarily responsible for krypton-85 emissions, ceased in 1992. The w Appendix B, section 4.7 has been changed to correspond to that in Volume 2, section

## II 05.02 (014) Air Quality

### COMMENT

The commentor questions why Volume 1, Table 4.7-3 and Volume 2, Table 4.7-1 list no from Argonne National Laboratory-West that are higher than those listed in the 1991 Management Information System and the 1991 Idaho National Engineering Laboratory Na Emission Standard for Hazardous Air Pollutants, Annual Report.

### RESPONSE

As indicated in footnotes on the tables cited by the commentor, the emissions estim existing facilities and reasonably foreseeable increases to the baseline. Included Fuel Cycle Facility at Argonne National Laboratory-West. This facility has signifi krypton-85 (11,500 curies) and xenon-131m (127 curies), which account for the diffe values listed in the tables and the values reported in the Radioactive Waste Manage System and 1991 Idaho National Engineering Laboratory National Emission Standard fo Air Pollutants, Annual Report.

## II 05.02 (015) Air Quality

### COMMENT

The commentor states that emissions and visibility impacts should be evaluated for equipment associated with plant services that would be needed to support the Region Centralization alternatives at Idaho National Engineering Laboratory.

### RESPONSE

The specific projects associated with the alternatives for Regionalization or Centr would not require additional fossil-fuel-burning equipment beyond that which is alr exception of one minor source, a diesel generator associated with the Fort St. Vrai Project. The emissions from this source would be very low, and the statement that not add a measurable increment to emissions at INEL is accurate. Visibility impact would be small. A visibility impact analysis was also performed for the closest Cl Moon National Monument) for the cumulative emissions of all applicable sources comp 2 alternative.

## II 05.02 (016) Air Quality

### COMMENT

The commentor states that mercury is shown to slightly exceed the State of Idaho cr alternatives. The commentor states that given the uncertainty known to exist in th

Complex model, it is not possible to judge the health implications of this informat  
RESPONSE

The mercury levels reported in the Draft EIS are the maximum 8-hour levels that would be expected. The EIS reflects State of Idaho standards effective May 1, 1994, for calculating the effect on air quality. The State of Idaho now requires that state annual average levels be discussed in Volume 2, section 5.7, revised calculations show that mercury levels are less than the Idaho standard. The revised mercury level is less than 3 percent of the state standard. The levels are predicted if mercury-bearing waste were processed at a very high rate. engineering controls could be employed to minimize and ensure that levels approaching the standard would not result.

The EIS has been changed from an 8-hour reporting level to a 24-hour reporting level.

## II 05.02 (018) Air Quality

### COMMENT

The commentor makes the following recommendations: (a) use the same baseline year for criteria pollutants, and toxic air pollutants, (b) clarify the distinction between projected emissions for some cases, and (c) present air emissions for 1990, 1991, and 1993 for each of these years.

### RESPONSE

The rationale for using different baseline years for radionuclides, criteria pollutants, and criteria pollutants follows: Generally, the most representative baseline year is the most recent year at the time the analyses were performed, the availability of data varied for the three pollutants. For radionuclides and criteria pollutants, 1991 was the most recent year available when the baseline studies were conducted, and these were the data that were used for SNF processing took place that year at the Idaho Chemical Processing Facility. longer performed at this facility and radionuclide emissions for this activity are of baseline conditions. Moreover, processing is an activity assessed in association with the alternatives, and inclusion of these emissions in both the baseline and alternative cause double counting. That is why the 1993 radionuclide emissions were used for the baseline. For toxic air pollutant emissions, only 1989 data were available and currently are available. The only distinction made between existing emissions and a future baseline involves specific projects that are expected to become operational before June 1, 1995 (that period covered by the EIS alternatives). These projects are identified in Volume 2 Appendix F-3. The analysis is conservative in that no credit is taken for future reductions. DOE does not agree that 3 years of emissions should be analyzed. Conservative emissions were used for the baseline year, and all impacts based on these estimates represent an upper bound of impacts that would actually occur. For example, the maximum emissions scenario for toxic air pollutants exceeds actual emissions by a substantial margin (as illustrated in 4.7-4) and bounds the baseline conditions.

## II 05.02 (019) Air Quality

### COMMENT

The commentor states that the only Air Quality Related Value considered was visibility. Justification was given for not including other Air Quality Related Values, such as particulate emissions from plants.

### RESPONSE

Air Quality Related Values other than visibility were assessed. Volume 2, section 5.7, discusses impacts to soils and vegetation and impacts due to secondary growth. All off-site concentrations of pollutants are below the secondary air quality standards, which have been established to protect effects on vegetation, property, or other elements of the environment. Standards for particulate emissions have also been established for fluorides, although impacts of fluoride emissions are only for comparison to the Toxic Air Pollutant Increments. Fluoride emissions associated with the alternatives would be very low and would not be expected to result in any impact. Deposition on local food crops has been directly assessed for radionuclides; the risk from ingestion of contaminated food products. With respect to other Air Quality Related Values, evaluations were performed and described for ozone formation, stratospheric ozone depletion, deposition, and global warming.

## II 05.02 (020) Air Quality

### COMMENT

The commentor points out that the statement "emissions of volatile organic compound to have a negligible effect on ozone formation" is incorrect. The commentor states inventory indicates emissions of more than 600 tons per year of volatile organic compound. The commentor recommends that the amount of ozone formation be estimated.

### RESPONSE

The 1990 emissions inventory for INEL quantifies the maximum potential emissions of compounds (VOCs) as more than 600 tons per year. VOC emissions from actual operations are 100 tons per year. VOC emissions from the proposed projects would be less than 10 tons per year, which is less than the applicable State of Idaho standards' significant level of 40 tons per year that would be used in the formation analysis. From Volume 2, Table 5.7-2 it can be seen that volatile organic compound emissions range from 5,583 kilograms (6.1 tons) per year for the No Action alternative to 8,800 kilograms (9.7 tons) per year for the Maximum Treatment, Storage, and Disposal alternative. The low potential for ozone formation from the proposed projects precludes the need for a detailed assessment. Therefore, requiring air quality permits, analyses for impacts resulting from specific pollutants would be performed, contingent on regulatory requirements.

## II 05.02 (021) Air Quality

### COMMENT

The commentor states that releases of carbon tetrachloride, freon, and greenhouse gases are extremely small compared with global loading, and considers this an unreasonable concern.

The statement in question attempts to characterize emissions associated with the alternatives. The potential for stratospheric ozone depletion (carbon tetrachloride and freon) and global warming (greenhouse gases, including carbon dioxide, methane, nitrous oxides, and chlorofluorocarbons) are global (not regional) effects, which are associated with global emissions. The alternatives represent an extremely small fraction of global levels, and it is reasonable to expect these emissions would have small impacts with respect to global effects. INEL has reduced or eliminated the use of chlorofluorocarbon compounds.

## II 05.02 (022) Air Quality

### COMMENT

The commentor requests that DOE demonstrate how the emission rates and concentrations of pollutants summarized in Volume 2, section 4.7 were calculated.

### RESPONSE

The methods used to calculate emission rates and concentrations are described in Volume 2, Resources, which is referenced in Appendix F-3. For radiological releases and assessments, details are provided in Estimated Radiological Doses Resulting from Airborne Radionuclides at the Idaho National Engineering Laboratory, and Maximum Individual, Critical Group, and Population Doses from INEL Proposed Action and No Action Sources, which are also referenced in Appendix F-3. The referenced reports are available for review in the reading rooms at the locations listed in the EIS.

## II 05.02 (023) Air Quality

### COMMENT

The commentor points out that previous documents have established that adequate upper atmosphere (upper height) data are not available for the Idaho National Engineering Laboratory vicinity. The commentor requests that the upper air meteorological data used for modeling be provided.

### RESPONSE

Verified measurements of on-site mixing height for the INEL vicinity are not available. However, original nonradiological analyses (modeling of the baseline concentrations and impacts) were performed using the upper atmosphere data available for the INEL vicinity.

conservatively assumed a mixing height of 100 meters for modeling of both short- (an average) concentrations. The radiological modeling (which only involves annual average height of 800 meters. Additional nonradiological modeling, which has since been per compliance with Prevention of Significant Deterioration (PSD) increment limits, use for 3-hour and 24-hour averaging periods, and 800 meters for annual average assessment considered more reasonable estimates for short- and long-term mixing heights. The value is that 150 meters is reportedly the lowest mixing height ever observed at IN Handbook, Page 4-48). The 800-meter value is recommended by the National Oceanic and Administration as appropriate for long-term modeling (Sangendorf, J., U.S. Department of National Oceanic and Atmospheric Administration, Averaging INEL Mixing Depths, Memo EG&G-Idaho, Inc., February 11, 1991). For short-term calculations, the same result 100 or 150 meters is used; this is because the highest short-term concentrations are during conditions of slight-to-moderate atmospheric stability (that is, stability cases mixing height data are not used by the Industrial Source Complex Short Term-2

## II 05.02 (024) Air Quality

### COMMENT

The commentor points out that the toxic standards are now listed as increments and Review Toxic Policy was eliminated.

### RESPONSE

Volume 2, Figure 4.7-2 has been revised to reflect recent updates to the Idaho Toxic Standards. The New Source Review Toxic Policy was incorporated into the Rules for Pollution in Idaho.

## II 05.02 (025) Air Quality

### COMMENT

The commentor points out that the power of 10 is missing in the value of foreseeable tetrachloride emissions in Volume 2, Table 4.7-2.

### RESPONSE

Volume 2, Table 4.7-2 has been corrected to show the value for foreseeable increase tetrachloride emissions as  $4.5 \times 10^{-5}$  kilograms per year.

## II 05.02 (026) Air Quality

### COMMENT

The commentor notes that the correct characterization for the area around the Idaho Laboratory site is "in attainment or unclassified" for all National Ambient Air Quality Standards.

### RESPONSE

The commentor is correct. Volume 2, section 4.7.4 has been revised to read: "The National Engineering Laboratory site is in attainment or unclassified for all National Standards."

## II 05.02 (027) Air Quality

### COMMENT

The commentor states that the estimated impacts on air quality, especially on visibility presented for operation of the New Waste Calcining Facility and questions whether they are included in Volume 2, Figure 5.7-4. The commentor states that NO<sub>x</sub> reduction in Calcining Facility plume should be evaluated.

### RESPONSE

The impacts on air quality have been assessed for emissions associated with the New Facility. These impact assessments included comparison with ambient air quality standards include potential impacts on visibility. Visibility impacts were indirectly assessed visual range used for the visibility analysis of alternative projects reflects conditions

Waste Calcining Facility was operating. Volume 2, section 5.7 discusses impacts to Immobilization Facility, which would eventually replace the New Waste Calcining Facility (has similar projected NOx emissions), has been evaluated for visibility impacts. The requirement to evaluate the New Waste Calcining Facility for NOx reduction. Visibility evaluated in conjunction with obtaining necessary permits.

## II 05.02 (028) Air Quality

### COMMENT

The commentor notes that: (a) the discussion of cumulative effects of airborne emissions. National Engineering Laboratory omits discussion of visibility impacts and does not evaluate effects of exposure to multiple pollutants or long-term dose or risk from historic operational accident scenarios do not seem reasonable.

### RESPONSE

Visibility impacts from airborne emissions are discussed in the Volume 2, section 5.15.8. Impacts assessed for the alternatives are cumulative because the analysis determines the potential visual resource over the existing background, which is representative of conditions without emissions.

Potential synergistic effects from multiple chemical exposures are extremely difficult to quantify quantitatively because there is insufficient data to indicate synergistic effects. The synergistic effects are small where the concentrations for each individual compound for the alternatives evaluated in this EIS. To ensure that potential impacts are based on high releases and exposure conditions were assumed. Further, the point of highest chemical release occurs at different times and places. It is unlikely that any one individual more than one chemical species at the concentrations reported in this EIS.

Radiation doses from historic operations are discussed in Volume 2, section 5.15.8. Data available in referenced technical support documents, which are available for review in the rooms that received copies of this EIS. DOE is not aware of any generally accepted method that has been developed to evaluate synergistic effects due to several airborne chemicals. DOE is aware that research into this area is continuing.

The evaluation of cumulative effects considers historic accidents only. The implementation of the National Environmental Policy Act (NEPA) at 40 CFR Paragraph 1508.7 specifies "impacts result from past, present, and reasonably foreseeable future actions..." DOE has consistently interpreted "reasonably foreseeable" to include construction, maintenance, and other planned activities, but not to include future hypothetical accidents, spills, and other unplanned activities. Potential chemical exposure resulting from operations is discussed in Volume 2, Appendix F-5.

## II 05.02 (029) Air Quality

### COMMENT

Referring to Volume 2, section 5.18.2, the commentor points out that application of the methods is not a mitigation measure.

### RESPONSE

The commentor is correct. While the information derived from the application of the methods may eliminate the need for mitigation measures, the process is technically a mitigation measure. The sentence in question has been revised. It clarifies that measures to avoid, minimize, and compensate for impacts are based on the results of refined modeling confirm the findings of the screening-level analysis; the area of Craters of the Moon would be perceptibly impaired as a result of projected impacts.

## II 05.02 (030) Air Quality

### COMMENT

The commentor points out that the key word "net" is missing from the description of Significant Deterioration analysis must be performed.

### RESPONSE

Volume 2, Appendix F, section F-3.3.1 has been revised to clarify that a Prevention of Significant Deterioration (PSD) review is required whenever any modification would result in a net increase in emissions.

of any air pollutant.

## II 05.02 (031) Air Quality

### COMMENT

The commentor states that trace elements such as nickel may also be emitted by combustion generators and boilers) associated with the Pit 9 waste retrieval project.

### RESPONSE

At the time the Draft EIS was prepared, no generators or boilers were proposed for Project. Since that time, however, the project has been expanded to include two boilers. Modeling now includes the projected emissions from these boilers, which include the nickel, lead, and chromium. Emissions tables and dispersion modeling results in the EIS are updated.

## II 05.02 (032) Air Quality

### COMMENT

The commentor notes that radiological assessment methodology for air impacts treats results as constants with no uncertainty or variability, which is not consistent with environmental risk assessment. The commentor recommends that confidence statements be included in the estimates of the true, but unknown, value being calculated or the true, but unknown, value.

### RESPONSE

The radiological assessment of air impacts used the GENII code to perform calculations. Results represent best estimates for dose to an off-site individual, on-site individual, and population. They are based on conservative release estimates, representative meteorological conditions, and conservative assumptions regarding the location and habits of the receptors (especially exposed off-site individual). The dispersion model algorithms are generally accepted for this type of assessment (as opposed to research applications, in which a quantitative uncertainty analysis would be appropriate), and the computer code has been benchmarked as defined by the Interagency Agreement. It can be said with confidence that the dose results, especially those for the exposed off-site individual, overstate the doses that would actually occur, yet they are below the most restrictive limit. Using a computer code that has been extensively validated for quality assurance requirements is considered sufficient for an assessment of this type.

## II 05.02 (033) Air Quality

### COMMENT

The commentor recommends that the EIS clarify that a segment of past meteorological data has been chosen for the radiological assessments to be representative of average conditions over the 10-year period covered by the EIS.

### RESPONSE

Volume 2, Appendix F-3.4.2 states that the meteorological data used for the radiological assessments were obtained at the various facility monitoring stations over the 5-year period 1987 through 1991. It was not explicitly stated that these conditions are assumed to be representative of average conditions over the 10-year period covered by the EIS. Volume 2, Appendix F-3.4.2 has been revised to clarify this assumption.

## II 05.02 (034) Air Quality

### COMMENT

The commentor states that when comparing predicted concentrations of toxic air pollutants with the annual average standards contained in the May 1, 1994, Idaho rules, the concentrations are compared to annual averages.

### RESPONSE

The analyses in Volume 2, sections 4.7 and 5.7 compare predicted 8-hour concentrations of toxic air pollutants with the annual average standards. The analyses for noncarcinogenic emissions have been revised to compare predicted annual average concentrations.

## II 05.02 (035) Air Quality

### COMMENT

The commentor questions the basis for  $1.0 \times 10^{+04}$  curies of noble gases from the Idaho Processing Plant listed in Volume 1, Appendix B, Table 4.7-3, and Volume 2, Table 4.7-3.

**RESPONSE**  
The value of  $1.0 \times 10^{+4}$  curies represents an upper bound to the annual emissions of Idaho Chemical Processing Plant for a recent 1-year period. The actual releases for 1994 are classified. Actual baseline krypton-85 emissions from this facility are very much less than the value of  $1.0 \times 10^{+4}$  curies was used in the radiological dose assessment. Because the release of krypton-85 at these levels is not a large fraction of the overall dose, this release was used for evaluation and comparison of alternatives required for a programmatic EIS.

## II 05.02 (036) Air Quality

### COMMENT

The commentor points out that Volume 1, Appendix B, Table 5.7-1 lists ammonium hydrofluoride as toxic air pollutants (carcinogens), yet these substances are not listed in the Air Pollutants Increments.

### RESPONSE

The commentor is correct. Ammonium hydroxide and hydrofluoric acid are not carcinogens listed in Idaho's Toxic Air Pollutants Increments. Hydrofluoric acid emissions were not included because total fluoride emissions are listed in Idaho's Toxic Air Pollutants Increments. Ammonium hydroxide emissions were assessed conservatively as ammonia, a substance that is listed in the Air Pollutants Increments. DOE has clarified that these pollutants are not carcinogens in the inclusion (as stated above) in the EIS.

## II 05.02 (037) Air Quality

### COMMENT

The commentor states that current emissions and projected increases should be listed in Volume 2, and the basis for projected increases in baseline emissions should be explained.

### RESPONSE

The comment concerns the listing of radionuclide emissions for potential projects. The commentor considers reasonably foreseeable increases to the baseline. These increases are currently listed in Volume 2, Table 4.7-1, but are not listed separately. They are listed separately in the Support Document for Air Resources, which is included as a reference for Volume 2. The emissions for these projects were estimated in the same manner as described for alternative projects in Appendix F-3.4.1.

## II 05.02 (038) Air Quality

### COMMENT

The commentor states that analyses of air impacts should be compared with Prevention of Significant Deterioration limits, which are typically two to four times more stringent than National Ambient Air Quality Standards. The commentor points out that the Idaho National Engineering Laboratory triggered the Prevention of Significant Deterioration baseline dates for nitrogen oxides and particulates and that the baseline conditions in Volume 2, section 4.7 are not Prevention of Significant Deterioration baseline conditions.

### RESPONSE

The baseline date for a criteria pollutant establishes the date to start tracking changes. Additional analyses have been performed to characterize the existing baseline conditions for the alternatives in terms of the amount of PSD increment consumed. The methodology used with the Idaho Division of Environmental Quality, and a report documenting the methodology, has been completed and included as a reference in Volume 2. The results indicate that

conditions are within allowable increment consumption limits. When the contribution of the alternatives are added, the amount of increment consumption remains below the amount for each of the alternatives. The PSD baseline analysis have been incorporated into Volume 2, Appendix F-3 has been revised to reflect the methods used to calculate PSD consumption.

## II 05.02 (039) Air Quality

### COMMENT

Commentors state that DOE should analyze the existing and potential air quality impacts on the Fort Hall Reservation using all wind roses that indicate possible contributions from the Idaho National Laboratory site.

### RESPONSE

The air quality analyses in the EIS were based on meteorological data appropriate to the site at INEL. The analyses used the hourly meteorological data obtained from three on-site stations for 1991 and 1992 and are graphically presented as wind roses in Volume 2, sections 4.7 and 5.7. Stations are in the southeast, central, and northern sections of INEL. Similar analyses were performed at each facility. Maximum emissions concentrations from each facility were summed at various locations to determine the maximum baseline air quality impacts from present operations and cumulative impacts from proposed actions. Additional analyses were performed to evaluate impacts at points beyond the site boundary were less than those at the boundary (such as impacts from a tall stack were located in close proximity to the boundary). Similar analyses to determine the air quality impacts to various locations on the Fort Hall Reservation can be found in Volume 2, sections 4.7 and 5.7. Impacts to the Fort Hall Reservation can be found in Volume 2, sections 4.7 and 5.7. Impacts would be small for the alternatives considered in this EIS.

## II 05.02 (040) Air Quality

### COMMENT

The commentor states that the Tribes object to any attempt to locate projects to avoid impacts to the Moon Class I area if such relocation results in impacts to the Tribes, especially if the impacts have not been evaluated.

### RESPONSE

There are no specific proposals to relocate projects to avoid impacts at the Class I Moon National Monument. However, in cases where visibility impacts to the pristine environment of the Moon are shown to be a potential problem, all options, including changing or not proceeding with a project, would be evaluated. Potential visual impacts must be further defined before projects can proceed. Additional emissions controls and relocation of projects may be required to avoid potential impacts below acceptable criteria. As changes in visual setting, particularly in the southern portion of the INEL site, are seen by the Shoshone-Bannock Tribe, adverse effect on an important Native American resource, the Shoshone-Bannock Tribe will be consulted before any project is developed that could have impacts to resources of the area.

## II 05.02 (041) Air Quality

### COMMENT

The commentor suggests that the impacts from fugitive dust emission modeling should be evaluated between fugitive emissions from temporary and permanent sources.

### RESPONSE

The text in Volume 2, Appendix F-3.4.3 has been revised to more clearly distinguish between sources that are temporary (such as construction and demolition projects) and those that are permanent (such as unpaved roads and landfill operations). The specific fugitive dust sources have been identified.

## II 05.02 (043) Air Quality



## COMMENT

The commentor notes there seem to be variations in the application of models from one to another, virtually no information regarding source terms is given, and it is difficult to know what emissions have been considered and what emissions data were used.

## RESPONSE

In general, models were applied consistently between sites. However, site-specific modeling required a unique application. For example, the commentor mentions that site boundary impacts were assessed at some sites, but in other cases, off-site locations are considered. The commentor identified the ambient air location of highest predicted impact to the public and the maximum pollutant concentrations at that location for comparison with applicable standards. At INEL, the maximum impacted ambient air locations tend to be along public roads that traverse the site. At other sites, the nearest ambient air location may be the site boundary, because it traverses the site.

Temporary fugitive dust activities such as construction and demolition are exempt from quality standards; nevertheless, fugitive dust impacts from construction activities reported in Volume 1, and Appendices A through F.

For the other DOE sites evaluated in Volume 1, source emission rates are provided, characteristics (e.g., elevations, velocity, temperatures) are not provided in all cases more appropriate for a site-specific EIS. A discussion of the modeling and emissions is in Appendices A through F.

## II 05.02 (044) Air Quality

## COMMENT

Commentors assert that DOE cannot avoid responsibility for its past practices of categorizing its past activities as irreversible commitments of resources. Commentors put forward no compelling argument for further degrading the air of both the occupied land surrounding the Idaho National Engineering Laboratory and object to any irreversible air quality resources that could affect the Tribes' air quality, and also tourism. Commentors state that DOE provides no assurances that controls would be installed to avoid adverse impacts on air quality and visibility.

## RESPONSE

The air quality impact analyses have detailed the potential for air quality impacts. The analyses, for the most part, have been conducted for the site boundary and road. Additional analyses have been conducted for the Craters of the Moon National Monument and Hall Reservation. The analyses for criteria pollutant impacts have shown that impacts are within applicable ambient air quality standards. PSD standards, which have been established to prevent the degradation of air quality, would be met. Toxic pollutant impacts would be within applicable criteria. Impacts to air quality and visual resources at the Fort Hall Reservation operations will be even less, and this should not impact tourism.

Visual resource screening analyses were conducted at Craters of the Moon National Monument. The analyses used a screening methodology to determine the potential for worst-case impacts (maximum operating scenarios and adverse meteorological conditions). These analyses used conservative assumptions, including that many of the important proposed sources of impacts incorporate no or minimal emission controls. In many cases, projects are in concept and adequate design of emission controls is not yet available. However, impacts are not underestimated when conservative assumptions are used. A key aspect of the screening is the distance from the source to the potential impact area. The analysis showed some potential for impacts during the worst-case conditions. Methods to decrease the impact have not been determined. If discussed in the EIS, they will likely include controls to further reduce emissions and improve visibility. Siting factors will also be considered, as will refined modeling analyses (screening analyses). Through the Idaho Division of Environmental Quality's Permit Review process, proposed projects are required to demonstrate that there will be no adverse impacts on air quality and on visibility at Craters of the Moon. Any controls needed to avoid adverse impacts on air quality and visibility would be specified in permits.

Impacts to visibility, as well as criteria and toxic pollutant loading, should not be considered irreversible commitments of resources, but rather short-term impacts over the project life. Volume 2, section 5.18 has been revised to state that impacts to air quality and visibility are not irreversible commitments of resources.

## II 05.02 (047) Air Quality

### COMMENT

The commentor points out that the model receptor grid spacing is very large, and that spacing is necessary in areas of maximum predicted impact.

### RESPONSE

After the Draft EIS was completed, DOE performed additional analyses, primarily for consumption. As part of this analysis, a finely spaced receptor array was developed with receptor points spaced at approximately 100-meter intervals in those areas where the impact was predicted to occur. This dense array has since been used in the PSD analyses and has been incorporated into the appropriate sections of the EIS.

## II 05.02 (048) Air Quality

### COMMENT

The commentor notes that statements in Volume 2 that ozone levels are "not recognized in the region" and that the Idaho Division of Environmental Quality has determined that the state is well below the standard inaccurately describe ozone levels. The correct situation is that the Idaho Division of Environmental Quality has no ozone monitoring data in the vicinity and is not aware of problematic ozone levels in the area.

### RESPONSE

The statements cited by the commentor reflect verbal comments that were obtained by the Idaho Division of Environmental Quality. The authors acknowledge, however, that the statements could be misinterpreted to mean that ozone levels are not a problem to substantiate this claim may not be available. The statements in Volume 2 have been revised as follows: "The Division of Environmental Quality has no ozone monitoring data and is not aware of problem ozone levels in the area."

## II 05.02 (049) Air Quality

### COMMENT

The commentor considers the statement that "no previous projects have consumed increment of the Moon National Monument) to be unreasonable.

### RESPONSE

The commentor raises a valid question. Increment consumption is established by applications submitted with PSD permit applications, and accepted by the Division of Environmental Quality. Although two PSD permit applications have been previously submitted for the INEL project of increment consumption at Craters of the Moon National Monument, if any, had not been withdrawn and currently is being modified. The other application (for the Special Manufacturing Capability at Test Area North) had not been formally "closed out" at the time the Draft EIS was completed. As a result of discussions with the Division of Environmental Quality, it was decided to firmly establish the amount of increment consumption at the time that the Special Manufacturing Capability permit application was submitted and accepted, as of May 1, 1998. Additionally, it was decided that further analyses showing increment consumption by the EIS alternatives was also required. These analyses have been completed. The commentor's concern has been revised to reflect the updated results.

## II 05.02 (050) Air Quality

### COMMENT

The commentor points out that the assumption of Gaussian dispersion tends to break down at short distances, or where flow direction changes. The commentor further states that Gaussian models seriously underpredict impacts in these scenarios, and predictions for the Idaho National Laboratory boundary locations may be low.

### RESPONSE

While it is true that Gaussian models used to estimate upper bound levels of toxic be subject to the shortcomings noted by the commentor, the Industrial Source Comple (ISCST-2) model is generally regarded as appropriate for the type of modeling performed virtually every nonradiological case modeled, the highest ambient air impact occurs at locations. In these cases, the transport distances are not long and are well within the ISCST-2 model is considered appropriate. Results of calculations indicate 80 t occurs in the first 20 miles. Calculational assumptions selected by DOE were conservative uncertainties in calculational models.

## II 05.02 (051) Air Quality

### COMMENT

The commentor notes that there is a lack of any recent or reliable data about the efficiency of the filtering and ventilation systems in the building where the Fort St. Vrain spent nuclear fuel is stored at the Idaho National Engineering Laboratory. The commentor further states that the lack of uncertainty about the degree to which radionuclides emitted from the spent fuel might enter the environment through the storage facility's stack.

### RESPONSE

There is no lack of recent reliable data about the effectiveness of the filtering at the Irradiated Fuels Storage Facility where Fort St. Vrain spent nuclear fuel is stored. The facility is equipped with high efficiency particulate air (HEPA) filters having a verified 99.97 percent efficiency. Filter efficiency has been verified annually using standard Diocetyl Phthalate Records of these filter tests are available from 1979 to the present.

Regarding the commentor's statement about releases to the environment, stack releases are monitored and records show that nearly all radioactivity has been below detectable levels. To accurately assess historical releases to the environment, samples were obtained from the facility since the facility was constructed. From the analysis of the facility's annual radionuclide emission rate and annual dose to a maximally exposed individual, the dose is  $4.8 \times 10^{-6}$  millirem, which is significantly less than 1 percent of the limit of 10 millirem for DOE facilities by the Federal National Emission Standards for Hazardous Air Pollutants.

## II 05.02 (052) Air Quality

### COMMENT

The commentor asserts that it is incorrect to state that the GENII code tends to overestimate doses. The commentor further asserts that neither the GENII code nor CAP-88 (with which it was compared) has undergone a comprehensive validation study in the Idaho National Engineering Laboratory.

### RESPONSE

The commentor refers to a statement in Volume 2, Appendix F-3 to the effect that the baseline assessment are not likely to underestimate actual baseline or future doses. The statement is that baseline results in the EIS (which were modeled with GENII) were contained in the 1991 and 1992 National Emission Standards for Hazardous Air Pollutants (which were modeled with CAP-88). A study benchmarking these models in INEL settings has been completed (Radioactive Waste Management Complex Low-Level Waste Radiological Performance Assessment) and is discussed in Volume 2, Appendix F-3. The point of the study is to compare the model, including source-term and receptor-related assumptions, produces results that are conservative. The EIS has been revised to clarify this.

## II 05.02 (053) Air Quality

### COMMENT

The commentor notes that Volume 1, Appendix A, Table 5.7-1 does not show tritium releases. The commentor further states that tritium releases are not shown because they are not significant.

Tritium emissions from the K-basins have not been monitored because the emissions are so small that they do not contribute a very small amount to the dose received by the maximally exposed individual. In 1993, the average measured tritium concentration at the Hanford Site boundary was 0.90 picocurie per cubic meter and the maximum concentration was 1.8 picocurie per cubic meter. In 1993, the dose to the hypothetical maximally exposed individual from tritium was 0.0001 millirem.

emissions to the atmosphere was estimated to be 0.01 millirem. Volume 1, Appendix been revised to reflect these data.

## **II 05.02 (054) Air Quality**

### **COMMENT**

The commentor suggests that releases from four thermal treatment facilities at the Engineering Laboratory should be included in the EIS.

### **RESPONSE**

The four facilities identified by the commentor are the Waste Experimental Reductio Experimental Pilot Plant, the Idaho Chemical Processing Plant Denitration Facility, Calcining Facility. These facilities exist at INEL and are included in the baselin site. The Idaho Chemical Processing Plant Denitration Facility uses the main stack Processing Plant and is included in that source. Other thermal treatment facilitie Volume 2, Appendix C. The sources of emissions from site facilities appear in Volu 5.8, and are discussed in Volume 2, section 7.3.

## **II 05.02 (055) Air Quality**

### **COMMENT**

The commentor suggests that there is a lack of information concerning model use and hinders review and verification of the EIS.

### **RESPONSE**

Volume 2, Appendix F-3 discusses air dispersion modeling data and assumptions and h each INEL facility. Actual and foreseeable doses are a very small fraction of esta and are well below the National Emission Standards for Hazardous Air Pollutants (40 limit of 10 millirem per year.

## **II 05.02 (056) Air Quality**

### **COMMENT**

The commentor asks about the purpose of the comparison of hazard indices contained Appendix B.

### **RESPONSE**

Hazard indexes are compared to show that the data indicate no change from the basel hazard indexes under any of the alternatives. Volume 2, section 4.7 discusses the emissions. DOE has expanded the language in Volume 1, Appendix B, section 5.12 to relationship between hazard indexes and reference concentrations or doses.

## **II 5.3 Cultural Resources**

### **II COMMENT**

Commentors suggest that requirements under Section 106 of the National Historic Pre implemented early in the project planning process at the Idaho National Engineering RESPONSE

DOE agrees that this evaluation should be done early enough to allow historic prope fully during site selection and facility design. Requirements of the National Hist implemented during conceptual design if DOE proceeds with a proposed project.

### **II COMMENT**

Commentors assert that the EIS does not adequately address impacts on cultural reso

alternatives affecting the Idaho National Engineering Laboratory.

#### RESPONSE

The EIS identifies the number of known sites (approximately 1,500) on and percentag surveyed only to indicate the magnitude of potential sites at INEL. Volume 2, sect of predictive models and discusses the National Historic Preservation Act inventori completed prior to any actions. Volume 2, section 5.19 further discusses the Natio Preservation Act Section 106 requirements concerning the evaluation of sites and mi A comprehensive inventory of prehistoric cultural resources within the boundaries o To date, surveys to identify these resources have been focused on areas where adver likely to occur (i.e., facility perimeters, along major roadways and utility corrid addition, a preliminary predictive model has been developed to identify zones of pr resource density across the entire 890-square-mile facility. This model can be use managers during the initial stages of project planning to avoid areas where prehist be particularly dense, thus reducing the impact of INEL activities on sensitive cul Refinement and testing of this model are also under way through the INEL Cultural R Office. This office also maintains a complete record of all cultural resource inve INEL, as well as a database of all known cultural resources. Prior to conducting a activities, INEL project managers are directed to consult with the INEL Cultural R Office to avoid damage to any sensitive materials. Under the 1992 Working Agreemen Shoshone-Bannock Tribes of the Fort Hall Indian Reservation and the Idaho Field Off Concerning Environment, Safety, Health, Cultural Resources and Economic Self-Suffic are consulted and are given the opportunity to comment on any INEL project that has impact any cultural resource.

Based on public comments, DOE has expanded the EIS definition of cultural resources Volume 2, section 4.9 now includes a list of plants and vegetation important to the

## II COMMENT

The commentor expresses the opinion that there are not adequate agreements in place Shoshone-Bannock Tribes' archaeological artifacts and that options for removal of t and study should be considered, including executing a curation agreement.

#### RESPONSE

DOE has initiated the Working Agreement, Policy on Native American Consultation to communication with the Shoshone-Bannock Tribe relating to treatment of archaeologic excavation, as mandated by the Archaeological Resources Protection Act, and protect remains, as required under the Native American Graves Protection and Repatriation A DOE's Native American Policy (Memorandum EH-1: Management of Cultural Resources at of Energy Facilities, U.S. Department of Energy, Washington, DC, February 23, 1990) with Native Americans during the planning and implementation of all proposed altern remains are discovered, DOE notifies all tribes that have expressed an interest in as required under the Native American Graves Protection and Repatriation Act. The opportunity to claim the remains and associated artifacts. Also, the DOE Idaho Ope preparing a curation agreement pursuant to the Archaeological Resources Protection drafting a programmatic agreement for the protection of historic properties pursuan Historic Preservation Act. The handling of Native American cultural resource items American Graves Protection and Repatriation Act will be addressed by both of these Mitigation measures will be developed after these agreements are implemented. Volu been changed to reflect these agreements.

## II COMMENT

The commentor suggests that the EIS include mitigation measures in case cultural re inadvertently discovered during construction.

#### RESPONSE

This EIS is a programmatic document, based on current information and designed to p decisionmakers a broad base of knowledge about the affected environment, any forese any potential mitigation measures for an identified environmental impact associated of action. Providing specific, detailed mitigation measures, especially in areas w impact is foreseen, is beyond the scope of this document. Each DOE operations offi developing mitigation agreements, including actions to be taken in the event of dis

resources or human remains during construction. Such agreements will be negotiated with tribes and State Historic Preservation Officers. These agreements would be referred to in this information.

## II COMMENT

The commentor asserts that contamination resulting from transporting or storing SNF for hunting and gathering, which is as an irreplaceable part of the food supply and an economic activity for the residents of the Fort Hall Reservation.

### RESPONSE

There is a comprehensive environmental monitoring program at INEL, and the results are in the INEL Site Environmental Report. The monitoring conducted to date has not shown any change in game species or food stuffs that would preclude or limit hunting and gathering. Monitoring programs gather game species and food stuffs from a wide area in southeastern Idaho well beyond the boundaries of INEL in all directions.

Volume 1, Appendices D and I discuss impacts from both incident-free transportation accidents. The analysis shows that impacts from transportation activities for all are small.

## II COMMENT

The commentor objects to DOE's cultural resource impact analysis, because it minimizes impacts by fragmenting them and focuses solely on material culture.

### RESPONSE

DOE performs an analysis first by looking at the individual parts. This approach allows for a detailed analysis of ecology, water use, land use, air quality, etc., to evaluate impacts specific to those resources. When impacts are evaluated, the overall impacts to the resources are evaluated, thereby providing a more comprehensive approach. DOE agrees that impacts to the Shoshone-Bannock Tribes include all disciplines in the EIS; however, it is not feasible to include all these areas under cultural impacts. DOE does not presume to know the locations, absence or occurrence of items, sites, or resources to the Tribes over the whole INEL site. Nor would it be more protective of the site to conduct a site-wide survey than to conduct a complete site-specific analysis in consultation with the Tribes prior to any surface- or subsurface-disturbing activities. Broadly, DOE's approach is to conduct an initial survey, consult with the Tribes, and develop appropriate mitigation measures. The actions may include mitigation of impacts up to or including relocation of the alternative site.

Volume 2, section 4.3 has been changed to discuss the Tribes' broad view of cultural impacts. The response to comment 05.03 (002).

## II COMMENT

Commentors assert that the EIS does not adequately address impacts on cultural resources and that the EIS requires DOE to continue consultations with the Tribes.

### RESPONSE

The number of known sites (approximately 1,500) and the portion (4 percent) of the sites that have been surveyed are identified in the EIS only to suggest the large number of potential impacts. Volume 2, section 4.4 discusses the use of predictive models and discusses the National Historic Preservation Act Section 106 requirements concerning the evaluation and mitigation of impacts.

A comprehensive inventory of prehistoric cultural resources within the boundaries of the facility. To date, surveys to identify these resources have been focused on areas where adverse impacts are likely to occur (i.e., facility perimeters, along major roadways and utility corridors). In addition, a preliminary predictive model has been developed to identify zones of potential impacts. This model can be used by resource managers during the initial stages of project planning to avoid areas where prehistoric resources are particularly dense, thus reducing the impact of INEL activities on sensitive cultural resources.

Refinement and testing of this model are also under way through the INEL Cultural Resource Office. This office also maintains a complete record of all cultural resource inventory at INEL, as well as a data base of all known cultural resources. Prior to conducting activities, INEL project managers are directed to consult with the INEL Cultural Resource Office to avoid damage to any sensitive materials. Under the 1992 Working Agreement with the Shoshone-Bannock Tribes of the Fort Hall Indian Reservation and the Idaho Field Office, Concerning Environment, Safety, Health, Cultural Resources and Economic Self-Sufficiency, the Tribes are consulted and are given the opportunity to comment on any INEL project that has impact on any cultural resource.

Based on public comments, DOE has expanded the EIS definition of cultural resources. Volume 2, section 4.9 now includes a list of plants and vegetation important to the area. DOE has increased its consultation with the Shoshone-Bannock Tribes. A series of consultations with tribal management and technical personnel from the Tribes and DOE have resulted in a better resolution of mutual concerns. DOE continues to meet with the Tribes and plans to implement the actions proposed in the EIS.

## II 5.4 Biological Resources

### II COMMENT

The commentor notes that many studies have been conducted by biologists, botanists, and archaeologists at the Hanford Site and Idaho National Engineering Laboratory areas with intriguing results.

Every effort has been made to review all pertinent studies for inclusion in the EIS. The commentor requested that the speaker identify any pertinent additional studies so they could be evaluated. None was provided by the commentor.

### II COMMENT

The commentor suggests that the EIS considers the Arco desert to be a wasteland suitable for spent nuclear fuel, which the commentor believes is a gross misunderstanding of the surrounding geography.

#### RESPONSE

DOE and the Navy consider sensitive ecosystems and habitats when designing and siting facilities to comply with the laws and regulations protecting wildlife resources, including those for threatened and endangered species, to ensure the impacts of proposed activities are minimal. In Volume 1, Chapter 5, measures for protecting ecological resources would be developed in consultation with the appropriate agencies if any sensitive ecosystems or habitats are identified on a project site. Preconstruction surveys would be conducted to determine the presence of these resources. INEL is a National Environmental Research Park.

### II COMMENT

The commentor states that Idaho National Engineering Laboratory operations have caused the death of animals and endangered species.

#### RESPONSE

DOE agrees with the commentor and notes that it has designated INEL a National Environmental Research Park. DOE considers threatened and endangered species and sensitive habitats when siting facilities and conducting its programs. It complies with the laws and regulations protecting wildlife resources, including those for threatened and endangered species, to ensure that the impacts of DOE activities are minimal. In Volume 1, section 5.7.7, measures to avoid or mitigate impacts to ecological resources would be developed in consultation with the appropriate agencies if any sensitive ecosystems or habitats are identified on a project site. Preconstruction surveys would be conducted to determine the presence of these resources.

## II COMMENT

The commentor asks about risks to the fragile ecosystem of marine waters near Seattle  
RESPONSE

Volume 1, Chapter 5, Appendices D and K, and Volume 2, Chapter 5 summarize the environmental impacts of all the alternatives considered in this EIS. The analyses show that the impacts would be small. While there are differences in the impacts among the alternatives, the differences themselves are not sufficient to distinguish between alternatives.

## II COMMENT

The commentor states that the EIS must address wildlife management practices at the Engineering Laboratory as well as the impacts to wildlife that could result from the use of the Tribes should be afforded hunting rights on the site.

RESPONSE

While DOE manages the game habitat at INEL, the State of Idaho manages wildlife and over hunting rights within the INEL boundary. Issues relating to wildlife management and hunting rights must be addressed to the state.

Impacts to wildlife that could occur as a result of the various alternatives, and the measures, are discussed in Volume 2, Chapter 5, sections 5.9 and 5.19.

## II COMMENT

The commentor states that Idaho National Engineering Laboratory impacts cannot be avoided at specific sites selected for certain new construction projects, and that DOE should protect wildlife habitat by clustering new facilities near currently disturbed areas.

RESPONSE

Volume 2, Appendix C specifies the location of potential disturbances. DOE has approved proposed activities in the most environmentally benign locations that will meet health and safety requirements. Siting was considered in the following order of preference: (1) locate in previously disturbed areas, (2) locate in existing industrial areas on previously disturbed areas, (3) locate in undisturbed areas, (4) locate outside, but immediately adjacent to, industrial area and away from existing industrial areas.

The three projects that would cause most of the disturbance outside and separate from industrial areas are the Idaho Waste Processing Facility, the Alpha-Mixed Low-Level Waste Facility, and the Alpha-Mixed Low-Level Waste Disposal Facility. All three projects are in the conceptual design phase and would require project-specific NEPA documentation before construction is committed. Because it is still in the design phase, the specific location for the Alpha-Mixed Low-Level Waste Facility is not well defined. The EIS states that it may be located near the Radiochemical Complex (RWMC) or at other existing industrial locations on the INEL site. For purposes of the ecological consequences section of the EIS, the Idaho Waste Processing Facility is located 2.5 kilometers (2.5 miles) east of the RWMC. This is the most conservative siting method that would result in the largest impact to ecological resources. Similarly, the Alpha-Mixed Low-Level Waste Treatment Facility and the Alpha-Mixed Low-Level Waste Disposal Facility may be located near existing INEL facilities. The most conservative assumption was used for the analysis that the Alpha-Mixed Low-Level Waste Disposal Facility would be built 4 kilometers (2.5 miles) west of the RWMC.

As stated in the EIS, DOE would perform site-specific preconstruction surveys to identify potential resources on the site to ensure that impacts from the proposed actions are identified and measures can be developed and integrated into the project.

## II COMMENT

The commentor states that Volume 1, Appendix F should include language to ensure that wetland resources are preserved, if such resources exist. The commentor also states that the impacts to wetlands on a proposed construction site is not addressed.

RESPONSE



As discussed in Volume 1, Appendix F, Part Two, sections 4.9.2 and 5.9.1, there are proposed SNF site at the Nevada Test Site (NTS); thus, no special preservation effort. Oak Ridge Reservation (ORR) wetlands are discussed in Volume 1, Appendix F, Part Two and 5.9.1. It is DOE policy to comply with Executive Order 11990, Protection of Wetlands, government agencies to avoid any short- and long-term adverse impacts on wetlands with practicable alternative. If ORR is chosen as a site for SNF management, the potential wetland resources on the site would be specifically analyzed, along with potential otherwise mitigate impacts. Unavoidable impacts to wetlands would be mitigated according to policy.

## II COMMENT

The commentor states that mitigation measures, including those for the desert tortoise addressed in Volume 1, Appendix F.

### RESPONSE

A biological opinion concerning the desert tortoise has been issued by the U.S. Fish covering current projects at the NTS. (See Volume 1, Appendix F, Part Two, section in Volume 1, Appendix F, Part Two, section 5.9, recommended mitigation measures include surveys for the tortoises and their removal from affected areas, as well as periodic backfilling, covering, or installation of tortoise-proof fencing around open construction excavations, and reducing speed limits on site roadways. After consulting with the Service and the Nevada Division of Wildlife, similar recommendations would be implemented appropriate, if NTS were selected as the location for a SNF facility. Providing such measures is beyond the scope of this EIS and will be addressed in tiering NEPA documentation appropriate.

## II COMMENT

Commentors state that the EIS failed to consider potential impacts on fish and wildlife of spent nuclear fuel and other hazardous materials. This includes accidents, alternative threat reduction, and mitigation of impacts to wildlife from transportation accidents.

### RESPONSE

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environmental all the alternatives considered in this EIS. The analyses show that the impacts of small, including the impacts to fish and wildlife. While there would be differences between the alternatives, these differences by themselves are not sufficient to distinguish between the alternatives. Volume 2, section 5.19 addresses mitigation for both operations and accident conditions. section 5.11 covers all transportation impacts, including incident-free transportation accidents. Regional traffic impacts are also covered. As noted in Volume 2, section movements of materials and people due to all alternatives would result in no change of U.S. Highway 20, the regional highway with the highest use around INEL.

An accident with a release of radionuclides or hazardous material into the environment temporary exposures of biota. The impact would likely be localized and of short duration. Fish and Wildlife Departments and Natural Resource Trustees would be consulted to determine the most appropriate response for the specific accident and current conditions. The emphasis would focus on cleaning the site and removing contaminants as completely and as rapidly as possible. While radiological impacts from accidents could result in loss of individual animal losses or large-area losses would not be anticipated. Impacts to fish would depend on the quantity spilled into the aquatic environment, and must be evaluated on a case-by-case basis. Volume 2, Chapters 4 and 5 have been modified to include information on threat reduction of the impacts of collision accidents on wildlife.

## II COMMENT

The commentor states big game kills by trains are not reported in the EIS, and increased kills by train transport are not addressed in the EIS.

### RESPONSE

Information was obtained from the State of Idaho Division of Wildlife Management concerning

involving trains killing large numbers of pronghorn antelope. This information has Volume 2, section 4.11. See also the response to comment 05.04 (011) regarding how to evaluate impacts of transportation accidents.

## II COMMENT

The commentor asks about depredation problems associated with antelope and elk in the Engineering Laboratory area.

### RESPONSE

The alternatives would disturb up to 726 acres of land outside of current facility. While depredation may increase, the increase is likely to be low because most of the land is located about 5 kilometers (3 miles) from the RWMC, which is located within the INE far from any croplands. Policies concerning restrictions on hunting at INEL are not in the EIS.

## II COMMENT

The commentor notes that a statement that no Federally listed species are expected during construction and operation of the spent nuclear fuel management facility is in conflict with Appendix F, Part 3, Table 4.9-1.

### RESPONSE

Volume 1, Appendix F, Part 3, Table 4.9-1 lists species that "potentially occur on Oak Ridge Reservation" but not necessarily on the project site. Volume 1, Appendix 4.9.4 describes the expectation of species occurrence on the proposed project site, most likely to occur on the project site, none of which is Federally listed. None of the species listed in Table 4.9-1 has been observed on the proposed project site. No species listed as threatened by the U.S. Fish and Wildlife Service, in accordance with the Endangered Species Act, occur on the site and, thus, they would not be impacted. Impacts to state-listed species are described in Volume 1, Appendix F, Part Three, section 5.9.1. There may be impacts on other special-status species, which consist of two plant and five raptor species. The effect to wildlife habitats is discussed in Volume 1, Appendix F, Part Three, section 5.9.2. The forested habitat would be a small percentage of the total forested area on or in the

## II COMMENT

The commentor expresses the opinion that storing spent nuclear fuel at the Savannah River Site is a potential ecological problem.

### RESPONSE

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environmental impacts of all the alternatives considered in this EIS. The analyses show that the impacts are small. While there are differences in the impacts among the alternatives, these differences are not sufficient to distinguish between alternatives.

For the Savannah River Site (SRS), potential effects from operations conditions would be disturbance of habitat, rather than effects from radionuclides. Potential effects from radionuclides result in exposures to biota. However, emergency response would limit the potential effects to a localized area.

## II COMMENT

The commentor suggests that terrestrial biota may be subject to more radiation exposure because human exposure can be limited by special clothing and protective equipment.

### RESPONSE

Terrestrial biota are not subject to exposure under conditions that would require special protective equipment for humans. Work areas where potential radiation exposure is monitored and site workers use protective equipment have controlled access measures that limit exposure. So long as exposure limits for humans are not exceeded, no substantial radiation exposure to biota is expected.

biota would be expected as a result of waste management activities at the proposed facility.

Volume 1, Appendix F, Part Two, section 5.9 has been modified to clarify that most activities take place in enclosed environments and that outdoor radiation exposures regulatory requirements.

## II COMMENT

The commentor states that until surveys are conducted at the Oak Ridge Reservation sensitive flora, fauna, and habitat is in question and could be a factor in selecti plan.

### RESPONSE

The commentor is accurate in stating that until site surveys are completed, the sta and habitat remain in question and could be a factor in the selection of the specif selected. The analyses in the EIS are based on existing documentation.

## II COMMENT

The commentor suggests that animals near proposed new or expanded facilities in Ida relocated to a similar environment.

### RESPONSE

Generally, it is not feasible to relocate all animals disturbed by construction act that would be displaced include insects, reptiles, and small mammals. Preactivity conducted to determine if any endangered species or sensitive habitats are in the a practical, proposed facilities are clustered near existing facilities to minimize i Measures to minimize impacts to wildlife at INEL are discussed in Volume 2, sectio

## II COMMENT

One commentor states that DOE and the Navy have failed to study the possibility tha Columbia and Snake Rivers to Idaho could pick up radioactive particles, contaminate wilderness areas, and impact endangered species. Another commentor states that the be a poor storage area unless the already "depleted salmon" are protected.

### RESPONSE

Volume 1, Appendix A, sections 4.8 and 4.9 have been modified to address potential life in the Columbia River. Volume 1, Chapter 5 and Appendix K, and Volume 2, Chap the environmental impacts of all the alternatives considered in the EIS. The analy of all alternatives would be small.

All liquid effluents from Hanford Site facilities are monitored to ensure that aqua protected. Fish populations are safe for human consumption. Radionuclide levels i Reach are not significantly higher than those of fish found upstream. Fish migrati River up the Snake River to Idaho would not pass through the Hanford area, because two rivers is downstream from the Hanford Site. Fish inhabiting or moving through would also not be expected to have elevated radionuclide levels.

Any new facility would be built using technologies to protect these resources, incl water-balance monitoring equipment. Excess process water from the proposed facilit before it is released to surface water or groundwater.

In some accident scenarios, such as a seismic event at Hanford with a frequency of every 1,000 years, contamination could reach the Columbia River. Individual fish i the river could become contaminated. However, contamination spread by the fish, an would be small compared with the environmental risk posed by more direct pathways i scenario. Monitoring at DOE facilities indicates the most critical pathways for en contamination are generally through direct airborne and waterborne releases, rather spread through animals or fish.

## II COMMENT

The commentor states that impacts of transport, storage, and accidental releases on and sensitive species should be considered.

RESPONSE

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environment all the alternatives considered in this EIS, including those to threatened and endangered species. Analyses show that the impacts of all the alternatives would be small.

Threatened and endangered species and habitats are considered in the design and siting of facilities. Volumes 1 and 2, section 7.2.1 identify all Federal environmental statutes including the Endangered Species Act, that may apply to the programmatic alternative management. DOE and the Navy comply with all applicable laws and regulations designed to ensure wildlife resources to ensure impacts are minimal. These regulations include U.S. Department of Transportation (DOT) regulations on transport of hazardous and/or radioactive materials. Minimizing impacts to sensitive species are described in Volumes 1 and 2, Chapter 5.

## II COMMENT

The commentor states that there are virtually no data or literature references to support Engineering Laboratory ecological analyses and conclusions.

RESPONSE

The Environmental Resource Document for the Idaho National Engineering Laboratory (INEL) provides an extensive compendium of documentation concerning the INEL environment and ecology. Additionally, Radioecology of the Idaho National Engineering Laboratory (Draft) provides a literature search and an evaluation of current INEL operations. Both of these documents are referenced in the EIS and are available in the rooms and information locations listed in the EIS.

## II COMMENT

The commentor questions the effects on endangered species in the Twin Falls Thousand Springs area as a result of impacts to the Snake River aquifer.

RESPONSE

Under all alternatives considered, possible future sources of aquifer contamination would be expected to improve under current waste management alternatives. Increased water use at INEL would range from 1.3 percent under the No Action alternative to 4.0 percent for the Ten-Year Plan alternative; or approximately 0.43 to 1.3 percent beneath INEL. Currently, a substantial portion of water pumped from the aquifer at the surface and eventually returned to the aquifer. The current water withdrawal represents 0.43 percent of a typical irrigation well pumped 365 days per year. Because of the small amount consumed, there would be a small impact to water levels or quantities in the aquifer. Endangered species in the Thousand Springs area. A discussion and evaluation of potential impacts to water quality and quantity under the alternatives analyzed is provided in Volume 1, Chapter 5 and Attachment A.

## II COMMENT

The commentor states that it would be inappropriate to ship spent nuclear fuel through the Puget Sound area.

RESPONSE

The EIS evaluates potential environmental impacts of transporting SNF in the Puget Sound area. Shipments of Naval SNF are made in accordance with all applicable regulations. Shipments of radioactive materials associated with Naval SNF have never resulted in a measurable release of radioactivity to the environment, nor has there ever been an release of radioactive material during shipment since the Naval Nuclear Propulsion Program. Potential impacts to the local environment at Puget Sound from transportation of SNF are discussed in Volume 1, Appendix D, Chapter 5 and Attachment A.

## II COMMENT

The commentor states that the EIS neither describes ongoing activities nor analyzes association with past and future activities and is therefore not comprehensive.

### RESPONSE

Volume 2, Chapter 4 describes the existing environment at INEL. Volume 2, Chapter current activities, facilities, and missions at INEL. Site-specific impacts, included in Volume 2, Chapter 5 and Appendix F. Volume 1, Chapter 5 and Appendix Chapter 5 summarize all of the alternatives considered in this EIS. The analysis of all alternatives would be small.

## II COMMENT

The commentor states that the Draft EIS should address loss of habitat at the Oak Ridge the effects on the regions ecosystems by a change in land use.

### RESPONSE

Both land use and habitat loss are considered in Volume 1, Appendix F. ORR occupies square kilometers (54 square miles). In 1980, DOE designated 54 square kilometers undeveloped ORR land to a National Environmental Research Park. Approximately 58 percent of ORR [80 square kilometers (31 square miles)] can be classified as undeveloped due to designation. By comparison, the SNF program would require about 0.36 square kilometers (0.14 square miles). Volume 1, Appendix F, Part Three, section 5.9 assesses impacts to ecological resources of the Centralization and Regionalization alternatives. Neither alternative would preclude impacts to ecological resources through alterations or loss of habitat.

## II 5.5 Geology

## II COMMENT

The commentor notes that no geologists from the Oak Ridge area were used to help prepare Appendix F, Part Three.

### RESPONSE

The document was prepared using existing references and currently published information cited for the Volume 1, Appendix F, Part Three discussion of ORR include current information in that area.

## II COMMENT

The commentor is of the opinion that the EIS is a coverup, especially regarding seismic geologic events.

### RESPONSE

The best available information relative to seismic hazards and geologic events is presented in Volume 2, section 4.6, the site-specific appendices to Volume 1, and associated reference information sufficient to allow independent evaluation of the seismic hazards and geologic events.

## II COMMENT

The commentor notes that the Knox Group is divided into five formations, not four.

are the Copper Ridge Dolomite, the Chepultepec Dolomite, the Longview Dolomite, the Formation, and the Mascot Dolomite.

RESPONSE

The EIS has been revised to incorporate the information.

## II COMMENT

The commentor states that the EIS does not address correcting current seismic defic National Engineering Laboratory facilities.

RESPONSE

DOE Order 5480.28, National Phenomena Hazards Mitigation, specifically requires fac reevaluated when there is any change in design and construction standards. Existin undergone continual safety analysis and seismic design review. Several of the proj Volume 2, Appendix C are proposed by DOE to replace or upgrade facilities at INEL. such as the transfer of fuels from potentially vulnerable facilities to modern faci the ongoing safety analysis and seismic design reviews. Volume 2, Table 2.2.1 addr seismic deficiencies identified with fuel storage facilities at INEL.

## II COMMENT

The commentor states that storing radioactive material in a seismically active area Engineering Laboratory could result in catastrophic consequences.

RESPONSE

Seismic hazards and geologic analyses can be found in Volume 1, section 4.2 and App and Volume 2, section 4.6 and Appendix F-2. Seismically induced accidents are dis section 5.14 and Appendix F-5. The results of accident analyses (including seismic indicate that the risk to the public from INEL operations is small. DOE takes seis seriously, and INEL uses independently and extensively reviewed analyses to support implementation of DOE Orders and standards. An INEL seismic hazard assessment was 1990. A more recent seismic hazard assessment for INEL is referenced in the EIS as Probabilistic Seismic Hazard Analysis for the Idaho National Engineering Laboratory the response to comment 05.05.01 (040).

## II COMMENT

Several commentors state that geologic conditions at the Idaho National Engineering result in a sequence of events that would cause contamination of the Snake River Pl RESPONSE

An accident scenario resulting in maximum potential for groundwater contamination a in the EIS in Volume 2, section 5.14 and Appendix F to determine the effects of suc Snake River Plain aquifer. The hypothetical accident involves the instant failure due to an earthquake. The groundwater analysis assumed failure of the containment measures to minimize flow from the waste tank into the soil immediately following t hypothetical scenario represents the situation with the maximum reasonably foreseea aquifer. Maximum radionuclide concentrations would be predicted to reach the INEL after the hypothetical accident in concentrations less than EPA maximum contaminant DOE derived concentration guidelines (DCGs). See also the response to comment 05.0

## II COMMENT

Commentors express opinions that the selection of the Oak Ridge Reservation as an performed in haste, and/or did not adequately consider the geology of the West Bear RESPONSE

The selection of ORR and NTS as alternative sites resulted from public comments rec scoping process for this EIS. Information about the site-selection process at ORR for Support in Preparing the Spent Nuclear Fuel and Idaho National Engineering Labo

Environmental Restoration and Waste Management Environmental Impact Statement, and Selection Decision Process Report.  
The West Bear Creek Valley site was selected for evaluation and comparison in this geologic information was considered in making this selection. Adequate information programmatic decisions and evaluate alternatives in this EIS.

## II COMMENT

The commentor states that significant adverse geologic events could cause radioactive  
RESPONSE

The general geological features of the alternative sites are described in Volume 1, impacts associated with geologic events are summarized in Volume 1, Chapter 5, section the geological features and potential dangers associated with those features are in through F for the alternative sites. DOE recognizes the potential adverse effects have on facilities, and the EIS includes analysis of accidents and the potential consequences with geologic events, such as earthquakes. The accidents evaluated included those probability ranging from once in 1 million years to once in 10 million years. As discussed in section 5.1.6, the probabilities of accidents with the potential for significant impacts are small. The risks to the public from radioactive releases would be small for all of the sites. See also the response to comment 05.05.01 (016).

## II COMMENT

The commentor states that it is appropriate to acknowledge the zinc and fluorspar deposits northeast of Knoxville, Tennessee, and southwest of the Oak Ridge Reservation, respectively. The commentor also notes that zinc prospects and sulfide mineralization may occur in the area.  
RESPONSE

As required by Council on Environmental Quality (CEQ) regulations, the description of the environment is no longer than is necessary to understand the effects of the alternatives; therefore, impacts to geologic resources are expected from any of the alternatives; therefore, impacts to geologic resources are not expected. If ORR is chosen as a site for new SNF management facilities, studies would be performed as necessary to determine the full extent of geologic resources at the site.

A discussion of the geologic resources at ORR is presented in the EIS in Volume 1, Three, section 4.6.

## II II COMMENT

The commentor questions the adequacy and conservatism of seismic hazard studies at Engineering Laboratory.  
RESPONSE

Seismic hazards and geologic analyses for INEL can be found in Volume 1, section 4. Appendix B, section 4.6; and Volume 2, section 4.6 and Appendix F-2. Seismically induced accidents are discussed in Volume 2, section 5.14 and Appendix F-5, and Volume 1, Appendix B, section 4.6. Accident analyses (including seismically induced accidents) indicate that the risk to operations is small. DOE takes seismic hazards very seriously, and INEL uses independent analyses to support the enforcement and implementation of DOE Orders and standards. Major DOE Idaho Operations Office-managed nuclear facilities currently in use at INEL have been evaluated to design basis accelerations that exceed accelerations that would result from a magnitude earthquake at the southern end of the Lemhi fault zone. There has been a commitment in the past several years to upgrade DOE Orders and standards related to natural phenomena hazards. DOE Order 5480.28, Natural Phenomena Hazards Mitigation, sets forth DOE procedures to assess, and operate DOE facilities so that workers, the general public, and the environment are protected from the impacts of natural phenomena hazards on DOE facilities. This Order specifies that facilities to be reevaluated when there is any change in design and construction standards. Facilities have undergone substantial safety analysis and seismic design review. Studies described in Volume 2, Appendix C of the EIS are proposed by DOE to replace or upgrade the seismic hazard site. Likewise, actions such as the transfer of fuels from potentially vulnerable

have resulted from the ongoing safety analysis and seismic design reviews. The data and methods used in the seismic hazard report referenced in Volume 2, section 4.6, were extensively and independently reviewed. This report includes graphs showing response versus acceleration for seismic events for each major facility at INEL. The seismic hazard analysis for the Idaho Chemical Processing Plant was included as an example of the information contained in the seismic hazard analysis [Site-Specific Probabilistic Seismic Hazard Analysis for the Idaho National Engineering Laboratory (Draft)]. The final versions of this report may be incorporated into architectural and engineering standards after review by the INEL Natural Phenomena Committee. The previous INEL seismic analysis (Earthquake Strong Ground Motion Estimates for the Idaho National Engineering Laboratory: Final Report) was also extensively reviewed and incorporated into INEL standards after review by the Natural Phenomena Committee in 1992. This report is referenced in Volume 2, section 4.6 and Volume 2, Appendix F-2 and contains location-specific seismic hazard information. The EIS summarizes current scientific evidence relevant to understanding the existing seismicity, identifying reasonably foreseeable impacts, and evaluating potential consequences. The seismicity analysis is based on methods generally accepted by the scientific community. See also the response to comment 05.05.01 (007).

## II COMMENT

The commentor states that the Basin and Range Province north of the Idaho National Engineering Laboratory lacks adequate seismic monitoring.

### RESPONSE

During 1991 and 1992, DOE increased its network of seismic monitoring stations from 10 to 20, including stations in the Basin and Range Province. This network supplements measurements by U.S. Geological Survey (USGS) facilities. INEL regularly exchanges data with other seismic networks around the region, including data for earthquakes that occur between networks. Currently, the networks are supporting studies of the 1994 Raney Peak earthquake sequence and have the 1983 Borah Peak earthquake studies.

## II COMMENT

The commentor questions why the overall level of seismic hazard calculated in the EIS for the Idaho National Engineering Laboratory is lower than the seismic hazard curves for either the Hanford River Site or the Nevada Test Site.

### RESPONSE

The possible reasons for the relatively low seismicity, with respect to the more seismically active Basin and Range Province, for the Eastern Snake River Plain (ESRP) are discussed in Volume 2, section 4.6. Differences noted by the commentor result from the site-specific data used to assess seismicity. INEL has modeled ground motions based on site-specific analyses. Ground motion characteristics result from using source parameters for Basin and Range Province earthquakes (e.g., lower recurrence intervals for the southern segments of the Basin and Range Province, Lost River, and Beaverhead), and the unique subsurface geology (interbeds of basalt and sandstone tend to deamplify ground motions).

Additional factors contributing to the relatively low seismic hazard for INEL are the distance from Basin and Range Province faults, INEL-specific attenuation characteristics, and the low seismicity of the ESRP.

The Hanford Site models use empirical data derived from California earthquakes and the 1906 San Francisco earthquake. SRS has a thicker layer of soil and subsurface geology that causes more scattering during transmission of seismic waves. Both of these conditions tend to increase seismicity at SRS.

## II COMMENT

The commentor notes that within 125 miles of the Idaho National Engineering Laboratory are several active fault segments, including 20 with proven late Quaternary or younger displacement. The commentor suggests that this observation is inconsistent with the relatively low seismicity at the Idaho National Engineering Laboratory presented in the EIS.



**RESPONSE**

In the Probabilistic Seismic Hazard Assessment studies [Site-Specific Probabilistic Analysis for the Idaho National Engineering Laboratory (Draft)] referenced in the E and determined the major seismic sources in the vicinity of INEL. Because most of noted by the commentor are some distance from INEL, they are not significant contri hazard. The closest and most significant seismic sources, the Beaverhead, Lost Riv considered in INEL seismic hazard assessments. The Probabilistic Seismic Hazard As INEL have been independently reviewed and are developed consistent with the require 5480.28, Natural Phenomena Hazards Mitigation. The details of the characterization seismogenic sources, and how they are incorporated into seismic hazard assessments Volume 2, section 4.6 or its references.

The possible reasons for the relatively low seismicity, with respect to the more se Province, for the ESRP are discussed in Volume 2, section 4.6. The differences not result from the site-specific data used to assess seismic hazards. In particular, motions based on site-specific analyses instead of empirical data. These curves re parameters for Basin and Range Province earthquakes with lower stress drops, lower for the southern segments of the Basin and Range Province faults, including the Lem Beaverhead faults, and the unique subsurface geology of interbeds of basalt and sed deamplify ground motions. Additional factors contributing to the low seismic hazar other DOE sites) are the distance from the facilities to Basin and Range Province f attenuation characteristics, and the low seismicity of the ESRP. See also the resp 05.05.01 (003).

**II COMMENT**

The commentor states that the coastal plain of South Carolina and Georgia is earthq faults in multiple directions" and is a poor site for temporary or long-term storag  
**RESPONSE**

The general geologic features of the alternative sites are described in Volume 1, C potential impacts associated with geologic events are summarized in Chapter 5, sect the geologic features and potential dangerous events associated with those features geologic events can have on facilities, and the EIS includes analysis of accidents consequences associated with geologic events, such as earthquakes. The accidents e with an estimated probability ranging from once in 1 million years to once in 10 mi described in Volume 1, section 5.1.6, the probabilities of accidents occurring with significant impacts would be small. The accident analyses (including seismically i indicate that the risk to the public from DOE operations would be small. Because D procedures and engineering design practices that minimize the effects of hazardous coupled with emergency response measures, the risks to the public from radioactive reduced.

The site-specific response can be found in Volume 1, Appendix C, section 4.6.3, whi region's geology, including fault systems and seismic history; section 5.8, which d of analyzed seismic events on both surface water and groundwater resources; and Vol Attachment A-2.1.3, which describes estimates of risk that consider both the probab consequences from a wider range of seismic events, ranging from local and regional documented earthquakes to postulated lower probability events with potentially grea

**II COMMENT**

The commentor quotes a Woodward-Clyde study, commissioned by DOE, as having more re measures of likely ground motions and suggests that DOE adopt these standards as an  
**RESPONSE**

DOE has adopted this study (Earthquake Strong Ground Motion Estimates for the Idaho Engineering Laboratory: Final Report) and has incorporated the resulting seismic g the architectural and engineering standards for INEL.

**II COMMENT**

The commentor states that a great deal more research, both onsite and in the surround necessary before the Snake River Plain can be declared "aseismic."

#### RESPONSE

Seismic hazards and geologic analyses can be found in Volume 1, section 4.2; Volume section 4.6; and Volume 2, section 4.6 and Appendix F-2. Seismically induced accidents in Volume 2, section 5.14 and Appendix F-5. The accident analyses, including seismic accidents, indicate the risk to the public from INEL operations would be small.

The assertion that the Snake River Plain has a low rate of seismicity is supported in Volume 2, Figure 4.6-3, which represents a summary of the best available data at the time the EIS was compiled, and states the years over which the data were collected. The addition of seismic events in the region would not change the conclusion that the Snake River Plain is in the EIS to eliminate confusion.

Empirical evidence does not support the commentor's assertion that a major seismic event is likely in the future on the ESRP. Studies of fault scarps on the ESRP indicate that a seismic event with a moment magnitude of 5.3 is the maximum event recorded in the rocks at the surface, from 1.2 million to 2,100 years old. Thus, there is long-term geologic evidence with which to assess its magnitude of seismicity. The moment magnitude estimate is conservative with respect to earthquake magnitudes observed in similar tectonic environments and the assumed instantaneous stress release. Further conservatism in the seismic hazard analysis in the EIS [Site-Specific Probabilistic Seismic Hazard Analysis for the Idaho National Engineering Laboratory (Draft)] is introduced through the use of a random ESRP earthquake, which has a moment magnitude of 5.5 to 6.0. The methods and data used in this study have been reviewed. The random earthquake is used to analyze the potential effects of potential events related to structures that do not have a surface expression.

Stress indicators show that the ESRP is subject to the same extensional stress as the Basin and Range Province. There is geologic evidence to support the hypothesis that the ESRP has the same rate as the Basin and Range Province but by the different, less seismically in basaltic dike injection. The rate and magnitude assumed for the random earthquake is conservative with respect to these observations. These observations also indicate that energy is being stored for release in a major seismic event. Other possible explanations for seismicity on the ESRP can be found in Volume 2, section 4.6. The hypothesis that stored elastic energy could lead to a catastrophic brittle failure of the crust below INEL is not supported by published earth science literature or the local geology of INEL. Despite mapping of INEL and the catastrophic faulting event has not been observed in surface basalt flows that are old.

The EIS summarizes existing credible scientific evidence relevant to understanding the environment, identifying reasonably foreseeable impacts, and evaluating potential consequences. The evaluation of impacts is based on methods generally accepted by the scientific community. The EIS evaluates the potential consequences of reasonably foreseeable events.

## II COMMENT

The commentor states the potential for major earthquakes on the Plain exists, and that earthquakes on the plain do not provide the clear threat to Idaho National Engineering Laboratory. Earthquakes on the fault systems north of the plain provide, the possibility of even the plain cannot be discarded.

#### RESPONSE

DOE assumes the commentor is referring to the ESRP. Empirical evidence does not support the commentor's assertion that a major seismic event is likely to occur in the future on the ESRP. Studies of fault scarps on the ESRP indicate that a seismic event with a moment magnitude of 5.3 is the maximum event recorded in the rocks at the surface, which range in age from 1.2 million to 2,100 years old. Thus, there is long-term geologic evidence with which to assess the magnitude of seismicity. The moment magnitude 5.3 estimate is conservative with respect to earthquake magnitudes observed in similar tectonic environments and the assumed instantaneous stress release. The possibility of a major earthquake on the ESRP was not discarded and has been considered in the seismic hazard analysis in the EIS [Site-Specific Probabilistic Seismic Hazard Analysis for the Idaho National Engineering Laboratory (Draft)] through the use of a random ESRP earthquake, which has been assigned a moment magnitude 5.5 to 6.0. The data and methods used in this study have been independently reviewed. The random earthquake is used to analyze the effects of seismic events related to structures that do not have a surface expression.

## II COMMENT

The commentor states that earthquake magnitudes used for seismic analysis in the EI more research, both onsite and in the surrounding region, is required to adequately seismic shaking possible on the INEL site.

### RESPONSE

The methods and data used in the Site-Specific Probabilistic Seismic Hazard Analysis National Engineering Laboratory (Draft) have been independently reviewed, and the a therein, including the analysis and earthquake magnitude estimates that resulted in scientifically defensible. The important parameters for the seismic hazard assessment Volume 2, section 4.6. More detailed discussions on INEL seismic hazard assessment Volume 2, Appendix F-2. Additional detail on parameter selection and the incorporation into the seismic hazard assessment can be found in the Site-Specific Probabilistic Analysis for the Idaho National Engineering Laboratory (Draft). In keeping with the of CEQ, the EIS contains only enough information to support decisions required by the To reduce the bulk of the document, references are cited that contain the relevant Empirical evidence does not support the commentor's assertion that a moment magnitude on the ESRP is too low for adequate seismic hazard analysis of ESRP earthquake source scarps on the ESRP indicate that a seismic event with a moment magnitude 5.3 is the recorded in the rocks at the surface, which range in age from 1.2 million to 2,100 long-term geologic evidence with respect to the ESRP geologic record with which to of seismicity of the ESRP. The moment magnitude 5.3 estimate is mildly conservative earthquake magnitudes observed in similar tectonic environments and the assumed ins release. Further conservatism in the seismic hazard assessment cited in the EIS [S Probabilistic Seismic Hazard Analysis for the Idaho National Engineering Laboratory introduced through the use of a random ESRP earthquake, which has been assigned a m 5.5 to 6.0. The methods and data used in this study have been extensively reviewed earthquake is used to analyze the effects of seismic events related to structures to expression. Seismic hazards and geologic analyses can be found in Volume 1, section Appendix B, section 4.6; and Volume 2, section 4.6 and Appendix F-2. Seismically is discussed in Volume 2, section 5.14 and Appendix F-5. DOE takes seismic hazards as INEL uses independently reviewed analyses to support the implementation of DOE Order The accident analyses (including beyond reasonably foreseeable accidents with potential than seismically induced accidents) indicate that the risk to the public from INEL small. Therefore, additional information on reasonably foreseeable seismic events impact would have no effect on the decision-making process. No new analyses are required because, in accordance with NEPA (40 CFR 1502.22), the current credible scientific information relevant to understanding the existing environmental reasonably foreseeable impacts, and evaluating potential consequences. The EIS uses reviewed analyses available, and the evaluation of impacts is based on methods generally scientific community. See also the response to comment 05.05.01 (001).

## II COMMENT

The commentor expresses the opinion that the discussion of the Nevada Test Site is magnitude 5.6 earthquake that occurred near Little Skull Mountain on June 28, 1992, factored into the analysis.

### RESPONSE

The information in Volume 1 is an overview of the more detailed discussions contained in 1 appendices. In Volume 1, Appendix F, Part Two, section 4.6.3, the discussion on includes the Little Skull Mountain earthquake and the problems associated with recurrence

## II The commentor states that the New Madrid Seismic Zone is close enough to the reactor at the University of

Missouri to potentially cause damage should there be a large earthquake over magnitude

seismic assessment for Missouri is based on outdated information.

#### RESPONSE

Research reactors are typically built to Uniform Building Code (UBC) requirements to meet Nuclear Regulatory Commission (NRC) requirements for power reactors. Because detailed seismic analysis is not likely to affect the assessment of impacts of the no more seismic data are required in the EIS.

The data source for the research reactor at the University of Missouri was the documentation of the research reactor in 1961. In 1974, a thorough evaluation of the site vicinity was conducted for siting the Callaway commercial power reactor. The 1961 analysis is more appropriate than an analysis done specifically for another facility. The analysis which demonstrates a low potential for seismic activity.

## II COMMENT

The commentor states that the seismic wave attenuation characteristics of the eastern United States are adequately represented.

#### RESPONSE

The fact that strong-motion earthquakes are felt over wider regions of the eastern United States than in the western United States is considered in DOE site-specific seismicity analyses of eastern United States sites. Any new DOE construction required by a decision would meet the stringent seismic hazard characterization requirements and design criteria which would include a detailed assessment of seismic attenuation characteristics.

## II COMMENT

The commentor states that the EIS Glossary definition of seismicity is incorrect.

#### RESPONSE

A new definition of seismicity, which relates to the location, size, and rate of occurrence, has been included in the EIS Glossary.

## II COMMENT

The commentor questions Volume 2, Figure 4.6-4 with respect to the relative magnitudes of the seismic hazard curves describing ground motions at Idaho National Engineering Laboratory and Savannah River Site.

#### RESPONSE

The reasons for a seemingly inconsistent seismic hazard at SRS with respect to INEL are the low attenuation characteristics of eastern bedrock, which makes sites in the eastern United States more susceptible to larger ground motions resulting from low-to-moderate magnitude earthquakes in Quaternary sediments, which are appropriate for recording surface faulting earthquakes widespread in the east. Typically, Precambrian to Mesozoic rocks are overlain only by Quaternary sediments. Therefore, the number of late Quaternary surface faulting earthquakes in the eastern United States is uncertain, which results in conservative seismic hazard estimates. Accident analyses of reasonably foreseeable accidents with potential impacts greater than seismically induced impacts indicate that the risk to the public from DOE operations would be small. Therefore, impacts from reasonably foreseeable seismic events with lesser potential impact would have no significant impacts.

## II COMMENT

The commentor maintains that the seismic hazards at the Nevada Test Site are severe. The commentor states that the Nevada Test Site is in a high hazard area near faults that have experienced earthquakes triggered by other regional seismic events. Additional testing at the Nevada Test Site could have caused surface and subsurface failure levels.

#### RESPONSE

The discussion of seismicity at NTS (Volume 1, section 5.2.4, and Volume 1, Appendix B) indicates that the Nevada Test Site is in a high hazard area near faults that have experienced earthquakes triggered by other regional seismic events.

section 4.6) will be revised to indicate that a moderate seismic potential exists at a management site. As stated in the 1993 Nevada Test Site Technical Site Information DOE, the southern Nevada region is generally characterized as an area of moderate seismicity including the proposed SNF management site, is located in Seismic Zone 2B, as defined in the Building Code of the International Conference of Building Officials. Zone 2B signifies moderate damage potential. Areas further to the west (western Nevada and California) are in Seismic Zones 3 and 4. Seismic Zone 3 signifies areas with a major damage potential. Seismic Zone 4 signifies areas with a major damage potential and is located on the western edge of NTS. Seismic Zone 4 areas are well to the west of the site.

NTS has probably experienced earthquakes associated with regional seismic events. The region is oriented favorably for site seismicity to be influenced by other regional seismic events. Determining exact relationships between regional seismic events is difficult. Nuclear testing has produced fresh fault scarps and surface cracks, generally local to nuclear tests. Recent geologic mapping of NTS shows faults that have ruptured in the past, presumably as a result of testing. However, wave propagation from nuclear testing may relieve tectonic stress. The hypothesis regarding the triggering of local earthquake events is still being evaluated and tested in the scientific community and is the best hypothesis. Any new DOE facilities required by decisions supported by this EIS will be designed to meet the requirements of DOE Order 5480.28, Natural Phenomena Hazards Mitigation, which requires a rigorous, quantitative assessment and mitigation of natural phenomena hazards.

## II COMMENT

One commentor notes that the high seismic hazard in the vicinity of Idaho National Laboratory demands that DOE commit to an ongoing program of geologic hazards studies. Commentor asks how basalt flows will interact with nuclear waste and how the risks will be minimized.

### RESPONSE

Seismic hazards and geologic analyses can be found in Volume 1, section 4.2; Volume 2, section 4.6; and Volume 2, section 4.6 and Appendix F-2. Seismically induced accidents are discussed in Volume 2, section 5.14 and Appendix F-5. DOE takes seismic hazards very seriously, and has independently reviewed analyses to support appropriate implementation of DOE Orders. There has been an extensive effort over the past several years to upgrade DOE Order 5480.28 to natural phenomena hazards. Studies have been under way for many years and are continuing to ensure that seismic hazard characterization is based on up-to-date information and methods. New geologic information on seismic hazard characterization is reviewed and additional geologic studies are needed.

DOE has analyzed the effects of a hypothetical lava flow event at INEL. The geologic flow is discussed in Volume 2, section 4.6.4, and the estimated consequences of such various alternatives are shown in Volume 2, section 5.14, Tables 5.14-3, -5, -6, -8. The methodology used for performing these analyses is documented in Volume 2, Appendix F-2.1.2. Accident Assessments for the Idaho National Engineering Laboratory Facilities. As part of these analyses, DOE used conservative assumptions to account for the uncertainty in model results involving molten lava coming into contact with radioactive materials. The public would be small and well below DOE's Nuclear Safety Policy.

DOE has considered the potential for a volcanic ashfall event at INEL in Volume 2, Appendix F-2.1.2. As stated in section 4.6.4, potential ashfall events are not expected. The risk associated with an ashfall event is bounded by the accidents evaluated in the Hanford Site resulting from the Mount St. Helens eruption and as part of the Assessment of Potential Volcanic Hazards for New Production Reactor Site at the Idaho National Engineering Laboratory determined that hazards from volcanic events would be small. Therefore, a silicic ash-flow hazard at INEL does not represent a reasonably foreseeable impact on the human environment.

A hypothetical accident involving the instantaneous release of the contents of a high-level waste container represents the situation with the maximum reasonably foreseeable impact on the Snake River resulting from geologic conditions at INEL and is discussed in Volume 2, section 5.14. Under this scenario, maximum radionuclide concentrations are predicted to reach the Snake River years after the accident and predicted concentrations will be less than EPA MCLs or DOE Order 5480.28, Natural Phenomena Hazards Mitigation, sets forth DOE procedures for assessing and operating DOE facilities so that workers, the general public, and the environment are protected from the impacts of natural phenomena hazards on DOE facilities. This Order specifies that the assessment be reevaluated when there is any change in design and construction standards. EIS for INEL have undergone substantial safety analysis and seismic design review. Several

described in Volume 2, Appendix C of the EIS are proposed by DOE to replace or upgr INEL. Likewise, actions such as the transfer of fuels from potentially vulnerable facilities have resulted from the ongoing safety analysis and seismic design review. No new analyses are required for INEL facilities because the EIS summarizes existin evidence relevant to understanding the existing environment, identifying reasonably and evaluating potential consequences. The evaluation of impacts is based on metho by the scientific community.

See also the responses to comments 05.08.01 (014) and 05.08.01 (030).

## II COMMENT

Commentors note that the Idaho National Engineering Laboratory is designated a Unif Seismic Zone 2B and suggest that this area is not of low seismic potential as indic RESPONSE

The UBC seismic hazard zones range from 0 to 4, with 0 being designated the lowest. The Snake River Plain of Eastern Idaho is currently classified as Zone 2B, based on meetings of the professional engineering community. A small portion of the INEL s No INEL facilities are located in Zone 3. The characterization of DOE sites as hav seismic potential is correct when taken in the context of UBC Zone 4, which include intense seismic activity. In fact, the UBC accelerations are up to twice those sho Earthquake Hazard Reduction Program Maps for most of INEL. Likewise, United State Service ground motion maps (1982 and 1990) show accelerations lower than UBC values comparisons point out that the UBC maps are extremely conservative for INEL and tha hazard is less than shown on the UBC map.

DOE Order 5480.28, Natural Phenomena Hazards Mitigation, requires that DOE faciliti natural phenomena hazards mitigation requirements. The UBC design basis accelerati 0.2g (the acceleration due to gravity is 1g). Most INEL moderate- or high-hazard f are designed to a design basis acceleration of 0.24g or higher. Low-to-moderate se for INEL is further supported by the accelerations recorded at the site from the Bo which ranged from 0.078g to 0.017g. This earthquake had a moment magnitude of 6.9 magnitude of 7.3).

Regardless of the adjectival characterization of the seismic hazard at the DOE site DOE Orders require a systematic quantification of the seismic hazard for its facili probabilistic estimates of seismic hazards at other DOE sites have been used in the DOE has prepared, and INEL uses, an independently reviewed probabilistic seismic ha This study estimates earthquake ground motions and how often they might occur. Thi independently reviewed and will be incorporated into the INEL architectural and eng after review by the site Natural Phenomena Committee per DOE Order 6130.1A, General Included in this study is an estimate of ground motions at INEL facilities from a m earthquake occurring at the southern end of the Lemhi fault zone near the site boun motions exceed those that would occur as a result of moment magnitude 7.0 earthquak ends of the Lost River and Beaverhead fault zones. The 1983 Borah Peak earthquake magnitude 6.9. A study has also been performed for the Navy's Expanded Core Facili presents detailed data and comparable results. See also the response to comment 05 Quantitative estimates of seismic hazards at INEL sites are in or referenced in sec Volume 1 appendices; Volume 1, Appendix D, section 4.2 ; and Volume 2, Appendix B, These estimates are more useful than adjectival or UBC characterizations for the de

## II COMMENT

The commentor indicates that the EIS is inadequate because no seismic hazard zone m Specific reference was made to Volume 1, Appendix D, Part B.5.2 referring to seismi "zone maps" and that three of four waste water pits are not up to current earthquak the commentor states that facilities should be reconstructed to meet current codes of the Idaho National Engineering Laboratory with facility locations should be adde RESPONSE

Seismic hazards and geologic analyses for INEL can be found in Volume 1, section 4. Appendix B, section 4.6; and Volume 2, section 4.6 and Appendix F-2. Seismically i discussed in Volume 2, section 5.14 and Appendix F-5. DOE takes seismic hazards ve INEL uses reviewed analyses to support the implementation of DOE Orders and standar

Volume 1, Appendix D, Chapter 4 contains sections that describe possible seismic hazard at the site, provide general background information regarding the seismicity at these sites, and provide references for more detailed information. In addition, the current UBC seismic classification provided as a means for comparing the potential for seismic hazards among sites. The effects of seismic failure of Naval SNF management facilities have been evaluated in Volume 1, Appendix D, Chapter 5 and Attachment F provide summary and detailed discussions of the analyses that were performed and the public health risks that might result from a seismic event at a SNF would be stored. The seismic events considered in the analyses included both an earthquake magnitude used as the basis for the design of the facility (design basis earthquake magnitude, which is more severe than that for which the facility must be designed (earthquake)). These analyses show that the risks associated with seismic events involved are small for all of the alternatives and sites considered.

The three water pits that the commentor refers to were built to standards that were in effect at the time they were built. These water pits have been reevaluated under current standards and found to be structurally adequate. An existing facility's seismic strength and on the building's specific characteristics as well as the seismic acceleration. All bounds any seismically induced failure.

The information on seismic hazards used in this EIS was obtained from the available site. Because this information is specific to each site, it is more useful in understanding seismic hazards than the classifications provided for large regions in the UBC maps. The evaluation was completed for all of the water pools at the Expanded Core Facility to show that they all can withstand earthquakes for both design basis events (peak ground acceleration of 0.24 g) and for beyond design basis events (peak ground acceleration of 0.4 g). In Volume 1, Appendix D, Attachment B that three of the water pools were designed to the seismic classification in effect at the time they were built is correct, but does not mean they will withstand or that they do not comply with current building codes or other applicable DOE Order 5480.28, Natural Phenomena Hazards Mitigation, sets forth DOE policy to design and operate DOE facilities so that workers, the general public and the environment are not impacted by natural phenomena hazards on DOE facilities. This Order specifically requires that facilities be reevaluated when there is any change in design and construction standards. Existing facilities have undergone continual safety analysis and seismic design review. Several of the projects in Volume 2, Appendix C of the EIS are proposed by DOE to replace or upgrade facilities. Likewise, actions such as the transfer of fuels from potentially vulnerable facilities resulted from the ongoing safety analysis and seismic design reviews.

The data and methods used in the seismic hazard report referenced in Volume 2, section 4.6, Site-Specific Probabilistic Seismic Hazard Analysis for the Idaho National Engineering Laboratory (Draft) were extensively and independently reviewed. This report includes graphs of seismic occurrence versus acceleration for seismic events for each major facility at INEL. The data were incorporated into the INEL architectural and engineering standards after review by the Phenomena Committee. The previous INEL seismic analysis (Earthquake Strong Ground Motion Estimates for the Idaho National Engineering Laboratory: Final Report) was reviewed and incorporated into the site architectural and engineering standards after review by the Phenomena Committee in 1992 and is referenced in Volume 2, sections 4.6 and F-2 and provides location-specific seismic hazard information.

Most facilities currently in use at INEL are designed to withstand an earthquake acceleration of up to 0.2g. All of the facilities at the site lie in UBC Zone 2B, which requires that earthquake accelerations of up to 0.2g. A small portion of the INEL site lies in UBC Zone 1, which has no facilities in that portion of the site. DOE seismic design standards for moderate seismicity exceed the UBC seismic Zone 2B design criteria.

The EIS was prepared using existing references and currently published information. The EIS is in a layered fashion and placed much of the technical details in appendices and documentation. The references cited for Volume 1 and for Volume 2 include current environmental studies and applicable environmental consequences for all sites evaluated. Environmental studies are referenced in Chapter 9 of both volumes and are available in reading rooms at the INEL for review by the commentor and other interested members of the public. Low-to-moderate seismic potential for INEL is further supported by the acceleration from the Borah Peak earthquake, which ranged from 0.078g to 0.017g. This earthquake had a magnitude of 6.9 (surface magnitude of 7.3).

The EIS summarizes all known credible scientific evidence relevant to understanding and identifying reasonably foreseeable impacts, and evaluating potential consequences. The EIS includes up-to-date reviewed analyses when available, and the evaluation of impacts is based on the best available information accepted by the scientific community. The analyses reported in the EIS evaluate the potential consequences, including direct, indirect, cumulative, irreversible and irretrievable productivity losses.

## II COMMENT

The commentor states that the description of the Snake River Plain as having low seismicity by the Idaho National Engineering Laboratory's 1979 to 1981 Quarterly Seismic Reports summarize data on earthquakes "registered on or originated on the Snake River Plain."

### RESPONSE

The INEL Quarterly Seismic Reports cited by the commentor, available at the INEL Test Facility, show far fewer earthquakes originating on the Plain than recorded by INEL seismographs on the Plain. For example, the January 1982 report shows 470 earthquakes recorded by INEL on the Plain for the months October through December 1981 with magnitudes ranging from 0.1 to 1.3. Out of 470 earthquakes, only one event, with a magnitude of 1.1, was possibly located on the Snake River Plain. These reports typically show one to two events per quarter originating with magnitudes ranging from 0.1 to 1.3. When this data is compared with Figure 4.6-3, which describes the Snake River Plain as having a low-level of seismic activity with respect to the Province. The term "aseismic" has been avoided in the EIS to eliminate confusion. Seismic hazards and geologic analyses for INEL can be found in Volume 1, section 4.6, Appendix B, section 4.6; and Volume 2, section 4.6 and Appendix F-2. Seismically it is discussed in Volume 2, section 5.14 and Appendix F-5.

The assertion that the Snake River Plain has a low rate of seismicity is supported by Volume 2, Figure 4.6-3, which represents a summary of the best available data at the time the EIS was compiled. The addition of subsequent seismic events in the region would not change the conclusion that the Snake River Plain has a low rate of seismicity with respect to the Province.

See also the response to comment 05.05.01 (007).

## II COMMENT

The commentor notes that the Borah Peak earthquake was a magnitude 7.3 and not a magnitude 6.9 as stated in an EIS reference.

### RESPONSE

The Borah Peak earthquake, as stated in Volume 2, section 4.6.1, had a surface wave magnitude of 6.9. The moment magnitude for this earthquake was 6.9. Seismologists prefer to calculate earthquake energy in terms of moment magnitude because it is based on the physical and repeatable measurements (such as surface rupture length) as opposed to a surface wave magnitude which is a one-time measure of a seismograph's response to an earthquake. Other measurements (such as Richter) cannot be determined for close, large events due to instrument saturation.

## II COMMENT

The commentor asserts that the EIS statement that the Hanford Site is historically seismically inactive is incorrect.

### RESPONSE

The seismic hazards at the Hanford Site are described in Volume 1, section 4.1, and are provided in Volume 1, Appendix A, section 4.6.3. The area of the Hanford Site has experienced several moderate-sized earthquakes. The largest earthquakes near the Hanford Site are an approximate magnitude 4.5 event in 1918 near the town of Corfu, 35 kilometers (22 miles) from the Hanford Site, and a second event with the same approximate magnitude and location in 1971 near the location of the N-Reactor. The earthquake within the Hanford Site occurred in 1971 near the location of the N-Reactor and had a magnitude of 3.8.

DOE Orders require rigorous quantification of seismic hazards. Seismic hazard studies have been conducted at the Hanford Site to incorporate geologic estimates for the frequency of earthquakes associated with geologic faults and tectonic zones, as reported in Volume 1. The Hanford Site is in a UBC Zone 2B (Zone 0 represents low risk and Zone 4 represents high risk). This leads to design requirements to withstand moderate earthquakes.



## II COMMENT

The commentor states that the Idaho National Engineering Laboratory is subject to moderate seismic hazard and that other facilities at Puget Sound Naval Shipyard, the Hanford Site, Los Alamos Laboratory, and Sandia National Laboratories have moderate-to-high seismic potential.

**RESPONSE**  
Estimates of seismic hazards at the sites considered are in or are referenced in Volume 1, through F, and Volume 2, Appendix F-2. Quantitative estimates are more useful than characterizations for the decision-making process. However, the comment is acknowledged and rephrased the description of seismic hazard at DOE sites.

DOE Order 5480.28, Natural Phenomena Hazards Mitigation, requires that DOE facilities meet natural phenomena hazards mitigation requirements. The UBC design basis acceleration is 0.2g (the acceleration due to gravity is 1g). Most INEL moderate- or high-hazard facilities currently in use are designed or have been evaluated to a design basis acceleration of 0.2g. Low-to-moderate seismic hazard potential for INEL is further supported by the acceleration from the Borah Peak earthquake, which ranged from 0.078g to 0.017g. This earthquake had a moment magnitude 6.9 (surface wave magnitude 7.3).

Regardless of the adjectival characterization of the seismic hazard at DOE sites as moderate, Orders require a systematic quantification of the seismic hazard for its facilities. Estimates of seismic hazards at other DOE sites have been used in the EIS when available. The Operations Office is preparing and the EIS uses a probabilistic seismic hazard assessment for facilities. This study estimates earthquake ground motions and how often they occur. The study has been extensively and independently reviewed and will be incorporated into design and engineering standards after review by INEL Natural Phenomena Committee per DOE General Design Criteria. A similar process was used in 1992 to incorporate a scientific seismic analysis of INEL into INEL architectural and engineering standards. Included are estimates of accelerations at INEL facilities that would result from a moment magnitude 7.0 earthquake occurring at the southern end of the Lemhi fault zone near the INEL boundary. These accelerations exceed those that would occur as a result of moment magnitude 7.0 earthquakes at the southern ends of the Lost River and Beaverhead fault zones. The 1983 Borah Peak earthquake had a moment magnitude 6.9.

The Lemhi Fault and other seismic sources are discussed in Volume 1, Appendix B, and in Volume 2, section 4.6 and Appendix F-2. Seismically induced accidents are discussed in section 5.14 and Appendix F-5. These accident analyses indicate that the risks to INEL are small from seismic initiated events.

Existing facilities at INEL have undergone substantial safety analysis and seismic design. The projects described in Volume 2, Appendix C are proposed by DOE to replace or upgrade existing facilities at INEL. Likewise, actions such as the transfer of fuels from potentially vulnerable facilities have resulted from the ongoing safety analysis and seismic design review. All other major, moderate- and high-hazard facilities currently in use at INEL were designed to withstand accelerations from a moment magnitude 7.0 earthquake at the southern end of the zone. This level of seismic safety is consistent with requirements contained in DOE Order 5480.28. The accident analyses (including beyond reasonably foreseeable accidents with potential for release) indicate that the risk to the public from alternative accident scenarios would be small. Therefore, additional information or characterization of reasonable seismic events with lesser potential impact would have no effect on the decision-making process. Detail and characterization for seismic issues is appropriate for the programmatic EIS made based on this document.

## II COMMENT

The commentor questions why the overall level of seismic hazard calculated in the EIS for the Idaho National Engineering Laboratory is lower than the seismic hazard curves for either the Savannah River Site, and why U.S. Geological Survey data are not used.

**RESPONSE**

The differences perceived by the commentor result from the site-specific data and models used for seismic hazards. Each site used data and models judged to be appropriate to comply with standards for that location. Regardless of differences in modeling approaches, the professional and scientific integrity of these discussions and analyses for the programmatic EIS are adequate for evaluation and consideration of alternatives required for the programmatic EIS.

response to comment 05.05.01 (003).

DOE Order 5480.28, Natural Phenomena Hazard Mitigation, sets forth DOE procedures to assess, and operate DOE facilities so that workers, the general public, and the environment are protected from the impacts of natural phenomena hazards on DOE facilities. INEL uses analyses and implementation of DOE Orders and standards.

U.S. Geological Survey (USGS) data are regional in scope and do not provide sufficient analysis of the programmatic alternatives discussed in this EIS.

## II COMMENT

The commentor states that a design basis earthquake using a two-segment rupture and 7.0 earthquake on the Lemhi fault is not conservative enough.

### RESPONSE

The Lemhi fault and other seismic sources are discussed in Volume 1, Appendix B, section 4.6 and Appendix F-2. Seismically induced accidents are discussed in section 5.14 and Appendix F-5. These accident analyses indicate that risks to the environment from seismic-initiated events are low.

Existing facilities at INEL have undergone substantial safety analysis and seismic design. The projects described in Volume 2, Appendix C of the EIS are proposed by DOE to replace existing facilities at the site. Likewise, actions such as the transfer of fuels from obsolete facilities have resulted from the ongoing safety analysis and seismic design. All other major, moderate- and high-hazard facilities currently in use at INEL were designed to withstand accelerations that would result from a moment magnitude 7.0 earthquake at the south fault zone.

The probabilistic seismic hazard assessment is intended to capture the effects of rare high-intensity seismic events. Seismic events were the only identified common-cause potential to initiate radioactive and toxic material releases to the environment. Releases and impacts from individual facilities were considered in the identification of accident scenarios analyzed in this EIS. These results are conservative and ensure that the two-segment rupture model is consistent with observations to date on Basin and Range general and paleoseismic indicators near INEL in particular.

See also the response to comment 05.05.01 (001).

## II COMMENT

The commentor states that seismicity at the Idaho National Engineering Laboratory is not included in the EIS analysis.

### RESPONSE

Volume 1, Appendix B, section 4.6 and Volume 2, section 4.6 discuss seismicity in the region. Volume 2, section 5.14 discusses how seismic events were used in the accident analysis. Accident analyses, including seismicity assumptions, are found in Accident Assessment for the Idaho National Engineering Laboratory.

## II COMMENT

The commentor suggests that the seismic study in Volume 2, section 4.6 is incomplete because ground acceleration curves for facilities other than the Idaho Chemical Processing Plant are not included.

### RESPONSE

The data and methods used in the seismic hazard report referenced in Volume 2, section 4.6 include graphs showing rate of occurrence versus ground motion for seismic events at INEL. The seismic hazard curve for the Idaho Chemical Processing Plant was included in the information contained in the INEL seismic hazard analysis [Site-Specific Probabilistic Hazard Analysis for the Idaho National Engineering Laboratory (Draft)]. This report was incorporated into the INEL architectural and engineering standards after it was finalized by the INEL Natural Phenomena Committee. The previous INEL seismic analysis (Earthquake Shaking Motion Estimates for the Idaho National Engineering Laboratory: Final Report) was reviewed and incorporated into the INEL architectural and engineering standards after it was finalized.

Natural Phenomena Committee in 1992. Earthquake Strong Ground Motion Estimates for National Engineering Laboratory: Final Report is referenced in Volume 2, section

## II COMMENT

The commentor notes that the Idaho National Engineering Laboratory is in an area of specifically referred to the Beaverhead, Lemhi, and Lost River fault zones.

### RESPONSE

Seismic hazards and geologic analyses can be found in Volume 1, section 4.6; Volume section 4.6; and Volume 2, section 4.6 and Appendix F-2. Seismically induced accid in Volume 2, section 5.14 and Appendix F-5. DOE takes seismic hazards very serious independently reviewed analyses to support the enforcement and implementation of DO standards.

DOE Order 5480.28, National Phenomena Hazards Mitigation, sets forth DOE procedures assess, and operate DOE facilities so that workers, the general public, and the env from the impacts of natural phenomena hazards on DOE facilities. This Order specif facilities to be reevaluated when there is any change in design and construction st facilities at INEL have undergone continual safety analysis and seismic design revi projects described in Volume 2, Appendix C of the EIS are proposed by DOE to replac facilities at the site. Likewise, actions such as the transfer of fuels from poten modern facilities have resulted from the ongoing safety analysis and seismic design INEL has prepared a probabilistic seismic hazard assessment for facilities at the s estimates earthquake accelerations and how often they might occur at facilities at been independently reviewed and will be incorporated into the INEL architectural an standards after it is finalized and reviewed by the site Natural Phenomena Committe conjunction with DOE Orders to design and build new facilities. Included in this s ground motions at INEL facilities that would result from a magnitude 7.0 earthquake southern end of the Lemhi fault zone near the site boundary. These ground motions that would occur as a result of magnitude 7.0 earthquakes at the southern ends of t Beaverhead fault zones.

Accident analysis results (including seismically induced accidents) indicate that t INEL operations would be small.

Major facilities currently in use at INEL were built to withstand accelerations tha earthquake at the southern end of the Lemhi fault zone.

## II COMMENT

The commentor notes that the West Valley Demonstration Project facility is only abo the probable causative structure for the 1929 Attica, New York, magnitude 5.8 earth RESPONSE

Volume 1, Appendix E, section 3.3.1 of the EIS has been revised to include seismic significance of seismic activity in the West Valley region.

## II COMMENT

The commentor expresses the opinion that the geologic map in Volume 2, section 4.6 because it does not define certain major geologic features; specifically, the Arco Lava Ridge-Hell's Half Acre Volcanic Rift Zone, and the Axial Volcanic Zone.

### RESPONSE

Rift zones at INEL, as discussed in the EIS, refer to volcanic rift zones in the re suggested by the commentor concerns continental or oceanic constructive tectonic pl while correct, is not appropriate with respect to local conditions.

Important regional geologic features are included in Volume 2, section 4.6. A map significant volcanic rift zones in and near INEL can be found in the Engineering De Water Resources Supporting Document for the INEL Environmental Restoration and Wast Management EIS (Draft) in Volume 2, Appendix F. Many geologic maps of INEL and ad available in the open literature. Some of this literature is cited and referenced i including USGS reports and maps.

DOE added a more detailed geologic map of INEL to the EIS.

## II COMMENT

The commentor suggests that analysis of seismic and volcanic hazards be fully reviewed by the Geological Survey and other qualified experts.

### RESPONSE

Consistent with DOE Orders and standards, INEL seismic hazards assessments and methods have been independently reviewed by many expert seismologists and geologists. These include the National Events Review Group, a panel of seismic, geologic and structural engineering experts; the National Seismic Siting and Design of High-Hazard Facilities; the Lawrence Livermore National Laboratory Defense Nuclear Facilities Safety Board; Woodward-Clyde, Inc.; Risk Engineering, Inc.; University of Utah; State University of New York at Binghamton; Southern California State University; Idaho State University; the U.S. Geological Survey; and Boise State University. Because of the extensive nature of this review, DOE believes additional review is not necessary. See also the response to comment 05.05 (015).

## II COMMENT

The commentor points out that the Uniform Building Code contains four Seismic Risk Categories.

### RESPONSE

The EIS has been revised to reflect that there are more than three Uniform Building Code Seismic Risk Categories.

## II 5.6 Land Use

## II COMMENT

The commentor notes that the list of Federal outdoor recreation facilities in Volume Three, section 4.2 should be expanded, and Figure 4.2-2 should be updated.

### RESPONSE

The list of Federal outdoor recreation facilities identified in the text and figure in Part Three, section 4.2 is not intended to be all inclusive. However, the list of recreation facilities has been revised to include other major facilities.

## II COMMENT

The commentor, referring to Volume 1, Appendix F, notes that the acreage needed for the Central and Expanded Core Facilities, whether 90 or 120 acres, is unclear.

### RESPONSE

Construction of SNF management facilities would require 90 acres. Under the Central and Expanded Core Facility would also need to be constructed; this would require an additional 30 acres. The data in Volume 1, Appendix F, Parts Two and Three, Table 3.2-1 for the Central and Expanded Core Facilities include the requirements of the Expanded Core Facility, which are discussed in Volume 1 to clarify the acreage requirements, a footnote has been added to Volume 1, Appendix Three, Table 3.2-1, and the text of Volume 1, Appendix F, section 3.2 has been revised.

## II COMMENT

The commentor supports the banning of grazing on Idaho National Engineering Laboratory land and the re-establishment of natural vegetation.

### RESPONSE

Grazing policies are not within the scope of this EIS. The U.S. Department of the Interior is responsible for those policies.

## II COMMENT

The commentor states that the EIS land-use analysis does not identify policies or process, or provide an opportunity for public input on specific projects.

### RESPONSE

The EIS identifies DOE land-use plans and policies applicable to INEL in Volume 2, land-use policies are also identified in Volume 2, section 4.2. For details of the commentor is encouraged to consult the specific documents referenced in the EIS, where reading rooms and information locations listed in the EIS. Also, DOE has established an Office, which is identifying stakeholder-preferred future use options at the 25 DOE sites by 1995. Future use options are defined as a select range of preferred uses forged with stakeholder desires and DOE missions, and tempered by technical, and legal constraints.

## II COMMENT

The commentor requests an explanation of how percentages were calculated for acres of National Engineering Laboratory under each alternative.

### RESPONSE

Calculations of the acreage that would be disturbed by proposed projects under each alternative based on figures contained in individual project data sheets found in Volume 2, Appendix 3.3 has been changed to show how the acreages disturbed were calculated.

## II COMMENT

The commentor states that the EIS fails to consider impacts of the alternatives on Idaho National Engineering Laboratory land such as hunting, grazing, and tribal ceremonial purposes.

### RESPONSE

Volume 2, section 4.2 identifies the portions of INEL that are used for hunting and section 4.4 discusses traditional resources that are of cultural or religious importance to Native Americans. All of these land uses are outside of the facility areas where the proposed various EIS alternatives would be implemented. Consequently, no impacts to hunting nor to tribal ceremonial or religious uses, are expected. The future use of land will be discussed with local Native Americans to assess any potential impacts of future proposed activities.

## II COMMENT

The commentor requests that the EIS describe and identify the locations of specific land-use decisions under the Federal Facilities Agreement and Consent Order, identify the role of regulatory agencies in making future land-use decisions under the Agreement and Consent Order for Idaho National Engineering Laboratory.

### RESPONSE

The specific location of proposed actions at INEL are identified in the project summary Appendix C. The number of acres disturbed for each project is also provided in this Appendix. The locations of projects not covered by this EIS will be identified in subsequent Environmental Response, Compensation, and Liability Act (CERCLA) documents.

The Federal Facilities Agreement and Consent Order (FFA/CO) process does not entail "decisions." Rather, assumptions for future land uses at INEL will be made for the appropriate level of cleanup at each operable unit. In August, 1994, the DOE issued for public comment the Idaho National Engineering Laboratory Long-Term Land Use Scenarios. This document set forth various land-use scenarios that could be assumed for long-term activities at INEL. Public comments on the document were received, and are being reviewed and addressed as appropriate.

In accordance with CERCLA and the FFA/CO, the Idaho Department of Health and Welfare Region X will be part of the decision-making process on the appropriate level of cleanup.

requested comments on the Idaho National Engineering Laboratory Long-Term Land Use Scenarios from the State of Idaho and EPA Region X.

## II COMMENT

The commentor states that the EIS needs to address whether the impacts from land use National Engineering Laboratory are permanent or temporary.

### RESPONSE

Volume 2, section 5.18 states that disposal of radioactive or hazardous wastes would be irretrievable (i.e., permanent) commitments of land resources under the Ten-Year Pl Treatment, Storage, and Disposal alternatives. The affected acreage is also identified for waste treatment, storage, and disposal activities would be reserved for those purposes. This land would be precluded during the time period addressed by the EIS.

## II COMMENT

The commentor states that the proposed placement of spent nuclear fuel facilities would be inconsistent with the DOE 1994 draft future land use plan for the Nevada Test Site which designates that portion of Area 5 as a "nonnuclear test area."

### RESPONSE

The NTS future land use plan has three area designations: underground nuclear weapon repository, proposed high-level radioactive waste repository area, and nonnuclear test area. The plan is broad, providing general guidance for future activities. The underground nuclear weapon repository area has general characteristics suitable for nuclear weapons tests, although some localized areas are unsuitable because of terrain, previous uses, local geologic features, or other reasons. The proposed high-level radioactive waste area has been reserved to support site characterization at Yucca Mountain, and is not available for other uses at this test area. This area is an area where weapons testing is not conducted and is available for other uses appropriate by DOE, such as siting SNF management facilities.

## II COMMENT

The commentor states that DOE has summarily dismissed the alternative of restoring the Idaho National Engineering Laboratory to pristine conditions as unreasonable and that DOE is ignoring the site. Additionally, the commentor states that the presence of cultural resources on the Idaho National Engineering Laboratory should qualify the Idaho National Engineering Laboratory as a historic resource, thereby requiring restoration of the site.

### RESPONSE

Environmental restoration activities at INEL are being conducted in accordance with the DOE Order of December 4, 1991. Restoration activities will comply with the requirements of CERCLA and the National Environmental Policy Act, but are designed to assure protection of the environment in a cost-effective manner.

## II COMMENT

The commentor states that the analysis of land-use impacts is fundamentally flawed because "there are no Native American treaty rights that would affect any future land use on the INEL site." The commentor states that the Fort Bridger Treaty expressly reserves the rights of the Tribes to use unoccupied lands of the U.S., and the Tribes will exercise these rights if the Idaho National Engineering Laboratory goes away or releases portions of land.

### RESPONSE

The commentor is correct that the Fort Bridger Treaty of 1869 reserves certain future lands to the Shoshone-Bannock Tribes to the extent that those lands are not occupied. The analysis of land-use impacts in the EIS is limited to the scope of the EIS. The time period for Volume 2 analysis is the 10 years from June 1995 to 2005; the time period for Volume 1 analysis is 40 years, with detailed impact analysis for the first 10 years.

actions occurring from June 1, 1995 to June 1, 2005. During the time periods covered does not plan to relinquish ownership and control of the INEL site. Discussions of 1869 in Volume 1, Appendix B and Volume 2 of the EIS have been changed to more closely address this issue.

## II COMMENT

The commentor requests information be included in the EIS on the approach related to that which would be used to transfer Idaho National Engineering Laboratory land to other sector and DOE's and other agencies' responsibilities in the land transfer process.

**RESPONSE**  
The lands and facilities that are evaluated under the alternatives in this EIS are transferred to other government agencies or the private sector within the time considered. Consequently, the subject of transfer of government lands to other government agency sector is outside the scope of this EIS.

## II COMMENT

The commentor objects to a land-use scenario projected by a draft DOE Idaho Operations and states that Idaho National Engineering Laboratory lands should remain as wildlife and not be returned to the public for uses such as housing.

**RESPONSE**  
This is in reference to a draft document entitled Long-Term Land Use Future Scenarios at Idaho National Engineering Laboratory. The purpose of the land use scenarios document is to provide decisions regarding environmental restoration activities at INEL by projecting reasonable scenarios for the next 100 years. The current land use status, that is, Federal Government land at INEL, would not change under any of the alternatives analyzed in the EIS.

## II 5.7 Utilities and Infrastructure

## II COMMENT

The commentor asks whether recycling and the use of lined evaporation ponds have resulted in an increase or decrease in net consumptive water use at the Idaho National Engineering Laboratory.

**RESPONSE**  
Currently, there are no major water recycling projects at INEL. Consumptive water use has probably increased since the use of lined evaporation ponds because water no longer infiltrates the ground. No studies have quantitatively evaluated the magnitude of increase since switching to lined ponds. However, it is likely that the increase is small with respect to total water use at INEL.

## II COMMENT

The commentor questions why the electrical usage rate at the Idaho National Engineering Laboratory is expected to decline.

**RESPONSE**  
Volume 1, Appendix B, section 4.13 describes the 1995 baseline electrical usage at INEL. It is expected to decline when Navy prototype training at the Naval Reactors Facility is completed.

## II COMMENT

The commentor notes that only sanitary waste water discharges are reported in Volume 1, section 4.13.4, and that additional waste water discharges from specific projects will be discharged to the River Plain aquifer. The commentor asserts that the EIS seriously underestimates the potential impacts of these discharges.

water discharge from 1989 through 1991, based on a comparison of discharges reported (537 million liters per year) with those reported in INEL Nonradiological Waste Management Information System (6.8 billion liters/year). The commentor asks how this difference whether this will impact the analysis of impacts on the aquifer.

**RESPONSE**

As used in Volume 1, Appendix B, section 4.13.4, the term "waste water" refers primarily to wastes. DOE has clarified this in the EIS. As noted in Volume 1, Appendix B, section 4.13.4, withdrawal from the aquifer by INEL is approximately  $1.9 \times 10^9$  gallons per year. Only a substantial portion is discharged to the surface and is eventually returned to the aquifer as presented in the EIS. Because of the small percentage of water consumed with rights, and volume of water in the aquifer under INEL, there would be a small impact on the aquifer under all alternatives considered.

## II COMMENT

The commentor identifies a discrepancy in terminology between sections regarding the Laboratory water rights.

**RESPONSE**

Volume 2, section 4.13 has been changed to refer to INEL water rights as a Federal

## II COMMENT

The commentor would like Volume 2, section 5.1.3 to clarify whether projected waste is limited to sewage.

**RESPONSE**

This discussion in section 5.13 has been modified as requested.

## II 5.8 Water Resources

## II COMMENT

The commentor states that the discussion in Volume 2 concentrates on radioactive and nonradioactive effluents.

**RESPONSE**

Contaminants, including nonradioactive contaminants, are discussed in Volume 2, section 5.8. Nonradioactive contaminants at INEL were included in the analysis process performed (Predicted Consequences on the Snake River Plain Aquifer of Alternative Actions 1 and 2). A screening identified just three analytes, all radionuclides, with plumes above current levels. These contaminants were selected for detailed analysis of potential consequences on the Snake River Plain aquifer and are the main constituents within the contaminant plumes. In addition, nonradioactive contaminants, are discussed in Volume 2, section 5.8.

## II COMMENT

The commentor suggests that there be more information on expected constituents and waste streams for proposed actions at the Idaho National Engineering Laboratory in the EIS. The commentor expresses the opinion that a decision of "no impact" can be inadequately characterized waste streams or source terms.

**RESPONSE**

Anticipated projects have been included in the EIS to present readers with as comprehensive a picture of forthcoming projects as is currently possible. These anticipated projects have been evaluated to attempt to bound reasonably foreseeable environmental impacts from such projects. Information on secondary waste generation would be available for an assessment of impacts management. NEPA status of environmental restoration and waste management projects



INEL is discussed in the Summary (see box titled Projects Related to Alternatives i of the Summary, and in Volume 2, Table 3.1-1.)

## II COMMENT

The commentor notes that data exist that indicate other contaminants in perched wat Waste Management Complex, Test Reactor Area, Idaho Chemical Processing Plant, and T have been detected in perched water zones, and that these data should have been inc Appendix B, section 4.8.2.

### RESPONSE

The EIS has been changed to address the comment by indicating the presence of other have been identified in the perched water at INEL.

## II COMMENT

The commentor states that in the Oak Ridge Reservation discussion, 914 meters (3,00 is inappropriately represented as being close to the source.

### RESPONSE

The discussion of water resources for ORR in Volume 1, Appendix F, Part Three, sect revised.

## II COMMENT

Commentors suggest addition of the location where Las Vegas currently gets its wate to the discussion on the Nevada Test Site in Volume 1 of the EIS.

### RESPONSE

Water use at NTS will not impact Las Vegas water use because NTS obtains its water groundwater basin that is separate from the Las Vegas groundwater basin. Additiona 70 to 80 percent of its water from the Colorado River. Volume 1, Appendix F, Part to more accurately reflect where Las Vegas gets its water.

## II COMMENT

A commentor states there is a need to clarify the assumption regarding the spent nu supply from the Area 5 wells and distribution system at the Nevada Test Site. A co that the increased use of an aquifer currently in overdraft should constitute a sig effect, regardless of the user's right to that water.

### RESPONSE

As indicated in Volume 1, Appendix F, Part Two, section 5.13, the water wells and p Area 5 of the NTS have sufficient capacity to meet the requirements for the propose proposed facility location is in the vicinity of the Area 5 water lines. Therefore infrastructure would be adequate to supply SNF facility water.

The commentor correctly states that water rights should not be a factor in the dete significance of groundwater use impacts, and in fact, those water rights given to t the area of NTS were not considered in the impact determination made in the EIS. T Federal water rights was included in the EIS for information purposes only.

The discussion of groundwater quantity issues in Volume 1, Appendix F, Part Two, se revised to include a more comprehensive analysis of potential impacts on groundwa the estimated perennial yield of the Frenchman Flat subbasin has been exceeded for with no decline in static water levels, it is likely that increased water use for S sustained. The overall impact of any groundwater withdrawal in Frenchman Flats is discharge in the deserts to the southwest of NTS. SNF operations would decrease th by 0.04 percent of the approximated 1992 discharge; therefore, impacts to groundwa be small from SNF operations. More detailed analysis, such as that proposed by th be done if the NTS were chosen as a site for SNF management activities.

## II II COMMENT

Commentors state that a summary table of water used and water consumed be provided alternative, as well as a discussion of impacts in Volume 1, Appendix B, section 5. RESPONSE

Volume 1, Appendix B, section 5.8 discusses the alternative that would represent the use/consumption and provides water consumption in both gallons and cubic meters. In the greatest projected water use is shown to have a small impact on the aquifer, the likewise be small. There is additional detail in Volume 2, section 5.8.

## II COMMENT

The commentor states that reference should be made to the increased consumption of National Engineering Laboratory as a result of the alternatives analyzed. RESPONSE

The use of groundwater by the alternatives analyzed in the EIS for INEL is discussed in section 5.8 and Appendix C. In general, increased construction activity and new facilities result in a net increase in consumptive water use. The maximum increase in net consumptive water use for any alternative is expected to be less than 5 percent of current water use at INEL. This is to reflect more accurate water use estimates.

## II COMMENT

The commentor discusses the use of the term "aquitard" in Volume 1, Appendix F, Part 2 to describe certain geologic units on the Oak Ridge Reservation. The commentor notes that published reports by State of Tennessee geologists, all the geologic units underlying the Reservation were referred to as "aquifers" and it was stated that sufficient water was usually obtained from wells at depths of 18 meters (60 feet) or less in the Conasauga group, notably the Pumpkin Valley shale unit of the Conasauga Group, were noted to be poor. RESPONSE

An aquifer is a body of rock or sediment in a formation, group of formations, or part of a formation that is saturated and sufficiently permeable to transmit economic quantities of water to wells or springs. An aquitard, on the other hand, is a confining bed that will tend to retard, but does not prevent, the flow of water to or from an adjacent aquifer. It may serve as a storage unit, but will not transmit water. The Geology Resources and Water Resources sections of the EIS were prepared from recently published material. No site-specific field study was conducted. Recently, there are several formations beneath ORR with varying ability to store and transmit water. It is agreed that the Pumpkin Valley Shale could very well be referred to as an aquitard. It has been shown to have poor transmissivity capabilities. Recently published reports, "Report: A Hydrologic Framework for the Oak Ridge Reservation, and Status Report on the Oak Ridge Reservation" have all used the term "aquitard" to describe the Pumpkin Valley Shale and the other geologic units beneath the ORR.

Pumpkin Valley Shale is the oldest of six formations within the Conasauga Group and is part of the group. No site-specific data are available to determine at what depth Pumpkin Valley Shale is at the West Bear Creek Valley site. It is logical, however, to think that at depth below the site, the water-bearing unit most likely to be encountered would be an aquitard. If the ORR is chosen as a site for new SNF management facilities, water and groundwater studies would be performed to identify and characterize the site. The level of detail desired by the commentor for the data analysis is not appropriate for this programmatic document, and would not provide any information to the decisionmakers. This broad environmental review document has been prepared in accordance with the provisions of NEPA and CEQ implementing regulations, which allow for a broad focus on the subject of the decision. Additional, more specific data, such as that proposed to be provided, if necessary, in further site-specific environmental documents. Geology and water resources for ORR are discussed in the EIS in Volume 1, Appendix B, sections 4.6 and 4.8.

## II COMMENT

Commentors state that the EIS treats the complex fracture flow system in the clastic system of carbonate rocks of the Oak Ridge Reservation simplistically, that the system modeled, and that the system is not well enough understood to support the broad conclusion that groundwater in the "aquitards" is essentially static or that these units are able to respond.

DOE agrees that the ORR groundwater system is complex. It is difficult to characterize highly fractured and folded complex geologic settings. However, a full and detailed complex fracture-contaminated flow processes on the ORR is beyond the scope of this EIS. The EIS description and analysis of hydrologic conditions at ORR was developed from hydrologic literature, including Status Report: A Hydrologic Framework for the Oak Ridge Reservation. Based on these sources, the EIS analysis of potential groundwater impacts of SNF storage assume that the aquitards "contain contaminants," but rather that these units are composed of short-flow paths and that solute residence times increase sharply with depth. In the intervals, estimates of residence times from carbon 14 measurements and modeling are thousands of years as stated in Status Report: A Hydrologic Framework for the Oak Ridge Reservation. Volume 1, Appendix F, Part Three, section 5.8.4 has been revised to more accurately reflect the EIS discussion of potential groundwater quality impacts.

Very little potential exists for contamination of the Knox aquifer from the operating management facilities. These facilities would be constructed using technologies that include containment, leak detection, and water-balance monitoring equipment. Therefore, no environmental consequences related to water resources are anticipated from the operating management facilities.

A detailed description of groundwater flow would require an in-depth site-specific hydrogeology study. If ORR is selected as a site for new SNF management facilities, a study will be performed.

Geology and water resources for ORR are discussed in the EIS in Volume 1, Appendix sections 4.6 and 4.8.

## II COMMENT

The commentor states that karst features at the Oak Ridge Reservation (e.g., sinkholes, etc.), exist in certain geologic units within the Conasauga and Chickamauga Groups, and that aquifers within those units are karstic.

### RESPONSE

This comment is addressed by statements included in the EIS, Volume 1, Appendix F, section 4.6. The EIS states that karst development is present to varying degrees in the Conasauga Group, most notably in the Maynardville Limestone, part of the Knox aquifer. It states that "Although no site-specific geologic characterization has been conducted at the Valley site, it appears the proposed Spent Nuclear Fuel Management Facility is located in the Conasauga Group strata not normally characterized by karst development." Site-specific hydrogeologic investigations would be necessary to verify this if ORR is chosen as a site for new management facilities.

## II COMMENT

Commentors state they are concerned with the high cost to owners/operators of private systems to conduct water quality testing due to the potential impact of past, present, and future management activities on the Snake River Plain aquifer.

### RESPONSE

Independent assessments of the Snake River Plain aquifer water quality at INEL confirm environmental monitoring results that indicate that no contaminants in concentrations above DOE DCGs exist beyond the INEL boundary. With improved management practices and remediation efforts planned or under way, it is likely that water quality in the Snake River Plain will continue to improve. Therefore, there is no INEL-related cost to local water resources outside the INEL boundary, because independent assessments indicate that aquifer contamination outside the INEL boundary is small with respect to EPA MCLs.

## II COMMENT

The commentor states that the potential exists for a deeper, more active flow regime at the Oak Ridge Reservation.

The commentor states that it is erroneous to dismiss the possibility of deep contaminant groundwater at the Oak Ridge Reservation, suggesting that the reason there is little contaminant transfer is that there is little data on the deep aquifer.

### RESPONSE

Information provided in Volume 1, Appendix F, Part Three, section 4.8 was developed from published hydrologic literature on the ORR including Status Report: A Hydrologic Framework for the Oak Ridge Reservation and recent site environmental reports. For the purpose of this information was beyond that which would be necessary to understand the effects of the ORR is chosen as a site for new SNF management facilities, site-specific groundwater monitoring was performed.

The EIS discussion of groundwater conditions at ORR and the EIS analysis of potential impacts, including the statement that there is little deep groundwater flow in the aquitards, were based on information and analysis in published hydrologic literature (Volume 1, Appendix F, Part Three, section 4.8 and references cited there.) These do not dismiss the possibility of deep flow, but state that water budget analyses and observed groundwater flow and near-surface conditions indicate that almost all groundwater flow is near the ground surface.

Geology and water resources for ORR are discussed in the EIS in Volume 1, Appendix F, sections 4.6 and 4.8.

## II COMMENT

The commentor notes that vadose zone conductivity values derived from slug tests at the Oak Ridge Reservation may be understated in the EIS.

### RESPONSE

It is true that smearing of clays by the drill bit during well installations, and over pumping during testing, could reveal conductivity values less than what actually exist in nature. This is noted in the Water Resources section of the EIS were obtained from Status Report: A Hydrologic Framework for the Oak Ridge Reservation. This reference cites that saturated hydraulic conductivity measurements were in fact conducted using infiltration tests as well as packer test results. Geology and water resources for the Oak Ridge Reservation are discussed in the EIS in Volume 1, Appendix F, Part Three, sections 4.6 and 4.8.

## II COMMENT

The commentor states that the discussion in Volume 1, Appendix B, section 4.8 on perched aquifer is incorrect. Perching layers are impermeable, not impervious, and so do not occur and impact the aquifer.

### RESPONSE

Perching layers are relatively impermeable. While some small amount of water may pass through an impermeable layer, the main flow is lateral until the edges of the impermeable bed are reached, then it continues downward. The section of the EIS cited by the commentor accurately describes the flow of water around and through these impermeable layers in the Snake River Plain aquifer.

## II COMMENT

The commentor notes that the likely source of nitrates detected in springs that flow through limestone is the Conasauga Shales at the Oak Ridge Reservation. This contamination is due to the inability of the shales to keep contaminants from migrating to the Knox aquifer.

### RESPONSE

Most of the Y-12 Plant is underlain by units included in the Conasauga aquitard. However,

Maynardville limestone (Knox aquifer) also underlies a portion of the Y-12 Plant. properties of these rock units, proposed SNF management facilities are designed to of waste water with hazardous chemical or radiological characteristics. These faci constructed using technologies that include secondary containment, leak detection, monitoring equipment. Therefore, no significant environmental consequences related are anticipated from the operation of SNF management facilities. Detailed analyses of existing contaminant sources and transport pathways are beyond If ORR is selected for new SNF management facilities, site-specific groundwater st performed. The level of detail desired by the commentor for the data analysis is n decisions that will be made based on this programmatic document, and would not prov that would assist decisionmakers. This broad environmental review document has bee accordance with the provisions of NEPA and CEQ implementing regulations that allow broad focus on issues related to the subject of the decision. More specific data, the commentor, would be provided, if necessary, in further site-specific environmen Geology and water resources for ORR are discussed in Volume 1, Appendix F, Part Thr 4.8.

## II COMMENT

Commentors discuss the porous nature of the Eastern Snake River Plain and the poten present, or future DOE activities related to spent nuclear fuel management at the I Engineering Laboratory on water quality of the Snake River Plain aquifer.

### RESPONSE

Water resources at INEL and impacts resulting from SNF alternatives are described i B, sections 4.8 and 5.8. There would be no significant impacts to the aquifer unde Environmental monitoring shows that INEL operations have not contaminated the Snake aquifer above EPA limits beyond the INEL boundaries. Liquid effluent monitoring an containment construction would limit operational releases from a new facility to ne modeling using assumptions, including scientifically defensible assumptions regardi increase the potential impacts to the aquifer from past, present, and future activi show that groundwater quality will not be significantly impacted, because radioacti contaminant discharges to the soil or aquifer would not occur in concentrations abo DCGs. Furthermore, it is likely that overall aquifer water quality will continue t regardless of the EIS alternative chosen for SNF management.

Water resources and impacts from all waste management and environmental restoration alternatives, considered for the INEL are described in Volume 2, sections 4.8 and 5 all the alternatives considered, the possible future sources of contamination would previous practices. This would be a result of waste management practices that incl discharge monitoring, as well as natural contaminant attenuation and radioactive de releases. Computer groundwater modeling using conservative parameters (discussed i Appendix F) indicates that existing contaminant plumes within the INEL boundary wou decrease at least through 2035. The modeling further indicates that overall aquife would actually improve in that period and probably continue to improve after 2035.

A hypothetical accident involving the instantaneous release of the contents of a hi to a once-every-50,000-years seismic event represents the situation with the most p Snake River Plain aquifer and is discussed in Volume 2, section 5.14 and Appendix F scenario, maximum radionuclide concentrations are predicted to reach the INEL bound concentrations less than EPA MCLs or DOE DCGs 300 years after the accident.

Independent assessments of the Snake River Plain aquifer water quality at INEL conf environmental monitoring results that indicate that no contaminants in concentratio DOE DCGs exist beyond the INEL boundary. With improved management practices and re efforts planned or under way, it is likely that overall water quality in the Snake the INEL will continue to improve.

As stated in Volume 2 Appendix F-2, the effects of porosity have been accounted for described. The analysis shows that for all alternatives considered, impacts would

## II COMMENT

The commentor states the need for accuracy in modeling impacts of Idaho National En waste management activities on the Snake River Plain aquifer.

**RESPONSE**

A description of water resources and potential environmental consequences to water including the Snake River Plain aquifer, is discussed in Volumes 1 and 2, sections analysis performed for the EIS integrated available data and technical information to evaluate contaminant transport and predict future trends in aquifer water quality was completed through 2035 to add assurance to the conclusions reached in the document concludes that overall aquifer water quality would actually improve over this period methodology and assumptions used for the computer modeling effort is in Volume 2, A

**II COMMENT**

The commentor suggests that the reburial of plutonium in Pit 9 will pose a threat to the aquifer.

**RESPONSE**

According to page 13 of the Pit 9 Demonstration Record of Decision (ROD), plutonium Subsurface Disposal Area, but not in interbeds 9 meters (30 feet) or 73 meters (240 surface). The presence of plutonium in the 34-meter (110-foot) sediment layer has been attributed to flooding of the Subsurface Disposal Area in 1969 from rapid thawing of flooding is now prevented by a 5-meter (15-foot) dike around the Subsurface Disposal Area. Transport modeling was conducted for the less than 10 nanocuries per gram transuranics to be left in or returned to Pit 9 to evaluate potential contaminant migration to the aquifer. Modeling results indicated that the Safe Drinking Water Act standard of 15 picocuries per liter alpha radioactivity will not be exceeded anywhere in the Snake River Plain aquifer layer of clean soil with a linear absorption coefficient of at least 500 milliliters per gram bottom of the pit and if the pit is backfilled to grade with clean soil. The Pit 9 the Pit 9 Administrative Record evaluated human health risks from 10 nanocuries per gram residuals left in the pit after cleanup. Modeling of radionuclide transport to the aquifer indicates that no migration to the aquifer is expected within 1,000 years. Residuals will be reevaluated in the baseline risk assessment to be performed as part of the

**II COMMENT**

The commentor states that Volume 1, Appendix B, Table 4.8-1 should include actual background levels and asks if groundwater includes the vadose zone, perched water, and aquifer.

**RESPONSE**

Table 4.8-1 did not include the detection limits and background values because this would complicate the table. The point being made by the table is that recent conditions are within background levels and detection limits. Detection limits and background level references in Volume 1, Appendix B, Table 4.8-1 and references in section 4.8. Groundwater, and the vadose zone are discussed separately in the EIS.

Volume 1, Appendix B, Table 4.8-1 specifically refers to groundwater quality in the aquifer. As discussed and defined in the EIS, locally saturated conditions above the perched water, while groundwater refers to usable quantities of water within an aquifer in the vadose zone is referred to as vadose water. Because perched water occurs within the vadose zone.

**II COMMENT**

The commentor notes that discussions in Volume 1, Appendix B, section 4.8 should compare aquifer conditions with both Environmental Protection Agency existing and proposed standards, and that proposed maximum contaminant levels are not appropriate for the quality in Volume 1, section 4.2 of the EIS.

**RESPONSE**

A comparison of each contaminant with existing EPA MCLs with proposed MCLs is in Volume 1, Appendix B, Table 4.8-1. The summary material in Volume 1 has been enhanced to compare contaminant levels, where established, with existing EPA MCLs.

For americium-241, plutonium-238, plutonium-239, and plutonium-240, comparisons have

gross alpha particle activity contaminant levels for drinking water. The EIS includes comparisons with proposed EPA MCLs because the proposed standards comparative benchmark for comparison of radionuclide concentrations than do the exi

## II COMMENT

The commentor states he would like to see a single data base for Snake River Plain the development of a new model to analyze groundwater contaminant dispersion at the Engineering Laboratory.

### RESPONSE

Most of the Snake River Plain aquifer data collected historically at INEL is retain INEL became involved in environmental restoration, a significant quantity of additi has been collected. Efforts have been made to integrate this data, with maintenanc within each contractor organization. With the realization that contractors would b recognizing the advantage to both the public and INEL, the integration of data base repository is being evaluated by DOE and the new INEL contractor.

The modeling efforts conducted for the EIS used the latest information and developm INEL personnel. Details regarding this modeling effort are discussed in Volume 2, Additional efforts are under way to model contaminant transport and dispersion in s environmental restoration activities associated with Waste Area Group 10 for the Co River Plain Aquifer Remedial Investigation/Feasibility Study. This modeling effort continue to be reviewed by EPA, the State of Idaho, and DOE in accordance with the

## II COMMENT

The commentor recommends further discussion of the extent to which contaminant migr groundwater at the Idaho National Engineering Laboratory would differ as a result o remediation under each alternative.

### RESPONSE

Remedial Action activities at INEL would not differ between the Ten-Year Plan; Mini Storage, and Disposal; and Maximum Treatment, Storage, and Disposal alternatives, a Volume 2, section 3.1.2. The only change in remediation activities occurs with the Only ongoing remediation efforts would be continued under the No Action alternative with this alternative have been analyzed and are discussed in the EIS.

The differences in groundwater contamination are minimal for each of the alternativ modeling conducted for this EIS indicates that under all alternatives, overall grou continues to improve. Volume 2, section 5.8 and Appendix C describe groundwater re and indicate that groundwater quality is likely to improve under each of the altern

## II COMMENT

The commentor states that increased water use at the Idaho National Engineering Lab surface subsidence and collapse.

### RESPONSE

High transmissivity (ability to transmit water) and productivity (ability to produc drawdown or water level decline in or near the well) of the Snake River Plain aquif collapse of the surface above a producing well will not occur. Historically, groun pumping has not been observed at INEL. Any potential increase to aquifer pumping u alternatives is less than a 5 percent maximum increase in current production at the Additional discussion and references on INEL groundwater can be found in Volume 1, section 4.8, and Volume 2, section 4.8 and Appendix F-2.

## II COMMENT

The commentor asks that DOE specify the degree of certainty and scientific basis fo in Idaho National Engineering Laboratory groundwater modeling predictions.

**RESPONSE**

High confidence in predicting future movement of existing contaminant plumes in the decades of monitoring by the USGS and others that have provided good estimates of plume kilometers (1 to 6 miles)] contaminant transport parameters and the importance of precisely known parameter) in contaminant reduction. For example, the tritium plume frequent samples in numerous wells has been receding in recent years. The position plume relative to the INEL boundary has been relatively stationary from 1980 to 1999 the rock and radioactive decay. The measured iodine-129 plume movement has been slow of the plume is shrinking. Predictive modeling of future contaminant movement is a quantitatively and independently measured trends. Parameters used in predictive monitoring discharge monitoring and control (as discussed below) ensures that there is a high these trends will continue.

INEL's decreasing impact on groundwater resources is verified by the results of groundwater monitoring conducted by independent agencies such as the USGS and the State of Idaho. Over These independent assessments confirm DOE environmental monitoring results that no concentrations above EPA MCLs or DOE DCGs exist beyond the INEL boundary. Together improved management practices and remediation efforts planned or under way, it is likely water quality in the Snake River Plain aquifer below INEL will continue to improve plumes (areas in the aquifer with contaminant concentrations above EPA MCLs or DOE continue to recede.

**II COMMENT**

Commentors discuss cleanup of the aquifer at the Idaho National Engineering Laboratory. Commentors state that cleanup of contaminated groundwater is not addressed in the EIS. The rationale is presented for eliminating this alternative from further consideration, will result from failure to conduct complete cleanup. In addition, a commentor states nothing about radioactive contamination of the Snake River Plain aquifer.

**RESPONSE**

Volume 2, section 3.1.2 of the EIS describes the alternatives for SNF management and environmental restoration at INEL within the 10 years covered by the EIS. All No Action alternative) include the completion of all remedial investigations/feasibility studies under the INEL FFA/CO. The draft ROD for the Waste Area Group 10 Comprehensive/Snake River Aquifer Remedial Investigation/Feasibility Study, scheduled for May 2001, will make the level of cleanup for the Snake River Plain aquifer.

Volume 2, Appendix C describes subsurface remediation projects at INEL. The evaluation bounds environmental impacts from environmental restoration (or cleanup) activities specific decisions related to cleanup at INEL generally are addressed under an environmental impact statement executed by DOE, EPA Region X, and the State of Idaho on December 9, 1991, the FFA/CO establishes a comprehensive process integrating the remediation requirements the corrective action requirements of the Resource Conservation and Recovery Act (RCRA) of Idaho's Hazardous Waste Management Act. Cleanup activities are conducted under schedule established in the FFA/CO. RODs under the FFA/CO process are signed by all and represent a joint determination that protectiveness will be achieved through implementation of selected remedy.

DOE is committed to implementing RODs that result from this process. The EIS does not evaluate alternatives for specific remedial projects because these are inherently project-specific because it is DOE policy to use the CERCLA process to consider the environmental impacts of actions.

**II COMMENT**

Commentors state that measurable effects on the Snake River Plain aquifer have occurred from Idaho National Engineering Laboratory activities and these effects should be discussed in excess of any water quality standard. Additionally, one commentor notes that water quality was compared with existing, rather than proposed, standards.

**RESPONSE**

Volume 1, Appendix B and Volume 2 of the EIS discuss natural water chemistry, past practices, resulting contamination levels in groundwater on the INEL site, at the site boundary. Contamination levels are presented even when they are below existing



standards. Because contamination levels are shown to be declining, and concentrations have been above levels that would prohibit any water uses, the subject was given appropriate Volume 1, Appendix B, DOE compares the water quality with both the current and the standards.

## II COMMENT

The commentor states that an explanation of the reasons that increasing subsurface attenuation and migration of localized contaminants is needed.

### RESPONSE

The commentor is correct, and the text has been changed to address the comment. The subsurface attenuation has been deleted from the text of Volume 2, section 4.8.

## II COMMENT

Commentors state they are concerned that geologic conditions and past practices at Engineering Laboratory could contaminate the Snake River Plain aquifer.

### RESPONSE

An accident scenario resulting in maximum groundwater contamination at INEL was analyzed and the results are presented in Volume 2, section 5.14 and Appendix F. The analysis was performed to evaluate the effects of such an accident on the Snake River Plain aquifer. The hypothetical instant failure of a high-level waste tank due to an earthquake with a probability of once every 50,000 years. For comparison, DOE and commercial reactors are designed for seismic events that might occur once every 5,000 to 10,000 years.

The groundwater analysis assumed total failure of the containment and no mitigating flow from the waste tank into the soil immediately following the failure. This hypothesis represents the situation with the most potential impact on the aquifer. Maximum concentrations would be predicted to reach the INEL boundary 300 years after the hypothetical failure, with concentrations less than EPA MCLs or DOE DCGs.

DOE is committed to operating INEL in compliance with all applicable Federal, state regulations and standards pertaining to protecting surface and groundwater resources. That previous waste discharges to unlined ponds and deep wells have resulted in the release of contaminants to the subsurface at INEL; however, because of improved waste management practices, discharges have been reduced or eliminated and regional groundwater quality continues to improve. Volume 2, section 5.8.6, the water resource impacts associated with the alternative. The conclusions are that implementation of any of the alternatives would result in no degradation of quality of water leaving INEL.

The protection of water resources is verified by the results of groundwater monitoring by independent agencies such as the USGS and State of Idaho INEL Oversight Program. These assessments confirm DOE environmental monitoring results, which indicate that no concentrations above EPA MCLs or DOE DCGs exist beyond the INEL boundary and that the concentrations of contaminants that would cause impacts exceeding those impacts assessed in accidents analyzed in Volume 2, section 5.14. With improved management practices and the efforts planned or under way, it is likely that water quality in the Snake River Plain will continue to improve.

## II COMMENT

The commentor asks what value defined the plume boundary in Volume 1, Appendix B, section 5.14.

The plume boundary is defined by concentrations greater than or equal to 0.5 picocuries per liter. The discussion in this section has been changed to incorporate this information.

## II COMMENT

The commentor states that the EIS does not address perched water associated with in

Idaho National Engineering Laboratory.

RESPONSE

The EIS in Volume 2, section 4.8, states that the occurrence of perched water bodies related to the presence of disposal ponds and other man-made surface-water features perched water bodies and are the ones of most concern. However, the EIS was modified to add zone disposal wells to the discussion.

## II COMMENT

The commentor suggests that the EIS incorrectly states that only tritium and nitrate Environmental Protection Agency drinking water standards at the Hanford Site. There are contaminants that exceed EPA numeric standards or risk-based thresholds used when establishing standards.

RESPONSE

The commentor is correct. The discussion in the document has been modified to address

## II COMMENT

The commentor suggests evidence of long subsurface flow paths beneath the Oak Ridge Reservation found by reviewing data from Martin Marietta Energy System's Offsite Well Monitoring. It has reported tritium levels in excess of background in wells south of the Oak Ridge Reservation.

RESPONSE

Adequate information is provided in Volume 1, Appendix F, Part Three, section 4.8, primarily from published hydrologic literature on the ORR, including Status Report: Framework for the Oak Ridge Reservation, and other recent site environmental reports that states that no evidence of contaminant migration along deep, long subsurface flow paths. Interpretation of ORR off-site groundwater monitoring results is beyond the scope of the EIS. The commentor also suggests that elevated concentrations of tritium would not necessarily indicate subsurface transport, but might be due to atmospheric or surface water transport of tritium from past ORR operations. Additional information on the off-site monitoring program has been added to the groundwater discussion in Volume 1, Appendix F, Part Three, section 4.8.2. However, the schedule of environmental monitoring at ORR and the amount of data produced by the monitoring is outside the scope of this EIS. See also the response to comment 05.08.01 (003).

## II COMMENT

The commentor notes that the EIS states the existence of one instance of a groundwater plume crossing the Oak Ridge Reservation boundary and then cites two. The commentor also references in the paragraph.

RESPONSE

Discussion of the solvent plume east of the Y-12 Plant is included in Volume 1, Appendix F, section 4.8.2 as additional supporting evidence of the one strongly suspected instance of a plume crossing the ORR boundary. This reference is not intended as a second instance of groundwater crossing the ORR boundary. The discussion in the document has been modified to clarify the groundwater resources for the ORR are discussed in the EIS in Volume 1, Appendix F, sections 4.6, 4.8, and 5.8.

## II COMMENT

Comments were received concerning DOE making a decision on the proposed alternative for the Snake River Plain aquifer. Information on the effect of aquifer heterogeneities on modeling to assess the extent of contamination in the Snake River Plain aquifer is not complete.

RESPONSE

The heterogeneities referred to in the comment are important locally, on the scale of the aquifer (to 330 feet) with respect to calculating the distribution of contaminants from a point source of contamination. Local heterogeneities in contaminant distribution are averaged out

1,000 meters (330 to 3,300 feet) and regional, 100 to 1,000 meters (330 to 3,300 feet) point source. Intermediate and regional scale parameters were used to calibrate flow transport models. The model parameter values chosen were calibrated with contamination distribution time and space and data from INEL. This data is equivalent to long-term data and serves as the best empirical data for intermediate and regional parameter estimation. INEL water resources and potential impacts resulting from the alternatives considered are described in Volume 2, sections 4.8 and 5.8 and Appendix F. In accordance with the requirements at 40 CFR 1502.22, the EIS summarizes all known existing credible scientific evidence, understanding of the existing environment, identifies reasonably foreseeable impacts, consequences, assumptions and limitations in the groundwater analysis and identifies them in Appendix F. As stated in the analyses, DOE used conservative assumptions to account for uncertainties in modeling the effects of proposed actions on groundwater quality. Results indicate that concentrations of contaminants above EPA MCLs at the INEL site boundary as a result of operations and proposed alternatives.

## II COMMENT

The commentor states that groundwater contamination should be stated in absolute terms (e.g., per year) and concentration differences from background for activities at Idaho National Laboratory, and that perched water quality data should be included in the EIS.

### RESPONSE

The effects of INEL operations on the Snake River Plain aquifer within the INEL boundary are discussed in terms of concentration, picocuries per liter for radionuclides, and milligrams per liter for contaminants. Absolute values of contaminant mass (metric tons per year, for example) are compared with regulatory guidelines, such as the Safe Drinking Water Act, which sets standards in terms of concentration. Because the concentrations of contaminants reported in the Snake River Plain aquifer outside the INEL boundary are DOE DCGs, regional groundwater quality is compared with background levels for those areas where they occur naturally and detection limits for those that do not.

In accordance with the provisions of NEPA and CEQ implementing regulations at 40 CFR 1502.22, the EIS summarizes existing credible scientific evidence relevant to understanding the effects of INEL operations on groundwater resources and evaluates potential consequences.

The net effects of INEL operations on groundwater resources are reflected in Snake River Plain aquifer monitoring results. Snake River Plain aquifer monitoring well data from wells in the Snake River Plain and other areas where contaminated perched aquifers may exist was included in the analyses in Table 4.8-1. The data indicate that no significant impacts at the INEL boundary or other contaminated perched aquifer releases to the Snake River Plain aquifer were not independently discussed in this EIS because Snake River Plain aquifer water quality near INEL discharges are adequately evaluated in Snake River Plain aquifer monitoring conducted by independent agencies. Evaluating additional perched water information would not be evaluating reasonably foreseeable adverse impacts. This conclusion is further supported by modeling conducted for this EIS, which included analyses for the most likely initial release of contaminated water in perched water zones (percolation ponds and injection wells) on the Snake River Plain aquifer. Furthermore, the CERCLA ROD for the perched aquifer at the Test Reactor Area indicates that no remedial action will be required because the perched water contaminants will not pose unacceptable risks or consequences to the Snake River Plain aquifer. Characterization of contaminated perched water is proceeding under CERCLA. The regional effects of contamination on the Snake River Plain aquifer are bounded by the high-level waste tank failure in the Accident Analysis section. This analysis indicates that there will be no significant impacts on the INEL boundary from the failure of containment of a high-level waste tank.

The curie content, volume, and rate of release of the source term used in this analysis are based on parameters that could be reasonably used to characterize contaminated perched water at INEL. Independent groundwater monitoring results, groundwater modeling results, and discharge control and monitoring suggest that contaminants in the Snake River Plain aquifer decrease with time. Snake River Plain aquifer monitoring results are discussed in section 4.8 and shown in Table 4.8-1. Additional detail on subsurface water monitoring is found in Volume 2, Appendix F-2.

No discernible water quality impacts are expected, since under normal operating conditions no discharges of contaminants to the soil or directly to the aquifer above EPA MCLs are expected. The hypothetical release due to an accident is discussed in Volume 2, Appendix F-5. Release amounts and modeled impacts in absolute terms and bounds any impacts from releases. Additional detail for the INEL is in Volume 2, section 5.8.

## II COMMENT

The commentor suggests that certain perched aquifer groundwater monitoring data be and that groundwater quality comparisons with proposed maximum concentration levels

### RESPONSE

The data on water quality are provided in Volume 2, section 4.8. Data presented in with EPA MCLs, although MCLs are drinking water standards, not groundwater standard requirement to report contaminants in the Snake River Plain aquifer relative to MCL although this is usually done. The EIS used proposed MCLs because, for comparison best available tool for individual radionuclides not having current MCLs. Other MC are either adjusted gross alpha, or a calculated 4 millirem-per-year dose, with the strontium-90, and radium-226/228, which have specific MCLs. Volume 2, section 4.8 clarify that while the proposed MCLs may change, they are used for groundwater qual purposes.

The net effects of INEL operations on groundwater resources are reflected in ground results presented in the EIS. Monitoring well data were included in the analysis t contaminant releases to the Snake River Plain aquifer. Independent groundwater mon suggest that contaminants in the Snake River Plain aquifer are likely to decrease w are discussed in Volume 1, Appendix B, section 4.8 and are shown in Table 4.8-1.

A description of water resources and potential environmental consequences to water including the Snake River Plain aquifer, are discussed in Volumes 1 and 2, sections analysis performed for the EIS integrated available data and technical information to evaluate contaminant transport and predict future trends in aquifer water qualit using conservative assumptions was completed through 2035 to add assurance to the c in the document. Section 5.8 concludes that overall aquifer water quality would ac this period. A discussion of the methodology and assumptions used for the computer Volume 2, Appendix F .

Key contaminants were selected by comparing the contaminant data with the current 4 proposed EPA 1991 MCLs and contamination guidelines found in DOE Order 5400.5, Radi Protection of the Public and the Environment, derived concentration guides, radionu Contaminants with concentrations 50 percent of either of the regulatory limits were contaminants. More detailed data on the results of groundwater monitoring at INEL reading rooms and the INEL Technical Library.

The data indicate that no significant impacts at the INEL boundary have resulted fr releases to the Snake River Plain aquifer. Current independent groundwater monitor contaminants in the Snake River Plain aquifer are generally decreasing with time. cited by the commentor, which are reported to be thousands of times above the MCLs, found in any reference such as gross alpha at Test Area North, or apply only to per Reactor Area. The EIS did not attempt to assess perched water, but rather concentr River Plain aquifer. The CERCLA ROD for Test Reactor Area indicates that no remedi required, because the perched water contaminants will not result in unacceptable ri the aquifer.

Other perched water is not independently evaluated, because Snake River Plain aquif impacts from INEL discharges are adequately evaluated in Snake River Plain aquifer by DOE and independent agencies. Evaluation of additional perched water informatio relevant to evaluating reasonably foreseeable adverse impacts. This conclusion is results of modeling conducted for this EIS that included analyses for the most like contaminated water in perched water zones (percolation ponds and injection wells) o Plain aquifer .

## II COMMENT

The commentor states that the discussion in Volume 1, Appendix B, section 4.8 on ex contaminant levels in groundwater at Test Area North is misleading because the EIS contaminants first exceeded standards at a time when the commentor says they should

### RESPONSE

The discussion in Volume 1, Appendix B, section 4.8 has been changed for clarificat on showing recent trends in groundwater quality at INEL. Any long-term accumulatio from these trends. Contaminant concentration data were reviewed for the period 198

modeling and sampling data have indicated that Snake River Plain aquifer contaminat INEL is decreasing with time.

## II COMMENT

Comments were received asking if any radioactively contaminated water has been found within National Engineering Laboratory boundaries.

### RESPONSE

Extremely low concentrations of iodine-129 and tritium have migrated outside the INEL site boundary. In 1992, iodine-129 concentrations were measured in two wells south of the INEL site boundary. EPA MCLs, as follows: (a)  $1.0 \times 10^{-5}$  picocuries per liter in Well No. 11, located 4 kilometers (4 miles) beyond the boundary, and (b)  $3.0 \times 10^{-5}$  picocuries per liter in Well No. 12, located 8 kilometers (8 miles) beyond the boundary (Mann, L.J., U.S. Geological Survey, personal communication with A.L. Lundahl, Science Applications International Corporation). Tritium concentrations observed much below MCLs just south of the INEL site boundary in 1985. By 1988, tritium had receded to within the INEL site boundary, and its size has continued to decrease. Conditions and Distribution of Selected Chemical Constituents in Water, Snake River Plain, Idaho National Engineering Laboratory. Cobalt-60, strontium-90, cesium-137, plutonium-240/241, and americium-241 have not been detected outside the INEL site boundary.

## II COMMENT

The commentor notes that the geology of the Oak Ridge Reservation would result in a leak were to occur.

### RESPONSE

A conservative analysis of the potential effects of a leak from an SNF storage facility is presented in Volume 1, Appendix F, section 5.8.2. The analysis found that exposures would be small. There is very little potential for contamination of the Knox aquifer from the operating facilities. The proposed SNF facilities are designed to have no liquid release of chemical or radiological characteristics through the use of modern technologies, including containment, leak detection, and water-balance monitoring equipment.

## II COMMENT

The commentor states concern that vadose zone contaminants and other buried waste at the Idaho National Engineering Laboratory Radioactive Waste Management Complex were not included in the EIS groundwater model and may constitute a significant source of future contamination to the Snake River Plain aquifer.

### RESPONSE

Vadose zone contaminants at the INEL RWMC were not included in the groundwater model. Vadose zone contaminants and other buried waste constituents at the INEL RWMC were included in the FFA/CO. Characterization of these constituents is in progress as part of ongoing investigations.

The net effects of INEL operations on groundwater resources are reflected in groundwater monitoring results. Snake River Plain aquifer monitoring well data from wells in the vicinity of the INEL site boundary included in the analysis that resulted in Table 4.8-1. These data indicate that tritium impacts to the Snake River Plain aquifer at the INEL boundary have resulted from RWMC releases to the Snake River Plain aquifer. Independent groundwater monitoring results show that contaminants in the Snake River Plain aquifer are likely to decrease with time. This is shown in Volume 1, Appendix B, section 4.8 and shown in Table 4.8-1.

Recently completed flood and erosion control construction at the RWMC will reduce tritium impacts through the unsaturated zone by minimizing surface flooding at the RWMC. This reduction effectively increases natural contaminant attenuation processes that occur in the vadose zone, reducing impacts on aquifer water quality.

It is likely that the effects of RWMC contaminants on the Snake River Plain aquifer are limited to a hypothetical accident scenario referenced in the EIS in Volume 1, Appendix B, section 4.8. A hypothetical accident, representing a reasonably foreseeable accident, includes groundwater contamination from a major contaminant release to the subsurface. The analysis indicates that the hypothetical

cause small impacts to the aquifer, with no contaminants above MCLs at the INEL bou

## II COMMENT

The commentor requests additional information on impacts from groundwater contamina  
RESPONSE

The purpose of this EIS is to evaluate the potential environmental impacts from pro this reason, assumptions were made to ensure that estimated doses are conservatively upper bound of potential impacts. The EIS is not intended to substitute for the as regulations. Any facilities constructed or operated under the chosen alternative w applicable regulatory requirements. In the example cited by the commentor, further chromium concentrations in groundwater at INEL is in Volume 1, Appendix B and in th sections of Volume 2.

Volume 2, section 5.12 discusses the potential health effects for on-site workers a analyses show that impacts would be small. The major impacts have been from past p impacts are projected to decrease because of changes in facility procedures. Subsu contaminant distribution are discussed in Volume 2, Appendix F-2.

## II COMMENT

The commentor states the Brookhaven National Laboratory is in the Long Island Nassa System, and the West Valley Demonstration Project is in the Cattaraugus Creek Aquif commentor also states that these have been designated as sole-source aquifers pursu Drinking Water Act and asks that the sensitivity and importance of these sole-sour considered in the selection of the interim alternative. Specifically, that Nationa documentation should include a detailed assessment of the potential groundwater imp  
RESPONSE

Volume 1, Appendix E, sections 3.1.1 and 3.3.1 have been revised to acknowledge sol designations for aquifers underlying these sites. More detailed aquifer characteri will be incorporated by reference. Detailed sole-source aquifer characterization d because this is a programmatic EIS. Potential impacts from alternatives considered quality are expected to be small. Subsequent actions that may result from this EIS project-specific NEPA reviews and compliance, but impacts from previous activities scope of this EIS.

## II COMMENT

The commentor states that the discussions of groundwater occurrence, movement, use, are not consistent between all sites. Without consistent information, there is lit The commentor also states that consistent data probably does exist through investig CERCLA and RCRA, state and Federal permitting, and engineering design studies for b  
RESPONSE

For the analysis of impacts at a programmatic level, the hydrological information and its site-specific appendices is sufficient for purposes of the EIS. Additional or project level will provide more specific information as needed. While it appear inconsistency among the various sites on hydrologic information summarized in Volum is largely a reflection of the differences in water uses, availability, water sourc that are important at each site. The appendices do, with minor exceptions, include information on lithology, water use, contamination, well yields, and consumption. exception, a reference for further detail is provided, and additional information f included in Volume 1 to balance the discussion.

## II COMMENT

The commentor suggests that the elevated nitrate, chloride, and sulfate levels foun vicinity of the Idaho Chemical Processing Plant are not the result of agricultural

might be attributable to the Naval Reactors Facility.

#### RESPONSE

The discussion in Volume 1, Appendix B, section 4.8 has been revised to state that nitrates, chlorides, and sulfates are the result of the disposal well and infiltrat Chemical Processing Plant. The related sections of Volume 2 of the EIS have also b no evidence to substantiate the suggestion that the contaminant levels at the Idaho Plant are caused by the Naval Reactors Facility.

## II COMMENT

The commentor states that in Volume 2, Appendix F-2.2.2 of the EIS, source terms fo from SNF storage uses Idaho Chemical Processing Plant Building 666 as the generic e commentor states that the facility is not generic or typical for the Idaho National Rather, the Idaho Chemical Processing Plant is atypical because it is the only faci standards. The commentor also states that because the other storage facilities wil near future and pose the greatest risk of discharge, the EIS must use one of these generic case.

#### RESPONSE

Volume 2, Appendix F-2.2.2 referenced by the commentor states that Idaho Chemical P discharge and a hypothetical discharge from a generic facility were used to generat bounding postulated leak scenario is greater than releases from any facilities at I Expanded Core Facility. Results indicate that there will be no contaminants above boundary resulting from a postulated operational leak.

## II COMMENT

The commentor points out that contamination of the Dublin-Midville aquifer (a regio water) underlying the Savannah River Site is more widespread than the text of the D is, the text notes that evidence of contamination has been found in only one produc commentor also notes that there may be an inconsistency in the discussion of contam aquifer.

#### RESPONSE

The text in Volume 1, Appendix C, section 4.8 has been revised to indicate that con trichloroethylene and tetrachloroethylene) have been detected above Primary Drinkin another well completed in the Dublin-Midville aquifer system.

Regarding contamination of the Gordon aquifer, there are several plumes of contamin none has moved offsite and none is available to off-site users of this aquifer. Cu efforts are intended to prevent off-site movement of this contaminated groundwater.

## II COMMENT

The commentor suggests DOE sum the pumping rates for all production/potable wells o National Engineering Laboratory to produce an estimate of maximum pumping capacity.

#### RESPONSE

While it is true that the capacities of all pumps could be summed to produce a maxi rate, the likelihood of all pumps operating at one time is very small. Even during extended power outage, it is unlikely that all pumps would be operating simultaneou maximum pumping capacity would not be reached. Maximum pumping capacities are ther relevant to assessing potential impacts from pumping.

## II COMMENT

The commentor states that contaminants released to the subsurface from the West Bea location at the Oak Ridge Reservation could reach the Knox aquifer, either directly that could rapidly transmit contaminants to areas underlain by carbonates, or indir macropores to Grassy Creek and entering the aquifer through losing reaches of the c

**RESPONSE**

Full resolution of these concerns would require detailed investigation of site cond pathways. If ORR were chosen as a site for new SNF management facilities, site-spe hydrologic studies would be performed. Available information provides a sufficient assessment that no significant environmental consequences related to water resource from the operation of SNF management facilities.

As discussed in the EIS, proposed SNF management facilities are designed to have no waste water with hazardous chemical or radiological characteristics. Facilities wo to prevent and minimize the impacts of leaks, including secondary containment, leak undetected subsurface release are, however, analyzed in the EIS (see Volume 1, Appe section 5.8.2). The analysis indicates that most radiological constituents would b standards at the point of release, and that additional substantial reductions in th constituents would occur as a result of dilution with groundwater and the receiving The worst-case undetected release is estimated to constitute less than 0.0003 perce average discharge of Grassy Creek at its confluence with the Clinch River. Any con Knox aquifer via the losing reaches of Grassy Creek would undergo a similar degree there would be little, if any, impact on water quality in the aquifer.

It is not likely that macropores would provide a direct connection to the Knox aqui proposed SNF management facility, because available information indicates that the Conasauga Group strata that are not normally characterized by karst development or well-connected to the Knox aquifer. (The only Conasauga Group information included aquifer is the uppermost formation in the group, the Maynardville Limestone). If a connection did exist and allowed an undetected release to reach the aquifer, diluti and in the aquifer would significantly reduce the potential for impacts on water qu Appendix F, Part Three, sections 4.6 and 4.8 for further discussion of site geology

**II II COMMENT**

The commentor states that past waste management activities have resulted in contami Clinch River and lakes near the Oak Ridge Reservation.

**RESPONSE**

Natural resources and impacts associated with the SNF management alternatives at OR discussed in Volume 1, Appendix F, Part Three. Current waste management problems, releases, and environmental restoration activities for cleanup of contaminated site the scope of this EIS. Contact public affairs personnel at ORR for information on upcoming opportunities for public comment.

**II COMMENT**

The commentor states that the EIS did not mention storm water runoff and storm wate National Engineering Laboratory wells as a source of waste water.

**RESPONSE**

The EIS does address the use of storm water injection wells used at INEL. This di in Volume 1, Appendix B, section 4.8; Volume 2, section 4.8; and Volume 2, Appendix discussion of this subject also can be found in the Water Resources Supporting Docu Environmental Restoration and Waste Management EIS, a reference used for the EIS, a rooms and information locations listed in the EIS.

**II COMMENT**

The commentor discusses the production of toxic materials upstream from the town of the South Carolina coast, particularly impacts to watersheds, such as the Savannah local and regional aquifers.

**RESPONSE**

Potential impacts to surface water and groundwater of the various SNF management al for SRS are evaluated in Volume 1, Appendix C, section 5.8. Cumulative impacts to presented in Volume 1, Appendix C, section 5.16.4. DOE expects the impact on water implementation of any of the alternatives under consideration to be small. Each of



contribute to the very small releases of radionuclides that normal SRS operations discharge through Federally permitted waste-water outfalls. In the unlikely event of a contaminants to either the ground or directly into the subsurface, DOE does not expect impacts to surface water or drinking water aquifers under SRS. Cleanup of groundwater resources from past waste management practices at SRS are not of this EIS. However, environmental restoration activities at DOE sites are performed in accordance with agreements negotiated with the appropriate regulatory agencies and in compliance with guidance and environmental regulations.

## II COMMENT

The commentor states that Volume 1, Appendix B of the EIS erroneously assumes that at the Idaho National Engineering Laboratory over the last 8 years is indicative of the future.

### RESPONSE

Volume 1, Appendix B, section 4.8.1 has been changed to address this concern. The very dry years at INEL, which may not be indicative of the future. The new discussion of dry years, surface water in the Big Lost River does not usually reach the western boundary because INEL is in a closed drainage basin, surface water never flows offsite.

## II COMMENT

The commentor expresses concern that, following the December 1991 tritium leak into individuals in Savannah received a high dose of radiation from drinking the water.

### RESPONSE

The maximum dose to the public resulted from individuals who drink Savannah River water from public drinking water supplies that use Savannah River water are at Port Wentworth, Beaufort-Jasper, South Carolina, both near Savannah, Georgia, the residence of the maximum dose to an individual consuming 2 liters of water per day from the Port Wentworth system was 0.030 millirem. The maximum dose from the Beaufort-Jasper system was 0.0096 millirem. The maximum dose from the Savannah River system was 0.8 percent and 0.2 percent, respectively, of the EPA drinking water standard (1 millirem per year). The maximum dose from this release to a hypothetical individual (301 bridge just downstream of SRS was 0.035 millirem. There are no known consumers of Savannah River water at that location. The City of Savannah does not use the Savannah River water for drinking water.

The low dose received by individuals consuming water from the two public systems may result in adverse health effects. The values are very much less than the variation that normally results from day to day and from place to place within any city. Radiation from both normal and off-normal occurrences from storage of SNF at SRS are projected to be less than that from the December 1991 tritium release.

## II COMMENT

The commentor notes that Volume 1, Appendix B of the Draft EIS does not address local basin flooding at the Idaho National Engineering Laboratory.

### RESPONSE

Local basin flooding at INEL is discussed in Volume 2, section 4.8.1 and Appendix F. Appendix B has been changed to discuss local basin flooding at INEL.

The DOE Idaho Operations Office recently completed constructing new flood and erosion control at the RWMC, which will reduce the potential of localized flooding at the complex. The INEL accident assessment summarized in Volume 2, section 5.14 considers flooding phenomena as potential initiators of facility accidents. Some potential accidents were not included in the detailed analysis because they were considered reasonably foreseeable, and some were included for detailed analysis because of their large potential consequences. The consequences of the high-level waste tanks were selected for detailed analysis over a flooding scenario because of the radioactive inventory in the high-level waste tanks could have a larger potential for resources than a flood. The impact evaluations show that the risk to workers and the public from operations would be small for all alternatives.

## II COMMENT

The commentor notes that past waste management activities have resulted in contamination of the Snake and Columbia Rivers.

### RESPONSE

No significant impacts to the Snake River and the Columbia River have resulted from surface water drains internally into natural sinks at or near INEL. No surface water drains into the Snake River. The protection of water resources is verified by the monitoring conducted by independent agencies such as the USGS and State of Idaho IN Program. These independent assessments confirm DOE environmental monitoring results that no contaminants in concentrations above EPA MCLs or DOE DCGs exist beyond the INEL. With improved management practices and remediation efforts planned or under way, the water quality in the Snake River Plain aquifer under the INEL will continue to improve. Future impacts to the Snake and Columbia Rivers resulting from INEL past, present, and future are likely to occur.

As discussed in Volume 1, Appendix A, section 4.8, tritium, iodine-129, and uranium have higher concentrations downstream of the Hanford Site than upstream, but well below guidelines established by DOE and EPA drinking water standards. Cobalt-60 and iodine-131 are consistently found in measurable quantities during 1989 in samples of Columbia River water at the Rapids Dam, the 300-Area water intake, or the Richland city pump house. In 1989, the strontium-90 concentrations were essentially the same at Priest Rapids Dam (upstream) and the Richland pump house.

## II COMMENT

Commentors express concern about existing contamination of the Clinch River and the East Fork Poplar Creek.

### RESPONSE

Existing contamination of the local surface-water bodies is acknowledged in the EIS and other surface waters have been affected by activities at ORR as well as by others from ORR. Water quality in the Clinch River is routinely monitored by the Tennessee Department of Environment and Conservation, and the USGS.

The Oak Ridge Reservation Environmental Report for 1992 summarizes 1992 Clinch River results at the Gallaher and Kingston water treatment plants. While radionuclides are significantly greater than zero in the treated water for a number of radioactive elements, concentrations are not greater than the EPA primary drinking water standards for any element. The environment affected by water resources at ORR is discussed in Volume 1, Appendix F, section 4.8.

The addition of Clinch River water to East Fork Poplar Creek is required by the Tennessee Department of Environment and Conservation in order to guarantee a minimum base flow as the limit of plant discharge permit are based on flow management in the creek. The purpose of the Project is to maintain a consistent flow in the creek of 7 million gallons per day for intended uses.

It is DOE policy to consider the protection of water resources in the design, construction, and operation of its facilities, and to comply with Federal, state, and local regulations and standards for protection of water resources. The proposed SNF management facilities are designed to have no contact with waste water with hazardous chemical technologies, which include secondary containment, detection and water-balance monitoring equipment. Therefore, no significant environmental impacts related to water resources are anticipated from the operation of SNF management facilities. Water resources at ORR are discussed in Volume 1, Appendix F, Part Three, section 5.

## II COMMENT

The commentor notes that the EIS should include a discussion of the impacts to the Sound.

### RESPONSE

The environmental consequences associated with storage of Naval SNF at Puget Sound

discussed in Volume 1, Appendix D, section 5.1.1. The environmental consequences f analyzed are based on estimates of the amount of SNF that would be stored at the sh and current knowledge of the design features associated with SNF storage systems. consequences for foreign fuel shipments are bounded by the analyses included in thi ports for shipment of Hanford N-Reactor fuel for overseas processing are discussed example for evaluation of reasonably foreseeable impacts. The review of the enviro associated with the alternatives shows that impacts on the environment from these a small. Foreign research reactor (FRR) fuel shipments and their impacts to the Port covered in the EIS entitled Proposed Nuclear Weapons Nonproliferation Policy Conce Research Reactor Spent Nuclear Fuel (Draft).

## II COMMENT

The commentor states that DOE would be required to apply for water rights to withdr water for new spent nuclear fuel storage and management activities at the Hanford S  
RESPONSE

As discussed in Volume 1, Appendix A, section 4.8.2, DOE withdraws water from the C under DOE's Federally reserved water rights. From a programmatic impact standpoint alternative would use approximately 1 percent of the baseline of total Hanford usag K). In general, new SNF facilities, if any, would use less water than existing faci analysis for any new SNF storage or treatment facilities would address water usage

## II II COMMENT

The commentor asserts that the EIS assumes no surface water flow onsite and that th affects the evaluation of Snake River Plain aquifer recharge at Idaho National Engi  
RESPONSE

The EIS does not make this assumption. Volume 1, Appendix B, section 4.8.2 discuss hydrogeology, which includes summary text regarding recharge of the Snake River Pla Infiltration along stretches of the Big Lost River, Little Lost River, and Birch Cr discussed in greater detail in Volume 2, section 4.8.1 and Volume 2, Appendix F-2.2 reference (Streamflow Losses and Groundwater Level Changes Along the Big Lost River National Engineering Laboratory) referred to by the commentor.

## II COMMENT

The commentor states that water tables at the Idaho National Engineering Laboratory and Nevada Test Site are contaminated with radioactive waste and that the Columbia contaminated by Idaho National Engineering Laboratory and Hanford Site waste.  
RESPONSE

DOE evaluated the impacts to groundwater quality of proposed actions, where appropri past practices have been analyzed to determine cumulative impacts. These analyses Volume 2, Appendix K and Volume 1, Chapter 5, Appendices A, B, and F.

## II COMMENT

The commentor suggests clarification of the discussion of the depth of excavation i Nevada Test Site.  
RESPONSE

As indicated in the preliminary design (Description of a Generic Spent Nuclear Fuel the Programmatic Environmental Impact Statement), the cask loading and unloading po receiving and canning building are the deepest structures in the facility and are 1 Allowing another 2 meters (6 feet) for secondary containment, leak detection system needs results in an estimated excavation depth of 15 meters (50 feet). As indicate F, Part Two, section 4.8.2, the depth to the water table in Area 5 is 244 meters (8 perched water tables have been reported at 21 meters (70 feet) in some locations of

programmatic nature of the EIS and the preliminary status of the facility design, to demonstrate that the excavation is expected to occur within the vadose zone at NTS.

## II COMMENT

The commentor has concerns about seismic safety and the contamination of water resources at the Site.

### RESPONSE

Volume 1, Appendix A, sections 4.8 and 5.8 discuss water resources at the Hanford Site, including seismic hazards, is discussed in Volume 1, Appendix A, section 4.6. Volume 1, section 5.2.6, the proposed alternatives for SNF management would have similar water resources. Impacts of management SNF at K-basin at the Hanford Site will be discussed in the EIS.

## II COMMENT

The commentor suggests that tougher water quality standards from the Clean Water Act in the EIS, rather than limits based on the Safe Drinking Water Act.

### RESPONSE

The Clean Water Act (CWA), 33 USC Section 1251 et seq., protects surface waters by discharge of pollutants to surface waters of the United States be controlled or prevented. EPA sets nationwide, industry-by-industry effluent standards. The CWA standards are (SDWA), 42 USC Section 300(f) et seq., ensures that water out of the tap is fit to drink. SDWA, EPA is responsible for setting national standards that must be met by the person providing water to the tap. The drinking water standards under the SDWA are specifically set to protect against adverse health effects to persons from the consumption of drinking water. Drinking water has become the key Federal reference point for prevention and cleanup decisions under the CERCLA.

For a number of reasons, it is difficult to conduct a simple comparison of SDWA standards. First, for any one contaminant, CWA standards vary greatly from state to state and by industry. The quality of the "receiving waters" for any given facility also affect the standards imposed under a CWA permit. Whether the facility analyzed in the EIS is a new facility or an existing facility also impacts the CWA permit standards. For some constituents, from some states, with a new facility, the CWA permit standards might be more stringent than under the SDWA. But it is definitely not a correct generalization that CWA standards are more stringent than SDWA standards, and in fact in many instances, the opposite is true.

Because the national standards set under the SDWA are more uniformly applicable to all facilities, they were analyzed in this EIS, and more important, because the SDWA standards are consistent with human-health based, rather than technology based, they were used in this EIS as a comparison point.

## II COMMENT

The commentor states that DOE's activities at Idaho National Engineering Laboratory irretrievably impact water resources.

### RESPONSE

Irreversible and irretrievable effects on resources are discussed in Volume 2, section 5.8.6. INEL have resulted in chemical and radioactive contaminant plumes in the Snake River Plain aquifer discussed in Volume 2, section 5.8.6. Water use and liquid effluent discharges at INEL have had minimal effect on Snake River Plain aquifer water quality and quantity.

Water resources and impacts resulting from all waste management and environmental remediation (SNF) alternatives considered for INEL are described in Volume 2, sections 4.8 and 5.2.6. For all alternatives considered, the possible future sources of contamination would be similar to previous practices. This would be a result of waste management practices, including monitoring, as well as natural contaminant attenuation and radioactive decay for high-level waste. Computer groundwater modeling using conservative parameters (discussed in Volume 2, section 5.2.6), indicates that existing contaminant plumes within the INEL boundary would continue to exist through 2035. The modeling further indicates that overall aquifer groundwater quality would remain acceptable.

improve in that period and probably continue to improve after 2035.

INEL's commitment to DOE policy regarding the protection of water resources is verified by groundwater monitoring conducted by independent agencies such as the USGS and State Oversight Program. These independent assessments confirm DOE environmental monitoring results that indicate that no contaminants in concentrations above EPA MCLs or DOE the INEL boundary. With improved management practices and remediation efforts planned it is likely that overall water quality in the Snake River Plain aquifer below the INEL will improve.

Recent improvement in groundwater quality at INEL is documented in report (e.g., Hydrology and Distribution of Selected Chemical Constituents in Water of the U.S. Geological Survey Review of the Production, Use, and Disposal of Groundwater and the Generation, Storage, and Processing of Radioactive Liquid Waste at the Idaho Chemical Processing Plant of the INEL Program) as referenced in the EIS. Although small irretrievable impacts to groundwater are possible, recent sampling results, computer modeling using mildly conservative assumptions, and liquid effluent discharge management ensure that impacts from current and future activities and future effects of past practices have a decreasing effect on aquifer water quality.

## II COMMENT

The commentor states that a full mass balance of water pumped from the aquifer and volume analysis must be conducted for the entire history of the Idaho National Engineering Experiment Station.

RESPONSE  
The net effects of INEL operations on groundwater resources are reflected in groundwater monitoring results. Monitoring-well data were included in the analysis that resulted in Volume 4.8, Table 4.8-1. This data indicates that to date no significant impacts at the INEL have resulted from INEL contaminant releases to the Snake River Plain aquifer. Independent monitoring results and groundwater modeling conducted for this EIS indicate that concentrations in the Snake River Plain aquifer are likely to decrease with time. These results are discussed in Appendix B, section 4.8 and shown in Table 4.8-1. Additional evaluation would not be necessary for evaluating reasonably foreseeable adverse impacts. Water usage is described in Volume 1. A description of water resources and potential environmental consequences to water resources, including the Snake River Plain aquifer, are discussed in Volumes 1 and 2, sections 5.1 and 5.2. Analysis performed for the EIS integrated available data and technical information to evaluate contaminant transport and predict future trends in aquifer water quality. This analysis was completed through 2035 to add assurance to the conclusions reached in the document. Section 5.8 concludes that overall aquifer water quality would actually improve over time. A discussion of the methodology and assumptions used for the computer modeling effort is provided in Appendix F of the EIS.

In accordance with NEPA regulations at 40 CFR 1502.22, the EIS summarizes all known scientific evidence relevant to understanding the existing environment, identifies potential impacts, and evaluates potential consequences.

A full mass balance and waste discharge volume analysis conducted for the entire history of the INEL will not change the conclusions reached in the EIS.

## II COMMENT

The commentor states that the EIS de-emphasizes impacts on water resources by categorizing water resources as an "Issue Not Discussed in Detail" and ignoring water resources in the analysis. The commentor further states there is an overemphasis on water usage, rather than groundwater contamination, in addressing water resources in Volume 1.

RESPONSE

In response to public comments, section 5.3.2.6 has been added to Volume 1 to address impacts on water resources. Based on the site-specific analysis in appendices to Volume 1, addressing water resources under "Issues Not Discussed in Detail" has been enhanced. The analysis indicates that radiological impact to water resources at each candidate site is small.

## II COMMENT

The commentor states that site-specific discussions on water resources and hydrolog should be compared, rather than just scattered throughout six appendices.

#### RESPONSE

Volume 1, Chapter 4 summarizes the pertinent characteristics of the affected enviro sites under consideration in the EIS. Detailed water resource and hydrological cha alternative sites under consideration are in Volume 1, Appendices A, B, C, D and F, Although not specifically provided in discussion or tabular form, a side-by-side co between the information in the site-specific appendices. Due to the complexity and hydrogeologic characteristics between sites, such comparisons are subjective and de interests of the reviewer, as well as decisionmakers.

## II COMMENT

The commentor states that water resources, and in the context of the comment perhap resources, would be unavoidably adversely impacted because only limited remediation

#### RESPONSE

The environmental restoration actions under the alternatives considered in this EIS provisions of the CERCLA. CERCLA procedures provide for ecological risk assessment of injury or potential injury to natural resources resulting from past releases of alternatives include projects for protecting the vadose zone and cleaning groundwat and/or retrieving buried wastes. In keeping with DOE's Native American Policy (Mem Management of Cultural Resources at Department of Energy Facilities, U.S. DOE, Wash February 23, 1990), DOE will consult with the Tribes during the planning and implem proposed alternatives. Additionally, DOE has implemented the Working Agreement, Po American Consultation to ensure communication with the Shoshone-Bannock Tribes con treatment of archaeological sites as mandated by the National Historic Preservation Resources Protection Act, and the protection of human remains under the Native Amer Protection and Repatriation Act.

The prediction of unavoidable adverse impacts to groundwater was based on analyses extent of known contamination and potential effectiveness of existing and reasonabl technologies. Note that the impacts will not be caused in the future but will be r actions and operations. CERCLA and the National Contingency Plan contain provision residual injury to natural resources and natural resource damage assessment. In a the DOE Idaho Operations Office notified the State of Idaho, the Shoshone-Bannock T Department of the Interior of potential injury to trust resources caused by past re

## II COMMENT

The commentor states that the spent nuclear fuel portion of the EIS does not discus restoration at Oak Ridge Reservation, and the adverse impacts for the Y-12 Plant ha

#### RESPONSE

Detailed analysis of existing contaminant sources and transport pathways are beyond If ORR is selected for new SNF management facilities, site-specific groundwater st performed.

Geology and water resources for ORR are discussed in Volume 1, Appendix F, Part Thr 4.8, and 5.8.

## II 5.9 Cumulative Impacts

## II COMMENT

The commentor states the EIS does not adequately discuss the direct, indirect, or c transporting spent nuclear fuel and other radioactive and hazardous materials.

#### RESPONSE

DOE believes the EIS and reference documents contain an adequate discussion of dire cumulative impacts of transporting SNF and other radioactive materials. Incident-f

hazardous materials results in essentially no impacts, as discussed in Volume 1, section 5.1. of highway, railway, and barge transportation impacts and potential accident impact section 5.1.

The cumulative impacts analyses are discussed in Volume 1, section 5.3 and Volume 2 Cumulative impacts of radioactive and hazardous materials transportation have been 1, section 5.3.2.

DOE conducted a comprehensive transportation cumulative impacts analysis, evaluating future shipments of radioactive material, which include radioactive waste and SNF. cumulative impacts analysis include past transportation activities, transportation in this EIS, reasonably foreseeable future transportation activities, and general t The analyses described in Volume 1, section 5.3 and Volume 2, section 5.15 show that exposing the public to radiation hazards is low, and the overall impacts under all analyzed in this EIS would be small.

## II COMMENT

The commentor expresses an opinion that DOE hides behind a claim of national security information from the public, and thus prevents an accurate assessment of impacts.

### RESPONSE

In recent years, DOE has released a significant amount of previously classified data release additional information as it is declassified. Most environmental monitoring and significant amounts of information are available to the public, such as the annual published for each site. Some data on DOE activities remains classified until release Energy. Volumes of publicly available data were used in the preparation of this EIS list of references for each volume and the associated appendices. This EIS contains for members of the public to interpret and evaluate impacts.

## II COMMENT

The commentor is of the opinion that the EIS should evaluate the impacts and conditions generations from now.

### RESPONSE

The time periods being considered in this EIS are 40 years for the programmatic management ultimate disposition, and 10 years for environmental management and waste management INEL. The EIS evaluates reasonably foreseeable impacts associated with the proposed alternatives analyzed in the EIS. Volume 1, Chapter 5 and Appendix K, and Volume 2 summarize the environmental impacts of all the alternatives considered in this EIS. the impacts of all alternatives would be small. Because of the speculative nature associated with projecting actions and impacts many years in the future, meaningful horizon is not possible. Whereas this EIS addresses interim actions until ultimate analysis of disposition options such as geologic disposal will entail analysis of potential into the future. Such analysis will likely be part of a future EIS.

## II COMMENT

The commentor states that the EIS does not address the environmental impacts of breeder fuel into Idaho.

### RESPONSE

The environmental consequences of all SNF alternatives, including those that involve SNF to INEL, are extensively discussed in Volume 1, Chapter 5. This discussion is 1, Appendices B and D. Volume 2, Chapters 4 and 5 further discuss environmental impacts relative to waste management and environmental restoration projects.

## II COMMENT

The commentor states that specified matters are not adequately addressed as required

Environmental Policy Act and pursuant to Council on Environmental Quality regulation Act. The matters specified by the commentor are the different types of SNF storage the need for potential SNF processing; cumulative impacts, similar impacts, and res future permanent disposal; a monitoring and safety program that provides independent conditions; and activities and past problems associated with SNF management.

#### RESPONSE

Decisions regarding wet or dry storage and processing will be based on future NEPA Cumulative impacts, including impacts from connected or similar actions are address section 5.3 and Volume 2, section 5.15; residual impacts, assuming this term applie cannot be avoided, are addressed in Volume 1, section 5.4 and Volume 2, section 5.1 monitoring and safety programs that are open to public review. Activities includin associated with SNF management are discussed throughout Volume 1 and its appendices issues being considered.

## II COMMENT

The commentor objects to DOE making a decision on the proposed alternatives when in extent of impacts to the Snake River Plain aquifer is not complete.

#### RESPONSE

The commentor refers to Volume 2, section 5.8.1. A sentence specifically refers to analysis for the impacts of a hypothetical leak to the soil from an SNF storage fac Volume 2, section 5.8.1 states that based on the bounding accident scenario for imp Plain aquifer discussed in Volume 2, section 5.14, the impacts to the Snake River P expected to be small. These hypothetical impacts are assessed with respect to EPA Subsequent analysis of the hypothetical SNF storage facility leak and documentation groundwater modeling for the EIS have been referenced in and are available with the are consistent with conclusions stated in the EIS regarding the impacts of alternat Water resources at INEL and potential impacts from the alternatives considered in t Volume 2, sections 4.8, 5.8, and Appendix F. In accordance with NEPA regulations a the EIS summarizes all known existing credible scientific evidence relevant to unde environment, identifies reasonably foreseeable impacts, and evaluates potential con Assumptions and limitations in the groundwater analysis are identified in Volume 2, used conservative assumptions to account for the uncertainty in modeling the effect on groundwater quality. Results indicate that under all the alternatives considere contaminants above EPA MCLs at the INEL site boundary as a result of operations und proposed alternatives. This would be a result of waste management practices, inclu discharge monitoring, as well as natural contaminant attenuation and radioactive de releases.

Independent assessments of the Snake River Plain aquifer water quality at INEL conf environmental monitoring results that no contaminants in concentrations above EPA M exist beyond the INEL boundary. With improved management practices and remediation or under way, it is likely that overall water quality in the Snake River Plain aqui continue to improve.

## II COMMENT

The commentor asserts that the conclusions on potential impacts are flawed and that on these conclusions, fails to pick the best solution.

#### RESPONSE

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environme all the alternatives considered in this EIS. The analyses show that the impacts of small. While there are differences in the impacts among the alternatives, these di are not sufficient to distinguish between alternatives.

Volume 1, section 3.1 and Volume 2, section 3.4 describe DOE's preferred alternativ SNF management, and SNF management, environmental restoration, and waste management

## II COMMENT



Many commentors state that the EIS needs to provide cumulative impact assessments of activities at the Idaho National Engineering Laboratory.

#### RESPONSE

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environment including cumulative impacts, of all the alternatives considered in this EIS. The impacts of all alternatives would be small. Each alternative includes the appropriate Volume 2, including decontamination and decommissioning

Volume 2, Chapter 4 discusses the current environment at INEL, including impacts from waste streams and emissions from INEL facilities, including characterization data and inventories, are referenced in Volume 2, Appendix F.

Volume 2, Appendix C discusses 49 proposed projects and ongoing activities at INEL. analyzed under each of the alternatives discussed in Volume 2, section 3.1 and include foreseeable actions. These actions are subject to the outcome of negotiations with the FFA/CO.

Mitigation measures are discussed in Volume 1, section 5.7 and in Volume 2, section See also the response to comment 04.01 (001).

## II COMMENT

The commentor states that while measuring small quantities, DOE loses sight of overall geology, and the national budget.

#### RESPONSE

DOE used the process described in regulations to ensure that the procedural requirements are satisfied. The scope of Volume 1 of this EIS is to evaluate impacts directly related to activities across the United States. The scope of Volume 2 is to evaluate impacts from management, environmental restoration, and waste management activities at INEL. In addition to the activities associated with the proposed action, plus past, current, and other activities are evaluated in Volume 1, section 5.3 and Volume 2, section 5.15. The evaluation includes an evaluation of the overall impacts to the human and natural environment, geological resources. Costs of the alternatives are summarized in Volume 1, section

## II COMMENT

The commentor states that the EIS violates the National Environmental Policy Act because impacts do not include an evaluation of supply and demand; for example, the demand for waste will increase.

#### RESPONSE

Volume 2 considers the potential consequences of a range of levels of waste and SNF. Under the Maximum Treatment, Storage, and Disposal alternative, INEL would respond to increased demand for management of waste and SNF. The assessment found that the impacts of other alternatives would be small. Cumulative impacts are included in the assessment of these impacts in Volume 2, section 5.1.5.

## II COMMENT

The commentor states that the EIS does not provide historical data on radioactive materials. National Environmental Policy Act requirements must be met in the EIS by providing an evaluation of cumulative impacts for past and proposed activities at the Idaho National Engineering Laboratory.

#### RESPONSE

Waste streams and emissions from INEL facilities, including characterization data and inventories, are included as references in Volume 2, Appendix F. Volume 2, Chapter 4 discusses the current environment at INEL, including impacts from past activities. The effects of the proposed action on INEL, as discussed in Volume 2, Chapters 2 and 4, and potential effects of the proposed reasonably foreseeable actions not associated with the proposed action, have been evaluated in Chapter 5.

## II COMMENT

The commentor takes the position that all projects included in the Nevada Test Site considered in the cumulative impact analysis for that site.

### RESPONSE

A site's master plan identifies all the projects desired to fulfill the current sit without regard to budgetary constraints, priorities, or current direction. It repr planning process, and remains relatively static. Projects are not well defined in contrast, the site 5-year plan presents more thorough development and definition of master plan that might be initiated or implemented over a 5-year period. In the 5-updated annually, projects are prioritized in light of the current site needs, budg current policy and direction. Because the 5-year plan identifies the mission-crit most likely to be funded and completed, it is a better indicator of planned activit master plan. Due to the nature of the planning and budget cycle, the 5-year plan i that are likely to be implemented in a 5-year period, but provides a longer perspec the 5-year plan is considered to be an appropriate basis for identifying projects f impacts. Additional discussion of the site master plan and relation to the 5-year impacts were added to Volume 1, Appendix F, Part Two.

Due to the nature of the planning and budget cycle, the 5-year plan is not limited to be implemented in a 5-year period, but provides a longer perspective. For these is considered to be an appropriate basis for identifying projects for analysis of c

Appropriate sections of the Nevada Test Site's Master Plan Projects are summarized Appendix F, Part Two.

## II COMMENT

The commentor asks that the EIS address the cumulative impacts from existing waste over the next 40 years at the Idaho National Engineering Laboratory. In addition, the EIS address the cumulative impacts from the waste at the Hanford Site and the p waste from the Trojan Nuclear Power Plant in the Columbia River basin.

### RESPONSE

Volume 1, Chapter 5 Appendix K, and Volume 2, Chapter 5 summarize the environmental the alternatives considered in this EIS. The analyses show the impacts of all alte The EIS addresses the cumulative impacts from current and future waste at INEL in V 5.15.

The EIS addresses the cumulative impacts from waste at the Hanford Site on the Colu Volume 1, Appendix A, section 5.8. The Trojan Nuclear Power Plant has operated wit accordance with 10 CFR 20. Operation ceased on November 9, 1992. On January 4, 19 General Electric Company announced that the plant would not restart, and the plant January 27, 1993. The decommissioning plan was submitted to NRC on January 26, 199 spent fuel management for the Trojan plant. This is outside the scope of this EIS.

## II COMMENT

In supporting the preference for the Planning Basis alternative, the commentor stat fully address the cumulative impacts (specifically to public health and safety) of many different proposed sites under the various alternatives, and states that addin duplicating them at several sites may negatively impact safety.

### RESPONSE

This EIS evaluates 10 sites as reasonable siting alternatives for some level of SNF The analysis in the EIS includes environmental considerations, socioeconomic impact the public from operations and reasonably foreseeable accident conditions, site-spe and other environmental factors for a number of options for managing SNF. Cumulati site-specific projects or missions that are planned to occur simultaneously with SN are discussed in Volume 1, Appendices A through F. The EIS concludes that the alte environmentally suitable for management of SNF, and that risks to the public or the

managing SNF at any of the 10 sites under consideration would be small even when ne involved. Discussions on public health and safety can be found in Volume 1, section site-specific Appendices A through F, and in Volume 2, section 5.12.

## II COMMENT

The commentor states that socioeconomic impacts are not fully addressed from a cumu therefore, socioeconomic impacts are underestimated. The commentor suggests that, point be included under "cumulative effects" that there are large socioeconomic imp before the project starts. Further, the commentor suggests that the EIS not assume mitigation measure of payments in lieu of property taxes unless a specific plan is commentor states that DOE does not pay property taxes and rarely makes payments in taxes.

### RESPONSE

The commentor is correct that DOE facilities generally do not pay local or state pr various mechanisms exist for DOE to compensate state or local governments in the fo lieu of taxes or "special burden" payments. Special burden payments help offset in and population caused by DOE facility construction and/or operation (which may put on local services, utilities, and infrastructures). Each situation requires an ind determine whether such payments would be authorized to the appropriate state or loc assessing socioeconomic impacts, the EIS does not presume that payments in lieu of states or local communities, but only discusses the possibility of such payments as adverse impacts.

Volume 1, Appendix F, Parts Two and Three, section 5.16 discuss potential socioecon cumulative perspective. These sections do not explicitly "identify large socioecon occurred before the start of the proposed project." Rather, potential cumulative are discussed in terms of "the impact on the environment that results from the inc action when added to other past, present, and reasonably foreseeable future actions socioeconomic impacts from the SNF management alternatives are compared with baseli demographic forecasts. The effects on these regional economic growth rates from pr management are relatively insignificant. DOE would evaluate the need to implement adverse socioeconomic impacts on a site-specific basis.

Impact avoidance measures discussed in Volume 1, section 5.7.2 of the EIS could be site-specific basis when more detailed socioeconomic analyses are conducted. Altho pay property taxes to local jurisdictions, Federal and civilian employees working a employed by sites, do. Infrastructure projects such as roads and other capital exp are financed by the Federal Government, reducing the fiscal impact on public financ jurisdictions.

## II COMMENT

The commentor states that the EIS is defective because Volume 2 does not adequately cumulative effects of shipping, receiving, processing, and storing nuclear waste at Engineering Laboratory; nor does it address the cumulative impacts of past disposal present management actions, and reasonably foreseeable actions regarding spent nucl commentor expresses concerns about the cumulative impacts to the Snake River Plain the Idaho National Engineering Laboratory and the level of detail in project summar cumulative impacts, and cites an example.

### RESPONSE

Volumes 1 and 2 of the EIS comply with CEQ regulations regarding assessing the cumu programmatic SNF management and SNF management, environmental restoration, and wast at INEL, respectively. The regulations at 40 CFR 1508.7 define "cumulative impacts from the incremental impact of the action when added to other past, present, and re future actions. Cumulative impacts of SNF management activities at INEL are discus Appendix B, section 5.16. Impacts of past practices and present conditions at INEL Volume 2, Chapter 4. This serves as a baseline to add incremental cumulative impac actions, as in Volume 2, Chapter 5. The projects described in Volume 2, Appendix C one or more of the alternatives considered in Volume 2; their combined impacts are, the analyses of environmental consequences in Volume 2, Chapter 5. Volume 2, secti

comprehensive discussion of cumulative impacts (including the Snake River Plain aqu past, present, and future actions of DOE and others. Topics addressed include land cultural resources, air, water, transportation, health and safety, waste management resources. To aid in readability, many of these impacts have also been described i The detail in Volume 2, section 5.15 is commensurate with the current state of plan development of such potential activities, including the example cited by the commen presented only to the extent known or reasonably foreseeable. Table 5.15-1 describ both onsite and offsite, that are not part of the proposed action but that have bee cumulative impact analysis. Volume 2, presents nonhealth-related transportation an cumulative impacts from the proposed, connected, and similar actions. See also the 05.09 (011).

See also the response to comment 05.09 (006) regarding impacts to the Snake River P

## II COMMENT

The commentor states that the EIS is inadequate because it fails to address the cum fuel shipments as they pertain to Idaho National Engineering Laboratory-specific pr  
RESPONSE

The impacts due to SNF shipments are described in Volume 1, Appendices D and I. Cu from SNF shipments are described in Appendices D and I for both radiological impact accident impacts. Cumulative impacts due to past activities are presented for each the start of operations at a site to 1993. Impacts through 2035 are in a range for alternative as an upper bound, which lends conservatism to the evaluation for alter transportation. No other cumulative impacts are related to transportation; thus, t

## II COMMENT

The commentor states that longshoremen, sailors, and the general public will receiv commercial shipping lanes are used and waste casks are off loaded in Portland, Oreg Washington, and trucked to the Hanford Site or Idaho National Engineering Laborator traffic in cars alongside or behind these waste shipments could receive a significa  
RESPONSE

DOE believes the EIS and reference documents contain an adequate discussion of dire cumulative impacts of transporting SNF and other radioactive materials. Incident-f hazardous materials results in essentially no impacts as discussed in Volume 1, sec of highway, railway, and barge transportation impacts and potential accident impact Environmental Consequences of Key Disciplines and Offsite Transportation of SNF sec The cumulative impacts analyses are discussed in Volume 1, section 5.3 and Volume 2 DOE conducted a comprehensive transportation cumulative impacts analysis, evaluatin future shipments of radioactive material, which include radioactive waste and SNF. cumulative impacts analysis includes past transportation activities, transportation actions in this EIS, reasonably foreseeable future transportation activities, and g activities.

The analyses described in Volume 1, section 5.3 and Volume 2, section 5.15.7 show t exposing the public to radiation hazards is extremely low and the overall impacts u alternatives analyzed in this EIS would be small.

## II COMMENT

The commentor notes that the second sentence in Volume 2, section 5.7.4.3.2 incorre cumulative impacts from all major sources after the baseline date must be below Pre Deterioration increment limits. Increases from minor sources also consume incremen  
RESPONSE

The commentor is correct in that it should be clarified that increases from both ma after the baseline date consume increment. In fact, the increment consumption anal performed considered all applicable sources that became operational (or will become baseline dates. The sentence has been revised to clarify that the PSD analysis con applicable major and minor source emissions that occur after the baseline dates.

## II COMMENT

The commentor states that all alternatives except the No Action alternative have a releases to the environment, which will exacerbate existing contamination by both c radiological materials.

### RESPONSE

The proposed SNF facilities are designed to have no liquid release of waste water w or radiological characteristics through the use of modern technologies, including s leak detection, and water-balance monitoring equipment. The analysis in the EIS in considerations, potential risks to the public from operations and reasonably forese conditions, site-specific cumulative effects, and other environmental factors for a managing SNF. Cumulative effect, involving existing site problems and site-specifi that are planned to occur simultaneously with SNF management activities are discuss Appendix F. The EIS concludes that the alternative sites are environmentally suita SNF, and that risks to the public or the environment due to SNF management would be new missions are involved.

Discussions on public health and safety are in Volume 1, sections 5.1 and 5.3 and s F.

## II 5.10 Safety and Health Effects

## II COMMENT

One commentor questions the use of legal limit radiation levels for DOE spent nucle and measured radiation levels for U.S. Navy spent nuclear fuel shipping casks.

### RESPONSE

Using legal limit radiation levels will overestimate potential impacts from DOE SNF assumption was necessary to maintain flexibility in the specific choice of shipping used by DOE. Even with this assumption, the risks are still small. The Navy inten shipping casks, which have been in use and for which there are measured radiation l required; therefore, these realistic measured data were used, and it was not necess assumptions to bound potential impacts.

## II COMMENT

The commentor states that transportation of radioactive materials involves minimal RESPONSE

The comment accurately reflects the analyses of impacts provided in Volumes 1 and 2 Volume 1, Appendices D and I. Volume 1, Appendix I summarizes the methodologies, k assumptions, and results of calculations for the transportation analyses. These an associated with the transportation of radioactive material would be small for all a The conclusion that such risks would be small is borne out by past experience with

## II COMMENT

The commentor states that traffic fatality risks are somewhat higher for Naval than The commentor states that the analysis uses the same documents for both Naval and n estimates and does not consider the increased non-Naval shipments.

### RESPONSE

Off-site shipments of non-Naval fuel are discussed in Volume 1, Appendix 1, while o non-Naval fuel are discussed in Volume 1, Appendices A, B, C, and F, for Hanford, I NTS/ORR, respectively. Off-site and on-site shipments of Naval fuel are discussed D.

DOE and the Navy reviewed their analyses of traffic fatality risks and did not iden

impacts would be small for both radiological and nonradiological risks. The differ between Naval and non-Naval SNF was considered in the analyses. When comparing Naval and non-Naval transportation impacts, some differences other t shipments are important. For example, all off-site Naval SNF shipments from shipya whereas all off-site test specimen shipments are by truck. The results are present number of each of these types of shipments. DOE shipments assume that all off-site by rail or by truck, and results are presented for both cases. Another example is shipments from Pearl Harbor have a portion of the trip on ocean transport vessels. for accident rates (Longitudinal Review of State-Level Accident Statistics for Carr Freight) lists a significantly higher nonradiological casualty rate for ocean trans nonradiological fatality rate listed for rail or truck transport.

## II COMMENT

The commentor states that the probabilities for transportation accidents represent single shipment. The commentor states that probabilities should be determined on t shipments, not individual shipments.

### RESPONSE

The results of the transportation accident risk assessment are cumulative risks tha shipments over the entire campaign (1995 to 2035). Probabilities for the maximum r transportation accidents are annual probabilities based on the total annual shipmen

## II COMMENT

Commentors identify issues regarding public and worker safety and risks, and the ef due to accidents caused by extreme weather and natural disasters at the facilities.

### RESPONSE

Volume 1, Chapters 3 and 5 and Appendices A through F, and Volume 2, Chapters 3 and F discuss risks to the public, workers, and the environment due to facility accident by extreme weather and natural disasters, such as high winds, floods, earthquakes, discussions include extensive evaluations and analyses of accidents. Protecting me public and workers from accidents is considered by DOE in the design, location, con operation of facilities. The analyses and other information in the EIS demonstrate and the public from all accidents, including those caused by extreme weather or nat small for all of the alternatives considered.

## II COMMENT

The commentor suggests that the EIS fails account for the long-term risks to the pu liability costs from damage scenarios under various options.

### RESPONSE

Even for INEL accidents with the maximum reasonably foreseeable consequences, and w unfavorable meteorological conditions, no long-term risks to the public are expecte 2, section 5.14, there is a potential for limited economic impacts associated with to public lands or up to a 1-year agricultural land withdrawal for land on and imme INEL. Relative to potential liability costs, DOE will use the statutory indemnity Price-Anderson Act (42 USC 2210) to ensure ready and prompt availability of funds t public for injuries and damages resulting from a nuclear incident arising from acti indemnified DOE contractors. Compensation provided under the Act would cover nucle arising at INEL, as well as nuclear incidents arising during the transportation of site.

Although the Price-Anderson Act is the primary means for compensating the public fo nuclear incidents, other remedies exist for claims not falling within the purview o claims against DOE or its employees may be cognizable under the Federal Tort Claims for environmental damage may fall within CERCLA. These and other laws afford any i mechanisms for seeking recovery for damages relating to operation of DOE facilities

## II COMMENT

The commentor suggests that DOE is not going to study ingestion of radioactive material at the National Engineering Laboratory because contaminated food and water would be impounded. The commentor also states that DOE's assumed cleanup of accidents does not account for particles by wind.

### RESPONSE

For INEL facility accidents with the maximum reasonably foreseeable consequences under unfavorable meteorological conditions, some restrictions on uses of agricultural products have been implemented in accordance with established Protective Action Guides. However, this ingestion of radioactive material has not been analyzed in the EIS. There has been potential for health effects through ingestion, as well as other pathways, and is discussed in Appendices A and F-4. The accident assessments summarized in Volume 2, section 5.1, discuss ingestion of radioactive materials. Resuspension of radioactive materials from the potential dispersion path. Wind-borne resuspension generally reduces the amount of material at distance from the point of release, but increases the area in which exposure might occur. Analyses generally did not take credit for mitigative measures. Nevertheless, the potential for exposure of workers from all accidents analyzed in the EIS would be small.

## II COMMENT

The commentor states that agency officials should be able to answer over the telephone such as what is the longevity of radioactive spent fuel.

### RESPONSE

Because agency officials are accountable for answers to technical information given over the telephone, it is unreasonable to expect all technical information to be immediately available to the telephone. In addition, agency officials consider it prudent to check answers, against available references or with technical experts before providing them to the public. Whenever possible, questioners were intentionally referred to specific locations where they would answer their questions in detail, in language agreed to by a wide range of representatives.

## II COMMENT

The commentor would like DOE to minimize worker and public exposure to radiation during operation, and maintenance activities, using the principle of the "as low as reasonably achievable" approach.

### RESPONSE

Maintaining occupational exposure to radiation and radioactive materials as low as reasonably achievable (ALARA) is an integral part of all site radiological control programs. In addition to implementing legally applicable radiation protection standards and to consider and adopt recommendations by authoritative organizations. Examples of such standards and organizations are DOE Order 5400.5, Radiation Protection of the Public and the Environment, the National Council on Radiation Protection and Measurements, and the International Commission on Radiological Protection. See also the response to comment 05.10 (029).

## II COMMENT

Commentors express the opinion that all facets of DOE's nuclear program are lethal to the protection of bureaucrats.

### RESPONSE

Hazardous material resulting from DOE's past, present, and future nuclear programs will be disposed of in a safe manner. This EIS addresses the programmatic management of SNF ultimate disposition, as well as environmental management activities at INEL over time. It concludes that there would be no significant environmental impacts under any of the alternatives being considered for implementation. Although vulnerabilities exist, DOE has the skill, scientific capability, and Secretarial mandate to safely manage SNF and INEL.

environmental restoration activities in the period covered by this EIS.

## II COMMENT

The commentor notes a typographical error on the first line of the last paragraph of Summary.

### RESPONSE

The commentor is correct that the word "facilities" should be "fatalities." DOE has corrected the Summary and in Volume 1.

## II COMMENT

One commentor refers to the degraded conditions in the Idaho National Engineering Laboratory as assessed by the Spent Fuel Working Group. This individual states that the known storage of spent nuclear fuel lead to the risk of radioactive contamination, health criticalities, meltdown, and explosions. Another commentor wants DOE to "address the problems that are a danger to us all."

### RESPONSE

Volume 2, section 2.2 discusses the vulnerability of SNF storage at INEL. Actions and vulnerabilities are identified in Volume 2, section 2.2, Table 2.2-1. Because of the criticality identified in Volume 2, section 2.2, a criticality at Building 603 at the Idaho Chemical Processing Plant was considered 10 times more likely than at a modern facility such as Building 666. Such a criticality is reported in Volume 2, section 5.14. The impacts to the public would be small; impacts to workers at the scene could vary depending on the circumstances of shielding by water and concrete, it is not likely that radiation exposure would be significant. Workers could have an increased risk of developing cancer over their lifetimes.

## II COMMENT

The commentor requests clarification of the phrases "high, though not fatal, dose" and "likely" in Volume 1, Chapter 5.

### RESPONSE

The phrases were used in reference to an estimated worker dose of 120 rem resulting from an accident. A dose of 120 rem is considered to be a dose with large potential health impacts. The population that receives short-term exposures may have individuals who die from a nominal dose level for death to an individual with no medical intervention is 300 times more likely than for an individual with no medical intervention at lower doses. Thus, a short-term dose of 120 rem could result in death in part of a population. A short-term dose of 120 rem could be considered a fatal dose for typical individuals. Occupational doses to workers are limited to 5 rem per year.

DOE has modified the EIS to clarify the phrases.

## II COMMENT

The commentor questions what number of latent cancer fatalities per year DOE considers acceptable.

### RESPONSE

DOE considers seriously the relationship between radiation exposure and the potential for cancer fatalities. Rather than a "number" of fatalities that is considered significant, DOE considers the likelihood of a latent cancer fatality to a member of the public or in its work for Nuclear Safety Policy states that "the general public be protected such that no individual be exposed to an additional risk to health and safety from the operation of a DOE nuclear facility a member of the general population are exposed." Quantitatively, the goal is to translate the chance of a fatal cancer to a member of the public of one chance in 500,000 per year of operations.



## II COMMENT

The commentor suggests that consistent definitions of maximally exposed individual exposed off-site individual (MOI) are needed. The comment cites the definitions in and text in Volume 2, Appendix F-3.

### RESPONSE

The definitions in Volume 1, Appendix H agree with the text in Volume 2, Appendix F. Volume 2 contains an expanded discussion of the details involved in evaluating the individual, appropriate for a site-specific NEPA document. The less-detailed text in Volume 1, Appendix H is appropriate for a programmatic analysis.

## II COMMENT

The commentor suggests that duplication of facilities and missions at several sites "safety" and, in fact, can degrade the safety posture of those facilities.

### RESPONSE

Volume 1, Chapter 5 summarizes the radiological and health and safety impacts associated with alternatives considered in this EIS, including using existing facilities and construction alternatives. Section 3.3.6 summarizes the cost evaluation. The health and safety of workers considered in the evaluation of these alternatives and the identification of a preferred alternative are provided on radiological and health and safety impacts, including facilities adequate for evaluating and comparing the impacts of all the alternatives. Volume 1 has been revised to indicate that there are no widely accepted equivalence values between radiological exposures or other health effects or environmental impacts.

See also the response to comment 04.04 (008) for management of SNF under DOE's preferred alternative.

## II COMMENT

The commentor states that a sentence in the Summary on public and worker health effects deleting the sentence.

### RESPONSE

The sentence states that radiation exposures also occur from natural sources. DOE has added for the reader to understand that natural radiation also contributes to the exposures. The EIS has been changed to clarify the intent of the discussion.

## II COMMENT

The commentor suggests that atmospheric testing be added to a discussion about underground releases.

### RESPONSE

Volume 1, section 4.4 has been changed to include discussion of atmospheric testing and releases.

## II COMMENT

Commentors suggest that adequate baseline health studies need to be conducted at all along transportation routes, and at proposed DOE sites to support risk factors used in the EIS. Commentors request that all epidemiological studies be included in this EIS, or if not performed, explain why and what other public involvement activities were conducted.

### RESPONSE

In March 1990, DOE announced that it will turn over responsibility for research on health effects to workers at DOE facilities and the public in surrounding communities to the Health and Human Services. DOE directed that all worker health and exposure data and releases of radioactive and toxic materials be released. Baseline health effects studies

workers and for members of the surrounding public are either under way or planned a facilities. Results of all studies are available to the public. Some persons have proposed performing epidemiological studies of the people living vicinity of installations performing work associated with atomic energy. However, studies that have been attempted, such as those in Great Britain, the level of radiation communities from man-made radionuclides is very low with respect to the variations radiation and other factors introduced by individual lifestyles. This, plus other nature and other industries in the communities, has made it impossible to perform a develop definitive conclusions. Efforts in this area are expected to continue, but study, the standards of the International Commission on Radiological Protection report data available.

The epidemiological studies of baseline health effects at all existing DOE sites are decisionmakers to discriminate between the alternatives discussed in this program are not relevant to any reasonably foreseeable adverse impacts.

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environment all the alternatives considered in the EIS. The analyses show that none of the alternatives adverse impacts.

## II COMMENT

The commentor notes that cancer fatalities are in the Summary as "one" for all alternatives  
RESPONSE

The values in the Summary were chosen for simplicity of presentation. The analyses appendices of the EIS provide health effects estimates for each site and alternative over a wide range and depend on a variety of factors. However, in all cases, the alternative than one fatal cancer would result from the activities under each alternative.

## II COMMENT

The commentor questions whether radiation from past practices may be the cause of health effects in the area and discusses previous releases and accidents at DOE sites  
RESPONSE

Analysis of impacts from past releases and accidents at DOE sites is not within the however, it is DOE policy to identify and correct any inadequate practices concerning arising from operation of its facilities. In this regard, accidents and accidental reported, and accidents resulting in significant releases from DOE facilities are in monitoring reports that are publicly available. Detailed descriptions of the event accidents or releases are outside the scope of the EIS. The Hanford Environmental project currently is evaluating past releases from the Hanford Site.

Analyses in the Health and Safety sections of both volumes of the EIS evaluated potential off-site public from both radiological and nonradiological hazards for actions resulting in this EIS. For all alternatives, impacts were estimated to be small, hypothetical one additional fatal cancer in the surrounding population over that which would occur presence of these DOE activities.

## II COMMENT

The commentor notes that insufficient information is provided on dose assessment method verification of the accuracy and representativeness of the predicted impacts and dose  
RESPONSE

Methods for estimating releases to water are described in Volume 2, Appendix F-2. estimating releases to air are described in Volume 2, Appendix F-3. Exposure and risk methodology is described in Volume 2, Appendix F-4. Additional information is available reference material available in public reading rooms and information locations list

## II COMMENT

The commentor suggests that nonradiation workers, visitors, and motorists at Idaho Laboratory should be defined as the maximally exposed individuals, rather than a si  
RESPONSE

Potential exposure to nonradiation workers, visitors, and motorists at INEL has been radioactive and nonradioactive releases from site facilities. Descriptions of the situations are contained in Volume 2, sections 5.7 and 5.12. Further information is in Volume 2, Appendix F-4. Although such individuals may be closer to some site facilities, workers spend only about 2,000 hours each year at the site; visitors and motorists have additional pathways for exposure are included for site-boundary residents that do not include visitors, and motorists. In particular, the potential for ingestion of radioactive materials is included in the evaluation for site-boundary residents.

## II COMMENT

The commentor points out that risk estimates for all alternatives are higher for this site than for other sites.

### RESPONSE

The estimates of risk from releases of radioactive and nonradioactive materials and on many factors. These include characteristics of the local population distribution, groundwater, and surface water. They also include the characteristics of the facilities addressed under each alternative. The assessment methods used for each site are described in Appendices A through F.

Specific information on the risks associated with the alternatives considered for Site 1, Appendices C and D, Chapter 5. The analyses in this EIS show that the risks for Site 1 are considered to be small.

## II COMMENT

Commentors question the effects from exposure to radiation and the methods for reporting and suggest that the EIS may not have used the most up-to-date or most accepted radiation effects dose response factors, particularly as related to induction of cancers.

### RESPONSE

The potential health effects from exposure to radiation are the subject of research throughout the world. Some published results have been subjected to enough review and confirmation by the community to become well accepted. Others have not stood up to careful scrutiny. Some are interesting, but unproven, hypotheses. None of these individual studies provides a basis for the risk factors necessary to support the type of analysis required for the EIS. These studies, by themselves, are not a technically sound basis for setting radiation standards or making decisions. The dose response factors for cancer induction used in the EIS were taken from the International Commission on Radiological Protection recommendations (1990 Recommendations of the International Commission of Radiological Protection), which reflect the most recent accepted analysis of all currently available data. The authors of ICRP 60 reviewed Volume 1, Appendix D and Volume 2, Appendix F-4 provide useful primers on radioactive dose, and resulting health impacts. Volume 2, Appendix F-4 provides a discussion of how cancer risks were calculated and how cancer risks were estimated.

## II COMMENT

The commentor questions how tritium could be present in urine after 400 days if its half-life is roughly 12 days.

### RESPONSE

The biological or retention half-life does not refer to the period of time required for the material to be eliminated from the body. It is an estimate of the time for half the material to be eliminated. If the remaining tritium will be eliminated in another 12 days, leaving one fourth of the original amount, then this amount will be eliminated in the next 12 days, and so on. It is possible that detectable amounts would be eliminated 400 days later. Additional background information is provided in the EIS.

radiation and its effects can be found in Volume 2, Appendix A.

## II COMMENT

Commentors suggest that the discussions of radiation and the term "latent cancer fa or insensitive.

### RESPONSE

The terms used in the EIS are not intended to be misleading or insensitive. They a used to describe the impacts being evaluated. A glossary is provided in the EIS to technical terms. With regard to the effects of radiation exposure, basic informati Volume 2, Appendix A. More detailed information is in Volume 2, Appendix F-4.

## II COMMENT

The commentor identifies specific inconsistencies within the EIS.

### RESPONSE

The section on Public and Worker Health Effects in the Summary has been modified to estimated health effects to the public include both operation activities and routin collective dose estimate provided in Volume 1, section 5.3.2 is to the worker, whic to the public.

## II COMMENT

The commentor asks whether the Advanced Neutron Source Facility and the Expended Co should be included in the assessment of potential impacts for the Oak Ridge Reserva

### RESPONSE

The Expended Core Facility was included in the analysis of potential SNF facilities Advanced Neutron Source Facility was evaluated separately. Both were included in e the maximally exposed individual. These assessments are in Volume 1, Appendix D an Appendix F, Part Three.

## II COMMENT

The commentor states that preservation of life and protection of property should be what government-sponsored activities are allowed.

### RESPONSE

The health and safety of people and the protection of property are accorded appropr deciding what activities could be implemented by the government (e.g., DOE Order 5 Environmental, Safety and Health Program for DOE Operations, Section 7, and EIS Vol Summary).

## II COMMENT

The commentor states that cancer morbidity, not just cancer fatality, should be use impact of radiation exposures.

### RESPONSE

The analyses of the potential effects of radiation exposure in this EIS do consider cancer fatalities and are based on the standards of the International Commission on Protection. Volume 1, Appendix D, Attachment F, section F-1.3.3 and Volume 2, Appe the terminology and risk factors used by the International Commission on Radiologic these factors were applied in calculating the effects on human health in this EIS. The International Commission on Radiological Protection defines "health detriments" of all fatal cancers, nonfatal cancers, and genetic effects. The health detriments radiation are calculated by summing all of these effects after multiplying each eff

intended to represent the severity of the impact of each type of effect on human health. EIS, the total health effects (deaths, nonfatal cancers, genetic effects, and other) may be obtained for the public by multiplying the latent cancer fatalities by the factors from the International Commission on Radiological Protection. Cancer fatalities were used to summarize and compare the results in the EIS because they are of the greatest interest to most people.

## II COMMENT

The commentor questions the accuracy of information in Volume 1, Appendix A on Hanford nuclear fuel management.

### RESPONSE

The information has been updated and the text clarified.

## II COMMENT

The commentor notes that the Summary presents numbers of fatal cancers in the population at each site for each alternative but does not give the sizes of the populations so that they are estimated.

### RESPONSE

Several factors in each site analysis affect the estimate of cancer fatalities, including which are different for each site. These data are provided in Volume 1, Chapter 5. The EIS has been changed to reference Chapter 5 to identify the source of this information. The EIS is presented in a fashion with respect to technical depth of information. The Summary was intended to provide information so that it would be generally understandable to nontechnical persons. Each volume presents expanded information with more technical detail, but are still accessible. The remaining chapters in each volume provide the technical information needed to support the conclusions. The appendices are the most technically detailed and provide sufficient information for a thorough technical review by specialists. The appendices also provide references to information on the methods and results of technical analyses.

## II COMMENT

The commentor asks why the computer code used to estimate health risks associated with releases was not site-specific.

### RESPONSE

Because the purpose of the analysis was to allow comparison among the alternatives, use of the same source input is appropriate. The computer codes used to estimate health risks with releases from DOE facilities allow the input of site-specific data. Wherever site-specific data was used for such input parameters as source terms, hydrology, and demographics. A generic meteorology classes D, E, and F were used in modeling, no credit was taken for site-specific height.

## II COMMENT

The commentor refers to Volume 1, Appendix B, section 5.12 and raises an issue that chemical impact focuses on cancer health effects. Some chemicals cause adverse noncancer effects at exposure levels below those predicted to cause unacceptable increases in cancer. The commentor states that the potential for synergistic effects from hazardous chemicals should be considered whenever possible.

### RESPONSE

Potential synergistic effects from multiple chemical exposures are extremely difficult to quantify because there is insufficient data to indicate synergistic effects. The potential for synergistic effects is small where the concentrations for each individual compound are low. For the alternatives evaluated in this EIS, to ensure that potential impacts are based on high releases and exposure conditions were assumed. Further, the point of highest potential impact was used.

chemical occurs at different times and places. It is unlikely that any one individual more than one chemical species at the concentrations reported in this EIS. Radiation doses from historic operations are discussed in Volume 2, section 5.15.8. available in referenced technical documents, which are available for review in read information locations listed in the EIS. DOE is not aware of any generally accepted methodology that has been developed to evaluate synergistic effects due to several constituents. DOE is aware that research into this area is continuing. The evaluation of cumulative effects considers historic accidents only. The implementation NEPA at 40 CFR, Paragraph 1508.7 specifies "that cumulative impacts result from past reasonably foreseeable future actions..." For cumulative impacts, DOE has considered "reasonably foreseeable" to include construction, operation, maintenance, and other not to include future hypothetical accidents, inadvertent spills, and other unplanned chemical exposure resulting from an accident is evaluated in Volume 2, Appendix F-5 response to comment 05.10 (021).

## II COMMENT

The commentor points out an apparent inconsistency between the dose reported in the waste disposal operations and the dose given in the Radioactive Waste Management Co Waste Radiological Performance Assessment.

### RESPONSE

The commentor is correct that the doses reported in the cited reference for the post period exceed those cited in the EIS for the operational period. However, the dose directly comparable because the assumptions used in each analysis are significantly cited in the EIS are evaluated at the site boundary and represent an upper bound for during the time period addressed in the EIS. The doses cited in the RWMC performance the post-institutional control doses evaluated for a location very near the waste (meters away) and represent an estimate of doses more than 100 years outside the time the EIS. During this post-institutional time period, it is assumed that no control individual from approaching the waste disposal complex. Therefore, it is reasonable post-institutional control period could exceed those cited in the EIS for the operation. Further, the doses reported in the RWMC performance assessment do not account for part of the RWMC under the CERCLA process. These remediation activities could significantly radiation doses expected from the RWMC over the long term.

The evaluation in Volume 2 of this EIS bounds environmental impacts from environmental cleanup activities at INEL. However, specific decisions related to cleanup at INEL addressed under an enforceable agreement executed by DOE, EPA Region X, and the State December 9, 1991, the FFA/CO. The FFA/CO establishes a comprehensive process that remediation requirements of CERCLA and the corrective action requirements of RCRA at Idaho's Hazardous Waste Management Act. Cleanup activities are conducted under the schedule established in the FFA/CO. RODs under the FFA/CO process are signed by a and represent a joint determination that protectiveness will be achieved through the selected remedy.

## II COMMENT

The commentor asks why the GENII-S computer code was not used for Hanford Site assessment the GENII used in the EIS.

### RESPONSE

GENII-S incorporates the same models and data inputs for dose parameters used. The codes yield the same results when used in the deterministic mode. However, GENII-S capability to calculate the uncertainty in the atmospheric dispersion factors or the receptor. These calculations are important particularly where short-lived radionuclide contributors and distances are long. The model does not allow for any decay on the and thus, overestimates releases. Considering these limitations, the use of GENII

## II COMMENTS

The commentor requests that radiation doses, expressed in effective dose equivalent Brookhaven National Laboratory, as well as for other sites considered in the EIS.

RESPONSE

Radiation doses, expressed in effective dose equivalent, are provided for current a National Laboratory in Volume 1, Appendix E, Chapter 3. The evaluation of potential alternative is in Appendix E, Chapter 4. No additional quantitative estimates of r presented in Chapter 4, because none of the alternatives would result in an increase Brookhaven National Laboratory.

## II COMMENT

The commentor asks whether the dose factors in Volume 2, Appendix F-4, Table F-4-5 by inhalation or ingestion.

RESPONSE

The values in Volume 2, Appendix F-4, Table F-4-5 are for unit intakes by ingestion referenced in the text under a discussion of dose evaluation for consumption of con

## II COMMENT

The commentor suggests that the high efficiency particulate air filter efficiency d not applicable to failed filters and cites a past occurrence at which a facility wa break.

RESPONSE

The EIS contains evaluations of atmospheric emissions for both intact and failed hi air (HEPA) filters. Several of the accident scenarios address situations in which assumed. These assessments provide an upper bound for the potential consequences o are discussed in Volume 2, section 5.14. Releases to the atmosphere from operating filtration systems are discussed in the Volume 2, section 5.7. The health and safe operational releases appear in Volume 2, section 5.12. The filtration systems are atmospheric emissions. Other systems, including emission monitoring and administra to ensure that filter efficiency is maintained.

To minimize airborne releases, projects involving radioactive particulates at INEL a double-confinement structure. Conservative assumptions normally are used to esti atmosphere, such as modeling only two filters in series when at least three are pla operations. Also, although HEPA filters have established particulate removal effic (down to diameters of 0.3 micrometers), a conservative efficiency factor of only 99 for operational safety and accident analyses. These filters are capable of removin 0.001 micrometers from an airstream, but the manufacturer performs the rating calib micrometers using a standard aerosol-generating device. The filters are tested ann to ensure that their efficiency is maintained.

Safety analyses for forthcoming INEL facility operations will not presume perfect H Additional precautions will be taken to minimize airborne releases. The pressure d filter is measured continuously to detect formation of any holes or insecure filter temperature will be measured to promptly detect a filter fire. Finally, radiation downstream of the filters to continuously monitor atmospheric releases. Detection particulates above the natural background levels would result in a prompt shutdown See also the response to comment 05.11.03 (009)

## II COMMENT

The commentor notes that data mentioned in the text of the EIS on off-site radiatio provided.

RESPONSE

References have been added to Volume 2, section 4.7.3 that contain the data. Spec yearly environmental reports for INEL for 1987 through 1991 (The Idaho National Eng Laboratory Site Environmental Report for Calendar Year 1991). These references are

rooms and information locations listed in the EIS.

## II COMMENT

The commentor suggests that the statistical presentation of risks is misleading.  
RESPONSE

The terms used to describe risk are not intended to suggest that individuals can have cancer. Risks applied to individuals reflect the lifetime probability of fatal cancer. Risks applied to populations reflect the number of additional cancers expected in that population. EIS are the standard used to describe the impacts being evaluated. With regard to exposure, additional basic information is provided in Volume 2, Appendix A. More detail is provided in Volume 2, Appendix F-4.

## II COMMENT

The commentor suggests that other locations or extraordinary circumstances could result in exposures and require a redefinition of the maximally exposed individual.

RESPONSE

Every reasonable effort was made to ensure that the doses estimated for the maximally exposed individual provide an upper bound for potential doses from site operations. For example, the evaluation corresponds to the highest air and ground concentrations where any member of the public resides. The dose pathways include conservatively high values for parameters such as inhalation rates and dietary intakes of locally grown produce. The scenario definition is "generic" and includes a set of standard pathways for radiation exposure. However, site-specific evaluation of these pathways is required.

The suggested approach of conducting personal field interviews to determine the potential for receiving doses in excess of the maximum individual doses is not warranted. This is not relevant to estimating foreseeable significant adverse impacts essential to reasonable alternatives.

## II COMMENT

The commentor questions the statement that less than 1 percent of the average radiation dose to a member of the United States population comes from the nuclear fuel cycle and asks for clarification.  
RESPONSE

For the average member of the U.S. population, about 82 percent of total radiation dose is from natural background, including radon (55 percent), cosmic radiation (8 percent), radon (19 percent), and natural radioactivity in the body (11 percent). About 18 percent of total dose comes from medical diagnosis and treatment (15 percent) and various consumer products (3 percent). Less than 1 percent results from the nuclear fuel cycle and global fallout. These facts are supported by the 1990 Recommendations of the International Commission on Radiological Protection.

## II COMMENT

The commentor suggests that DOE adopt an informal de minimis criterion to avoid unnecessary expenditure of resources in protecting and reassuring the public.

RESPONSE

DOE has not adopted a de minimis dose level for members of the public. Balancing dose level versus cost to further reduce the dose to the public is accomplished at DOE facilities in the context of state and Federal regulations applicable to exposure of the public to radiation. Balancing of public dose versus cost is effective in preventing the expenditure of resources on the already-low public exposures from radionuclide releases at DOE facilities. It is not the intent of this EIS to establish de minimis goals for DOE facilities.



## II COMMENT

The commentor asks if the term "health effects" in Volume 1, Appendix B, section 4 should be interpreted as "latent cancer fatalities."

### RESPONSE

There is no page 4.11-7 in Volume 1, Appendix B. DOE assumes that the commentor is on page 4.12-1 of the Draft EIS. The commentor is correct. The text has been changed to "latent cancer fatalities" instead of "health effects."

## II COMMENT

The commentor suggests that health risk-based standards be used to develop chemical hazard indices where possible.

Health risk-based standards were used to develop chemical hazard indices where possible. Where risk-based standards were not available, other methods were used. This methodology is described in Volume 2, Appendix F-4.

## II COMMENT

The commentor states that, contrary to statements in the EIS, measurable increments in emissions could result from spent fuel alternatives and suggests that the cited statement be clarified.

### RESPONSE

The statement in Volume 1 cited by the commentor has been clarified.

## II COMMENT

The commentor suggests that actual risk values be given and that the bullets in the Summary of the Draft EIS be used as a summary.

### RESPONSE

The EIS Summary is intended to summarize the information in a manner that would be understandable by nontechnical persons. The first three chapters of each volume contain more technical detail, but are still summary in form. Remaining chapters in each volume provide technical information needed to support the conclusions. The appendices are technical and provide sufficient information for a thorough technical review by specialists. The references that provide even more information on the methods and results of the technical review. The Summary has been revised as suggested.

## II COMMENT

The commentor notes that the Idaho National Engineering Laboratory has kept radiation levels low and that it is a safe area.

### RESPONSE

The comment is noted.

## II COMMENT

The commentor expresses doubt that there are no significant adverse health effects from radiation exposures typical of those received by populations surrounding commercial nuclear reactor facilities, and does not believe that the Centers for Disease Control and Prevention have adequately studied the effects of radiation exposure on human populations.

### RESPONSE

The effects of radiation exposure on human populations has been studied by many different studies.

in addition to the Centers for Disease Control and Prevention. The International Commission on Radiological Protection (ICRP) has reviewed the state of knowledge of the effects of low-level radiation exposure were adequately represented by the 1990 Recommendations of the International Commission on Radiological Protection. The ICRP concluded that the effects of low-level radiation exposure were adequately represented by the 1990 Recommendations of the International Commission on Radiological Protection. These high-dose exposures (B142, Page 142 of ICRP Publications). These high-dose exposures were used in the EIS to estimate the health impacts for radiation exposures. The health impacts to the public associated with the various alternatives would be less than occupational-accident caused fatalities. (See Volume 1, section 5.3 and Volume 2,

## II COMMENT

The commentor states that the latent cancer fatalities appear to be high (1.6 latent cancer fatalities per year, centralization at the Savannah River Site) and asks that these numbers be checked. Additionally, the commentor asks if there are ways, such as more shielding, to reduce the risk.

DOE believes that the analytical approaches and technical information used in the EIS are accurate and reliable. Every attempt was made to ensure the data are accurate. The analytical approaches used in the analyses supporting this EIS were reviewed and evaluated by independent contractors. The information in the EIS also underwent internal DOE review, and all data provided were considered in preparing the EIS.

More shielding will not be added, as designs comply with NRC regulations applicable to spent nuclear fuel materials transportation. These regulations are found in 10 CFR Part 71, which include packaging design requirements and package certification testing requirements. Comprehensive design and safety analyses and results of the required testing are submitted to NRC for use. This certification testing involves the following components: heat, physical abuse, unyielding surface, water submersion, puncture by dropping package onto a rigid surface. Some of the required tests simulate maximum reasonably foreseeable accident conditions.

## II COMMENT

The commentor raises questions regarding complete reliance on high efficiency particulate filters for preventing emissions of radioactive particulates, especially those less than 0.3 micrometers in diameter.

RESPONSE  
To minimize airborne releases, projects involving radioactive particulates at INEL use a double-confinement structure. Conservative assumptions normally are used to estimate releases to the atmosphere, such as modeling only two filters in series when at least three are planned for operations. Also, although HEPA filters have established particulate removal efficiencies (down to diameters of 0.3 micrometers), a conservative efficiency factor of only 99 percent is used for operational safety and accident analyses. These filters are capable of removing 0.001 micrometers from an airstream, but the manufacturer performs the rating calibration using a standard aerosol-generating device. The filters are tested annually to ensure that their efficiency is maintained.

Safety analyses for forthcoming INEL facility operations will not presume perfect HEPA filter performance. Additional precautions will be taken to minimize airborne releases. The pressure differential across the filter is measured continuously to detect formation of any holes or insecure filter seals. Filter temperature will be measured to promptly detect a filter fire. Finally, radiation detectors downstream of the filters to continuously monitor atmospheric releases. Detection of releases above the natural background levels would result in a prompt shutdown.

## II COMMENT

The commentor raises the issue that the most recent numbers on radiation were not used in the EIS.

RESPONSE

Volume 1, Appendix F, Figure 4.7-2 provides information on natural background radiation levels in homes (inhaled). The information referenced is from the 1987 publication by the National Council on Radiation Protection and Measurement, Ionizing Radiation Exposure to the United States. This reference provides a number that is recognized nationally. The

indicative of the natural background radiation found in the Oak Ridge area. Values areas within the country are still being studied and may differ; they may be smaller in some areas and larger in others. This information does not affect the analysis, and there have been a brief discussion of occupational and public health and safety for ORR is included in Appendix F, Part Three, section 4.12.

## II COMMENT

The commentor observes that health and safety impacts from the Idaho National Engin have apparently been minimal.

## RESPONSE

RESPONSE  
The cumulative impacts analyzed in Volume 2, Chapter 5 for all of the alternatives agree with this observation.

## II COMMENT

The commentor does not want any additional spent nuclear fuel or activities at the  
RESPONSE

The anal

RESPONSE

The analysis in Volume 1, Chapter 5 and Volume 1, Appendix F, Chapter 5 indicates that environmental consequences of the alternatives considered in the EIS would be small including ORR. Therefore, bringing in additional SNF is not likely to add to environmental hazards that may already exist at this site. See also the response to comment 01

## II COMMENT

The commentor asks whether a quantitative uncertainty analysis should be done for the  
RESPONSE

Volume 2

Volume 2, section 5.1 and Volume 1, Appendix D, section F.1.5 have been revised to of uncertainty analysis. In general, however, environmental impact analyses are de reasonable projection of the upper bound for potential environmental consequences. of appropriately conservative assumptions and analytical approaches. In this conte that an assumption or analysis would tend to overproduced, rather than underpredict However, overly conservative analyses do not provide a useful basis for comparing a the aim has been to avoid overconservatism and base the environmental impact analys levels of conservatism so that the relative impacts of alternatives can be accurate The analysis of the impacts of normal operations and hypothetical accidents are bas require input data and a model or analytical method for projecting potential impact input data for each analysis is slightly different. Socioeconomic analyses are bas for example, while air resources analyses are based on estimated releases of pollut models are also fundamentally different for similar reasons. For all analyses wher assumptions have been required, generally accepted engineering and scientific appro to ensure that these assumptions are not outside the range of uncertainty usually a Detailed uncertainty analyses can sometimes be useful to evaluate environmental imp particularly valuable when projected impacts are large and it is important to know projections are. However, quantitative estimates of uncertainty in impacts for hyp are difficult to determine. When appropriately conservative estimates of impacts a exact degree of uncertainty diminishes in importance. The estimates of impacts in enough that detailed quantitative uncertainty analyses are not necessary to meet th

## II COMMENT

The commentor suggests that professional engineers review Idaho National Engineerin facilities and questions the accountability of personnel who sign off DOE safety do

## RESPONSE

RESPONSE  
All DOE facilities are reviewed for hazard classifications per DOE Order 5481.1B, S  
Review System. Higher-hazard facilities require extensive safety analysis and revi

includes independent reviews of these analysis summarized in safety evaluation report. The safety basis of the facility are approved by the Program Senior Official at DOE. Office of Environmental Safety and Health Oversight (EH) conducts independent review documents and must agree with all assumptions, conservatisms, and analyses. This is parameters and hazard classification of the facilities personnel conducting these reviews professional engineers. See also the response to comment 06.02 (019).

## II COMMENT

The commentor is concerned that the EIS underestimates the tritium release from the an accident. The commentor estimates that the tritium release to the environment will be higher than estimated by the EIS.

### RESPONSE

Volume 1, Appendix A, section 1.1.2 has been revised to show that the amount of tritium is approximately 134 curies.

## II COMMENT

The commentor claims that past court cases have rejected shipments of nuclear waste from Sound's ports and that current government procedures do not adequately guarantee the safe transport of this fuel.

### RESPONSE

DOE complies with the DOT regulations for the transport of radioactive material. DOE is designed to protect workers and the public by minimizing the risks associated with radioactive material. The EIS analyzes a full range of alternatives, from no action to extremely limited transport of radioactive material, to centralization, which involves transport of radioactive material. For all alternatives, the potential risks from transportation include the risks associated with maximum reasonably foreseeable accidents. The potential consequences of maximum reasonably foreseeable transportation accidents are discussed in Volume 1, Appendices D and I. Although the consequences of an accident of this type and the probability of such an accident having high consequences is on the order of one chance in a million, the consequences of most accidents, including those with a probability of occurring would be less than those of the accidents analyzed.

With more than 50 years of radioactive material transportation in the commercial sector, there have been few transportation accidents involving radioactive materials, and only a little or no release of radioactivity. Nonetheless, emergency response teams are stationed throughout the United States to respond quickly in the event of a transportation accident. The importance of preparedness for potential accidents involving transportation of radioactive material is emphasized by the Federal Emergency Management Agency (FEMA) which provides training and materials to local responders to prepare them to handle accidents properly. DOE provides for Radiological Emergency Response teams, which consist of trained experts equipped and prepared to quickly respond and assist local emergency response personnel if requested. This response network, along with preventive safety measures, such as shipping container design and testing, and adherence to regulations, supports the continued safe shipping of SNF.

SNF shipping containers that could be handled by longshore workers are designed to meet international standards for safety, including radiation levels at the outside of the container. This EIS analyzes transportation from ports of entry. The potential for radiological releases from workers is within the scope of the EIS entitled Proposed Nuclear Weapons Nonproliferation Concerning Foreign Research Reactor Spent Nuclear Fuel (Draft).

As stated in this EIS, the Atomic Energy Act of 1954 authorizes DOE to establish standards to protect health and minimize dangers to life and property. Radiation protection standards are set to limit radioactive releases to as low as reasonably achievable (ALARA) levels in recognition of the health risk associated with exposure to radiation. In addition, DOE adopted and enforces safety and health protection requirements that are equivalent to those issued by the Occupational Safety and Health Administration (OSHA). DOE designs, locates, constructs, and operates facilities that provide a level of safety that is within the safety requirements for work in all comparable job categories, including high-hazard occupations such as construction. The issues discussed in Volume 1, section 5.1.1; Volume 1, Appendices A through D, Chapter 4; section 5.12. Health and Safety sections of both volumes of the EIS evaluate both radiological and nonradiological impacts to the health of workers at DOE facilities. For all alternatives,

small. The Navy complies with OSHA regulations in the nonradiological occupational occupational medicine area.

## II COMMENT

The commentor suggests that a caveat be added to Appendix F to show that exposure from a reasonable foreseeable accident is in addition to exposure from natural background.

**RESPONSE**  
Volume 1, Appendix F has been changed to reflect the commentor's suggestion.

## II 05.10.01 (001) Worker

### COMMENT

The commentor states that chemical exposure risks are not included in the analysis impacts for hazardous chemicals at the Nevada Test Site.

### RESPONSE

Chemical exposure risks associated with on-site transportation are associated only with accidents, because, during normal transportation, the chemicals are in sealed containers. Appendix F, Part Two, section 5.11.1 states that the transportation accident risk is evaluated for the chemical spill accident at the Expanded Core Facility in Volume 1.

## II 05.10.01 (002) Worker

### COMMENT

The commentor, quoting a passage from Volume 2, which states that "industrial hygiene hearing protection for all workers," asks whether Idaho National Engineering Laboratory provides hearing protection for all site employees. The commentor suggests that if they do, no effort has been made for all site workers.

### RESPONSE

INEL procedures cover all workers for all operations. DOE Orders are used to enforce mandatory compliance with Title 29 CFR 1910, Occupational Safety and Health. DOE Occupational Safety and Health Program for DOE Contractor Employees at Government-Owned Contractor-Operated Facilities, provides additional guidance for DOE contractor employees at government-owned, contractor-operated facilities and specifically requires compliance with protection requirements.

## II 05.10.01 (003) Worker

### COMMENT

The commentor suggests that workers may not be safe near leaking radioactive containers at the Hanford Site, while an effort is made to stop the source of the leak.

### RESPONSE

DOE considers worker safety in its planning before performing any work in a radioactive area. DOE policy regarding worker exposure to radioactivity is to minimize the exposure to a level that is reasonably achievable. Radiation workers are intensively trained and follow rigorous procedures to ensure safety. Also, workers have the authority to stop any work if conditions are unsafe. Work is not resumed until conditions are declared safe.

## II 05.10.01 (004) Worker

### COMMENT

Commentors raise issues about the health and safety of the workers at DOE and Navy

### RESPONSE

As stated in the EIS, the Atomic Energy Act of 1954 authorizes DOE to establish standards to protect public health and minimize dangers to life and property. Radiation protection standards for radioactive releases to as low as reasonably achievable (ALARA) levels in recognition of the health risk associated with exposure to radiation. In addition, DOE adopted and enforces safety, and health protection requirements that are equivalent to those issued by OSHA. DOE locates, constructs, and operates its facilities in a way that provides a level of protection for workers in private industry for all comparable job categories, including occupations such as construction. Analyses are discussed in Volume 1, section 5.1. Appendices A through D, Chapter 4; and Volume 2, section 5.12. Health and Safety analyses in the EIS evaluate radiological and nonradiological impacts to the health of the public. For all alternatives, impacts would be small. In the nonradiological and occupational medicine area, the Navy complies with OSHA regulations.

## II 05.10.01 (005) Worker

### COMMENT

Commentors raise the issue of potential radiation exposure to longshore workers in response to the EIS.

SNF shipping containers that could be handled by longshore workers are designed to meet international standards for safety, including radiation levels at the outside of the containers. This EIS analyses transportation from ports of entry. The potential for radiologic impacts to workers is within the scope of the EIS entitled Proposed Nuclear Weapons Nonproliferation Concerning Foreign Research Reactor Spent Nuclear Fuel.

## II 05.10.01 (006) Worker

### COMMENT

The commentor states that not all adverse properties of toxic and radioactive materials may be exposed are addressed in the EIS.

### RESPONSE

The risk of contracting fatal cancers from exposure to radiation was used as a measure of health throughout the EIS to provide a consistent document and to allow ready comparison of health impacts, such as those from exposure to chemical carcinogens. When nonfatal genetic effects from radiation are included in the analysis, the lifetime risk increase for exposure for fatal cancers to  $7.3 \times 10^{-4}$  per rem of exposure for all health effects compared to the baseline. For these health effects are provided in Volume 2, Appendix F-4.

The risk factors for cancer induction used in the EIS have been taken from the most recent Commission on Radiological Protection recommendations (1990 Recommendations of the Commission of Radiological Protection), which reflect the most recent and most wide of all currently available data. The authors reviewed all available studies. Volume 1 of the EIS provides a useful primer on radioactivity and radiation dose. Volume 2, Appendix F, discusses how radiation doses were calculated and how cancer risks were estimated. Analysis of exposure of workers to toxic materials is addressed in Volume 2, section 5.1. Inventory of potential chemical releases at INEL was reviewed and all potentially toxic materials included in the analysis, even those that are only suspected of having adverse health effects. The records of all reported occupational injuries and illnesses, regardless of cause, are included in the analysis. Potential future health impacts to workers.

## II 05.10.01 (007) Worker

### COMMENT

The commentor notes that Volume 1, section 4.12.1 does not mention anything about worker safety beyond radiation exposure and that there have been quite a number of off-site releases at the 100-K area fuel storage basins and spent nuclear fuel storage areas. The commentor suggests that occurrences for the last 5 years at the Hanford Site be summarized in the EIS.

The EIS has been changed to provide additional worker safety and health information.

## **II 05.10.01 (008) Worker**

### **COMMENT**

The commentor indicates that Idaho National Engineering Laboratory workers would no risks just to have a job.

### **RESPONSE**

DOE is formally committed to protecting the safety and health of its workers, the p environment. See the response to comment 05.10.01 (004).

## **II 05.10.01 (009) Worker**

### **COMMENT**

Commentors suggest that potential impacts to workers are deemphasized because they various sections of the document rather than in one place, and noted that the EIS d names and affiliations of those who prepared the various sections.

### **RESPONSE**

EIS preparers, their affiliations, their education, and their years of experience a and 2, Chapter 6. DOE is solely responsible for the preparation and content of the final form. Although various consultants assisted DOE in preparing this document, technical review and approval of the document.

## **II 05.10.01 (028) Worker**

### **COMMENT**

The commentor suggests that the national average value for radiation doses from rad value to use in describing the Oak Ridge Reservation area.

### **RESPONSE**

Radon doses were included as part of the description of natural background radiatio vary widely at individual locations, as well as across the nation. Results from in specific locations, change with time due to a variety of factors. Therefore, natio most useful for describing natural background from radon under most circumstances.

## **II 05.10.01 (029) Worker**

### **COMMENT**

The commentor notes that estimated radiation doses for one alternative appear to ex occupational administrative control level, and suggests a lower standard be applied

### **RESPONSE**

The purpose of the EIS is to evaluate the potential impacts from proposed activitie assumptions were made to ensure that estimated doses are conservatively high and re bound of potential impacts. Although conservatively high, the analysis shows poten the alternative in question would remain within legal limits for occupational expos intended to substitute for the assessments required by regulations or by DOE Orders constructed or operated under the chosen alternative will comply with applicable re

## **II 05.10.01 (030) Worker**

### **COMMENT**

The commentor states that the EIS does not adequately address worker fatalities fro accident conditions as a basis for comparing alternatives.

### **RESPONSE**

Volume 1, Chapter 5 discusses the disciplines studied that result in potential impa general interest, or may help discriminate among sites. The impacts from radiation from operations and accident conditions were analyzed for all alternatives containe

are summarized in Volume 1, Appendix K, Table K-2.

## II 05.10.01 (031)Worker

### COMMENT

The commentor states that contamination as a result of past nuclear weapons activities poses potential health and safety threats to many defense workers and surrounding communities.

DOE's policy is to identify and correct inadequate practices concerning safety past or present operation of its facilities. DOE, with the assistance of other agencies, has initiated many in-depth investigations into these potential health and safety concerns and taken corrective actions as soon as possible in cooperation with the respective stakeholders within existing budgetary constraints. Detailed descriptions of the events concerning releases are outside the scope of the EIS.

## II II COMMENT

The commentor states the source term inventories in Volume 1, Appendix I-20 to I-23 that no explanation was found to account for how the list was reduced. The commentor states that spent nuclear fuel typically contains a large number of fission products and their modeling purposes, the list is often truncated by combining certain parent-daughter nuclides, eliminating the minor contributors to dose.

### RESPONSE

In some cases to facilitate modeling, the radionuclide distributions for representative source terms were truncated to eliminate minor contributors to dose. The radionuclides eliminated account for a small percent of the total dose. Volume 1, Appendix I has been revised to clarify this point. The information is contained in documents referenced in Volume 1, Appendix I.

## II COMMENT

Commentors express a lack of confidence in the transportation analyses because they are in the EIS to explain how the numbers were obtained. For example, one commentor questioned the centralization at the Idaho National Engineering Laboratory requires fewer shipments from the Hanford Site, when 80 percent of DOE spent nuclear fuel is already at the Hanford Site. There are concerns regarding the verification and testing of computer codes used in the EIS. Some commentors question the transportation accident probabilities used and are concerned about the transportation accidents caused by substance abusers. Additionally, commentors question the effects of individuals in Idaho transportation corridor cities have been evaluated.

### RESPONSE

Volume 1, Appendix I summarizes the methodologies, key data, assumptions, and results for the transportation analyses. Details on the methodology, computer programs, models, and calculations are contained in supporting technical documents that are referenced in the EIS. For example, in Volume 1, Appendix I, DOE Complex Wide Spent Nuclear Fuel Shipment Estimates, DOE Programmatic Spent Nuclear Fuel Management Environmental Impact Statement, is a summary of details on fuel transportation. Therein it is noted that the Hanford fuel shipping casks, whereas most of the INEL fuel is shipped in casks holding only 25 kilograms of fuel, whereas most of the INEL fuel is shipped in casks holding only 25 kilograms of fuel, fewer shipments of fuel required to move fuel from INEL to Hanford than from Hanford to INEL. Supporting technical detail is so extensive that it could not physically accompany the EIS. Supporting technical documents are available in the reading rooms and informally identified in the EIS.

The computer codes used in the transportation analyses included the generally accepted impact assessment programs RADTRAN 4 and RISKIND, and the generally accepted transportation routing computer codes HIGHWAY and INTERLINE. These computer codes have been used by many agencies in numerous EISs, environmental assessments, and other analyses. The codes have undergone rigorous independent review and were determined to be adequate for use in the analyses. The computer codes were also chosen to be complementary in order to balance potential consequences with risks of transportation. The derivation of the transportation accident probabilities is described in Volume 1, Appendix I.



accident probabilities used in the EIS are based on historical statistics observed industries and account for many phenomena, such as weather, road conditions, and so on. The transportation analysis evaluated shipments from their point of origin to their destination. Incident-free and accident risk transportation analyses are presented for the entire Idaho, if a shipment happened to travel through, originate, or terminate in Idaho. The accident consequence analyses are presented for transportation accidents with probability of occurrence exceeding  $1E-7$  per year. The results are for various combinations of population (i.e., rural, urban, and suburban) and meteorology. Results were not given for specific locations because of the large number of towns and cities along a transportation route in which accidents occur. Instead, the results were presented for accidents in various population density zones: rural, suburban, and urban. To determine which accident corresponds to their town or city, the commenter matched their particular population density zone to a population zone analyzed in the EIS. The consequences of a transportation accident in a suburban area such as Idaho Falls would look up the consequences of an accident in a suburban area; these consequences are representative of the consequences in Idaho Falls or Pocatello.

## II COMMENT

The commenter notes that transportation impacts are underestimated and that transportation impacts have been trivialized by the comparison with traffic fatalities.

### RESPONSE

Analyses in the transportation sections of both volumes of the EIS evaluated potential impacts on the public from the transportation of radioactive material using models, data, and were chosen to overestimate the actual impacts of transportation. For all alternatives, impacts from transportation would be small.

The comparison of transportation risks with traffic fatalities is appropriate because the risk from vehicular transportation accidents is from traffic fatalities that are not associated with radioactive material or exposure to radionuclides released during a transportation accident. A comparison is needed to provide some point of reference or perspective for the risk management. There was no intention to trivialize transportation risks.

## II COMMENT

The commenter states that the transportation assessment for the waste being sent offsite was not identified and may present cumulative impacts and waste management concerns for which impacts are not analyzed in the EIS.

### RESPONSE

The comment refers to Volume 2, section 2.2.7, which discusses off-site incineration residuals to INEL as one of the existing options for treating low-level waste generation. This section does not discuss the transportation assessment for shipping waste offsite. A transportation assessment is included in Volume 2, section 5.11. Volume 2, Table 5 lists anticipated waste shipments associated with each alternative, including shipments from a private-sector facility. To bound the transportation assessment, the private-sector facility is assumed to be located in the southeastern United States, which maximizes the shipping distance. Incident-free and transportation accident analyses include the assessment of waste management and treatment. These were also included in the cumulative impact analyses.

## II COMMENT

The commenter questions the use of average annual risk for transportation impacts with a large difference in the number of yearly shipments.

### RESPONSE

The total cumulative risks from transportation for the period 1995 through 2035 are listed in Chapter 5 of the EIS. The total cumulative risk accounts for all years, including years when the number of shipments is high; however, the annual shipping rates are not expected to be large, so the average annual rate was considered. The EIS Summary has been changed to add clarifying words as agreed with EPA.

## II COMMENT

The commentor expresses an opinion that contractors at the Hanford Site are in a co situation and their assessment of contamination of the Columbia River lacks credibili  
RESPONSE

This specific issue discussed is not within the scope of this EIS; however it is th other Federal agencies to ensure that their contractors are not placed in or allowe interest situations. This EIS was thoroughly reviewed by DOE technical experts to and accurate. See also the response to comment 03.03 (008) regarding DOE credibili

## II COMMENT

Commentors express general fears about the "dangers" of nuclear power; about residu spent nuclear fuel, and/or radioactivity; and what they breathe, drink, and eat. S recent health concerns with their families or neighbors, or the effect on property should occur.

RESPONSE

DOE is aware of general public fears regarding radiation and radioactivity. The EI cumulative effect of DOE and Navy operations at the 10 candidate sites for SNF man The EIS concludes that there is no significant risk due to operations or reasonably involving SNF management, including transportation at any of the candidate sites. comment 05.15 (005) regarding property values.

## II COMMENT

The commentor states that public exposures from past releases such as the accidenta unknown.

RESPONSE

Radiation exposures resulting from past accidents, including the 1978 accidental cr assessed as cited in Idaho National Engineering Laboratory Historical Dose Evaluati cited as a reference in Volume 2, section 5.14.1.

The 1978 accident involved an unplanned nuclear chain reaction at the Idaho Chemica shielded hot cell. The incident lead to an estimated release of 620 curies, result dose of less than 0.1 millirem to the general public. There were no on-site or off

## II COMMENT

The commentor states that while sodium does not have a maximum contaminant level, i recommended level and does have an effect on humans.

RESPONSE

Although sodium levels exceed the recommended levels in isolated groundwater areas disposal has decreased in recent years. Sodium levels are shown on Table 2-4 in th Engineering Design File, available in reading rooms and information locations liste concentrations in the Snake River Plain aquifer are at or below background concentr boundary. There are no increased effects on off-site populations from sodium in gr On-site groundwater used for human consumption complies with drinking water quality established in the Safe Drinking Water Act.

## II COMMENT

The commentor does not want to receive indirect exposure from radioactive contamina chain.

RESPONSE

The EIS evaluates the potential indirect exposure from contamination in the food ch

the risks of radiation exposure to the public and to workers would be small for all based on evaluations of operations and analyses of potential facility and transport sections in the EIS that cover public safety include Volume 1, Summary, Public and Effects; Volume 1, sections 5.7.10 and 5.7.12; Volume 1, Appendices A through F, se Occupational and Public Health and Safety, and Facility and Transportation Accident Summary, Accident section; and Volume 2, sections 3.3.11, 3.3.13, and 4.11.4.

## II COMMENT

The commentor states that probabilistic risk assessments are unreliable and should radiological risks to the public or as the basis for decisions.

### RESPONSE

The accident analyses in the EIS used combinations of deterministic and probabilist Deterministic assessments are based on inductive reasoning wherein the analyst eval proposed initiating events such as equipment failures, human failures, and natural Probabilistic assessments are based on deductive reasoning wherein the analyst assu as the release of radioactive materials from a facility) and then evaluates the nec to produce the assumed result. Risk professionals and analysts consider these tech complementary. In the EIS, reasonably foreseeable accidents over a range of likeli using these techniques. The EIS concludes that risk to workers and the public woul alternatives considered.

## II COMMENT

The commentor states that public health analyses may not be adequate due to the lac and materials characterization.

### RESPONSE

Many sites are preparing separate EISs on waste management, including SRS and Hanfo waste characterization will be analyzed for impacts to public health in those EISs. Volume 1 of this EIS covers SNF management. Radiological impacts are addressed in because these impacts are of greatest significance in managing this material, and a the public.

DOE has added better references to Volume 2 to characterize waste streams and has a mapping to those references.

## II COMMENT

The commentor asks why the time period for obtaining occupational injury and illnes its contractors differs from that for private industry.

### RESPONSE

The evaluation in the EIS is based on the latest available reported data from each for obtaining occupational injury and illness rates differ because DOE and the Nati report their data at different intervals.

## II COMMENT

The commentor states that the analysis of worker doses emphasizes large accidents a address smaller events, such as unscheduled maintenance, that may give high doses t commentor asks if these are included under routine operations.

### RESPONSE

As discussed in Volume 1, Appendix F, Parts Two and Part Three, section 5.15, the a considered a range of events from comparatively frequent operational upsets to very each range of frequency, accidents with the most severe potential consequences were the accident analysis evaluates the upper bound of consequences for the smaller, mo described by the commentor. In addition, these smaller events are included in the conditions. Potential impacts to workers from operations are based on historical d

records include any doses from unscheduled maintenance and other high-dose activities in the dosimetry database. (See also Volume 1, sections 3.3.2 and 5.1.1 and Appendices A and B)

## II COMMENT

The commentor finds a paragraph on radiological health effects difficult to follow  
RESPONSE

Volume 1, Appendix F, Part Two, section 5.12 has been reworded to clarify its meaning.

## II COMMENT

Commentors raise questions about or state that the EIS did not adequately discuss the impacts on the public and environment as a result of operating facilities.

RESPONSE

Volume 1, Chapter 5 and Volume 2, Chapter 5 discuss radiological and nonradiological impacts on the public relating to SNF management activities and environmental restoration and waste management activities at INEL. For all alternatives considered in this EIS, impacts would be limited to safety impacts to the public from the rest of DOE's operations are beyond the scope of the EIS.

## II COMMENT

Commentors state that radiological health impacts other than fatal cancer, total deaths, and genetic effects are not addressed in this EIS.

RESPONSE

Risk of fatal cancers from exposure to radiation was used as a measure of impact throughout the EIS to provide a consistent document and to allow ready comparison with impacts, such as those from exposure to chemical carcinogens. Nonfatal health effects from radiation are a legitimate concern and are included in the EIS. Volume 1, section 5.1 is changed to clarify fatal and nonfatal cancers and genetic effects.

The EIS analyses of the potential effects of radiation exposure do consider health effects, fatalities and are based on the standards of the International Commission on Radiological Protection. The term "health detriments" includes the total impact of all fatal cancers, nonfatal cancers, and genetic effects. The health detriments caused by any exposure to radiation are calculated by taking into account the effects after multiplying each effect by a weighting factor intended to represent the relative contribution of each type of effect has on human health.

Volume 1, section 5.1 discusses the terminology and risk factors used by the International Commission on Radiological Protection, which are consistent with those used by NRC. These factors are used in calculating the effects on human health. Cancer fatalities were used to sum up the results in the EIS, because this effect was viewed to be of the greatest interest to the public. It states that the number of total health effects (deaths, nonfatal cancers, genetic effects on human health) may be obtained by multiplying the factor of 1.46 times the latent cancer risk.

## II COMMENT

The commentor questions the safety of spent nuclear fuel when in a shipping cask, and the potential radiation exposure of 10 millirem per hour at 1 meter from the surface of the cask.  
RESPONSE

The comparison of the 10 millirem radiation dose with a chest x-ray was intended to show that the projected doses would be small. DOE did not intend to imply that there would be no health effects associated with exposure to a shipping cask. In fact, no members of the public are exposed to a radiation dose of as much as 10 millirem because they would be at greater distances from the cask and exposed for much shorter periods of time.

## II COMMENT

The commentor questions the presentation of radiation dose and risk impact in Volume Table 3-1 as an example and states that as radiation exposure doubles, the chance of increases by approximately a factor of 10.

**RESPONSE**

The comment is inaccurate. In Volume 1, Appendix D, Table 3-1, units are the lifetime over the entire 40 years for the alternatives listed in the table. The numbers are hour.

## II COMMENT

Commentors suggest that estimated releases from proposed facilities are too near the dose limit established under the National Emission Standard for Hazardous Air Pollutants should be implemented to reduce the dose to as low as reasonably achievable.

**RESPONSE**

The purpose of the EIS is to evaluate the potential environmental impacts from proposed facilities. For this reason, assumptions were made to ensure that estimated doses are conservatively an upper bound of potential impacts. The EIS is not intended to substitute for the National Emission Standard for Hazardous Air Pollutants or any other regulatory requirements, including assessments of radiation doses under the National Emission Standard for Hazardous Air Pollutants.

## II COMMENT

The commentor expresses an opinion that DOE is not fully committed to protecting public safety.

**RESPONSE**

The Secretary of Energy has publicly affirmed that DOE policy and practice now place environmental considerations above other program goals. DOE is working to expediently rectify and eliminate adverse environmental impacts as a result of previous practices committed to protecting the safety and health of its workers and the public, and to the environment. DOE intends to design, construct, and operate all proposed facilities at a level of safety and of safety assurance that complies with applicable Federal, state, and DOE Orders.

## II COMMENT

The commentor questions whether the environmental, safety, and health effects of the radioactive releases from the K-basins have been adequately considered.

**RESPONSE**

The health effects for members of the public from radioactive releases are described in Appendix A, section 4.12.2. This section describes the environmental monitoring and consequences to the public from the Hanford Site. Volume 1, Appendix A, section 5.1 describes releases and dose consequences to the public from current activities at specific facilities.

## II COMMENT

The commentor questions whether public health impacts are underestimated in the EIS.

**RESPONSE**  
DOE believes that conservative analyses have been used to estimate public health impacts. Discussion of this matter has been added to the EIS. The environmental impact analysis produces a reasonable projection of the upper bound for potential environmental consequences. requires the use of appropriately conservative assumptions and analytical approaches. "conservative" means that an assumption or analysis would tend to overproduce, rather than underproduce, impacts.

any adverse impacts. However, overly conservative analyses do not provide a useful alternatives. Therefore, the aim has been to avoid over conservatism and base the analyses on realistic, site-specific information wherever possible. Each alternative uses similar methods and levels of conservatism so that the relative impacts of alternatives are assessed.

The analysis of the impacts of operations and hypothetical accidents are based on two elements: input data and a model or analytical method for projecting potential elements necessarily introduce some uncertainty in the estimated level of impacts or nature of the input data for each analysis is slightly different. Socioeconomic analyses are based on projected budgets, for example, while air resources analyses are based on estimated budgets. The analytical models are also fundamentally different for similar reasons. Therefore, uncertainty varies among the analyses in the EIS. However, for all analyses where assumptions have been required, generally accepted engineering and scientific approaches to ensure that these assumptions are not outside the range of uncertainty usually a Detailed uncertainty analyses can sometimes be used to evaluate environmental impacts particularly valuable when projected impacts are large and it is important to know the range of potential impacts. However, quantitative estimates of uncertainty in impacts for hypothetical accidents are difficult to determine. When appropriately conservative estimates of impacts are available, the degree of uncertainty diminishes in importance. The estimated impacts in the EIS that detailed quantitative uncertainty analyses are not necessary to provide a mean potential consequences.

## II COMMENT

The commentor notes that EIS doses reported in rem are not defined as either "committed effective dose equivalent" or "total effective dose equivalent."

### RESPONSE

For readability, the generic term "dose" is used throughout the EIS in place of the terms "committed effective dose equivalent" (CEDE) or "total effective dose equivalent" (TEDE); that is, the doses reported in the EIS are TEDE; that is, the reported dose accounts for external radiation sources as well as the 50-year CEDE from internal sources. For the accident analyses in the EIS, the TEDE is generally dominated by inhalation and ingestion pathways. On the other hand, occupational doses from operations are almost entirely EDE. In either case, it is appropriate to identify and provide that doses from both external and internal pathways are accounted for.

## II COMMENT

The commentor states that Volume 2, section 4.7.3 overestimates the significance of radiation when compared with other exposures and that exposures that are a small fraction of background radiation are not necessarily "acceptable" because the public is usually unaware of fluctuations in exposure to background radiation.

### RESPONSE

Volume 2, section 4.7.3 presents a comparison of doses from INEL activities to background doses and attempts to call these doses acceptable.

## II COMMENT

The commentor asks if multiple sclerosis was included in the health effects studied by the National Engineering Laboratory or anywhere else.

### RESPONSE

Multiple sclerosis was not one of the health effects studied for INEL or any of the health effects considered were the ones generally associated with exposures to radiation or nuclear activities. The clearest indications of the effects of DOE activities discussed in the EIS are the effects of radiation exposure have not indicated any association between radiation exposure and multiple sclerosis. Multiple sclerosis has been studied by medical researchers. For more information, contact the Multiple Sclerosis Society at 800-624-8236.

## II COMMENT

The commentor suggests that, with regard to incident-free transportation calculations, be an oversimplification in either the radiological or the nonradiological models observed in the range of results presented.

### RESPONSE

DOE has reviewed the models used for incident-free transportation calculations for nonradiological fatalities and has not identified any over-simplifications. The basic conclusion is apparently a comparison of the range between truck fatalities and rail general population presented in Tables I-15 to I-19 of Appendix I. Radiological fatalities include both fatalities for the general population and for workers.

## II COMMENT

The commentor objects to the characterization of a 34-percent increase in cancer risk.

The term "minimal" relates to the overall risk from operations of SNF facilities at 1 year of operations would be  $2.9 \times 10^{-2}$ . In other words, a 34-percent increase in still a very small number.

## II 5.11 Accidents/Releases

## II COMMENT

The commentor is concerned about the effects from even small accidents.

### RESPONSE

Volume 1, Chapters 3 and 5 and Appendices A through F; and Volume 2, Chapters 3 and F discuss risks to the public, workers and the environment due to a range of large discussions include extensive evaluations and analyses of accidents. Small accidents the analysis, particularly if they have a high probability of occurring. The EIS shows workers and the public from all accidents would be small for all of the alternative

## II COMMENT

The commentor states that, although there are no known disasters in handling of the exists, no one can say that a disaster will not be created.

### RESPONSE

Volume 1, Chapters 3 and 5 and Appendices A through F, and Volume 2, Chapters 3 and F, discuss risks to the public, workers, and the environment due to facility and transportation including SNF- handling accidents. The EIS analyses also evaluate the potential consequences of accidents. These analyses have been extensively reviewed. The EIS shows that the public from such accidents would be small for all alternatives considered.

## II COMMENT

The commentor questions the rationale of including analysis of a spent nuclear fuel involving a release of large amounts of radioactive materials, as the historic transportation accident shows no such releases.

### RESPONSE

DOE agrees with the commentor's assessment of the historical safety record for SNF activities. Consequently, DOE assigned a probability of  $1 \times 10^{-7}$  (one in one million) for SNF transportation accidents accompanied by a large release of radioactivity.

## II COMMENT

The commentor suggested that a rural population would represent a "best case scenario" in the event of a release from containment at the Oak Ridge site.

### RESPONSE

This comment concerns the description of the existing socioeconomic conditions provided in Chapter 4. These generalized population distributions were not used in accident assessment as discussed in Volume 1, Appendix F, Part Three, section 5.1 distributions in the most populous sector were used to maximize potential radiation population.

## II COMMENT

The commentor states that DOE should more fully study the potential effect of mass storage tanks at the Idaho National Engineering Laboratory regarding impacts on all downwind, and on the site.

### RESPONSE

The evaluation of facility accidents in the EIS considered a range of large to small maximum reasonably foreseeable accidents. Reasonably foreseeable accidents as defined in Recommendations for the Preparation of Environmental Assessments and Environmental Statements include those for which impacts may have very large or catastrophic consequences. Chapters 3 and 5 and Appendix F discuss risks to the public, workers, and the environment for all of the alternatives considered.

The maximum reasonably foreseeable accident considered in the EIS with a potential River Plain aquifer was the immediate release of 300,000 gallons of radioactive liquid waste tank at the Idaho Chemical Processing Plant. The assessment, discussed in Volume 1, shows that the impacts to the aquifer would be small; for example, drinking water supply was not exceeded at the site boundary. No adverse impacts to other life forms would be expected from an accident.

Also discussed in Volume 2, section 5.14 is the maximum reasonably foreseeable accident result in an airborne release of radioactive or hazardous material at INEL. This evaluation of an earthquake at the Argonne National Laboratory-West Hot Fuel Examination Facility. Table 5.14-4, should such an incident occur, a potential exists for limited adverse impacts on wildlife onsite or downwind of the facility. No impacts would be expected to endanger species for this or any other reasonably foreseeable accident.

## II COMMENT

Commentors state that there are significant safety problems at the Idaho National Engineering Laboratory including historical accidents, and operational incidents.

### RESPONSE

DOE's accident history at INEL has been compared with other industries, as summarized in section 5.14.1. This comparison shows that the accident rate at INEL is lower than private industrial work. Past accidents were analyzed in Idaho National Engineering Laboratory Historical Dose Evaluation, and reasonably foreseeable accidents were analyzed in Appendix F for Idaho National Engineering Laboratory Facilities. Protection of members of the public against accidents is considered by DOE in the design, location, construction, and operation of facilities. The EIS shows that the risk to workers and the public from facility accidents is low for all of the alternatives considered.

## II COMMENT

The commentor states that the work-day population of the Idaho Chemical Processing Plant is 1,000, and that DOE does not explain why a lower number of workers was used in the potential collapse of the main stack caused by an earthquake.

### RESPONSE



A seismic event large enough to cause a stack collapse would clearly initiate an emergency response. Workers would either take cover or evacuate as directed by the emergency response. A qualitative assessment of the number of workers either within the range of the stack normal evacuation path might be impeded by debris from the stack collapse indicated workers could be affected.

## II COMMENT

The commentor states that the more material that exists at a particular location, the more likely an accident will occur.

### RESPONSE

DOE agrees with the comment. The likelihood of accidents as assessed in the EIS depends on the handling rate and the amount of waste. Both of these considerations were included in the analyses discussed in Volume 1, site-specific Appendices A through F, and Volume 2,

## II COMMENT

The commentor states that the analysis associated with a radiological release following the Snake River Plain aquifer.

### RESPONSE

In terms of the consequences to the Snake River Plain aquifer, the maximum reasonable accident analyzed with a potential impact was a release of the entire contents of a tank at the Idaho Chemical Processing Plant. This potential accident is discussed in Volume 1, Appendix F-5. The analysis assumed a seismic event of sufficient magnitude to cause a tank failure, and 300,000 gallons of high-level waste to be released to the soil beneath the tank. Migration of contaminants into the aquifer showed that even without any mitigation the maximum concentration of radionuclides at the nearest site boundary was within required drinking water standards.

The analyses of accidents described in Volume 1 and Volume 2 of this EIS include analyses of accidents that might release radioactive material to the Snake River Plain aquifer or to the environment. These analyses are described in Volume 1, Appendices B and D, and in Volume 2, section 5. These analyses show the risks to the public and workers would be small for all of the accidents considered.

## II COMMENT

The commentor expresses the opinion that the fuel handling control systems at the Idaho Chemical Processing Plant are inadequate, and suggests the likelihood of a criticality may be higher than in the EIS, particularly as the Idaho National Engineering Laboratory consolidates, and stores more spent fuel. The commentor states that a criticality accident at ICPP-666 would have a frequency closer to  $1\text{E-}01$  per year rather than  $1\text{E-}03$  per year. Thus, the commentor's evaluation of an inadvertent nuclear criticality in ICPP-666 is needed to complete the EIS.

### RESPONSE

DOE established an estimated annual frequency for a criticality accident during SNF handling in a water pool by consensus of a group of experts. To the knowledge of these experts, there has never been a criticality accident anywhere in the world during storage of SNF in a water pool. The experts' consensus was that a frequency of  $1\text{E-}4$  events per year was a representative value for accidental criticality in a water pool throughout all DOE SNF handling and storage. DOE's consensus that controls in effect at a specific facility and the condition of the facility may justify the use of a larger or smaller value, but that overall the probability is in the range of  $1\text{E-}03$  to  $1\text{E-}05$  events per year. Detailed review of the EIS would reveal the reasons for the frequency of this accident in specific facilities.

Based on this consensus, the estimated annual frequency for a criticality accident is  $1\text{E-}03$  per year in Volume 1, Appendix B. The higher frequency of occurrence was based on the storage arrangement, and the type, age, and condition of fuel in ICPP-603. ICPP-603 and storage arrangements for fuel in ICPP-666 are better than for fuel in ICPP-603. DOE expected that the frequency of occurrence of an accidental criticality in ICPP-666 would be lower than in ICPP-603. Accordingly, a starting estimate of  $1\text{E-}04$  per year is more appropriate.

ICPP-666 has a larger fuel inventory than ICPP-603. Methodology was established an EIS to adjust the frequency of occurrence for fuel inventories and for the number of operations. It was determined that a fuel inventory difference does not directly affect the occurrence of an inadvertent criticality, but only indirectly through an effect on the future

as it was in the past. Accordingly, it is appropriate to use  $1E-04$  per year as the estimated frequency of occurrence of a criticality accident in ICPP-666.

The commentor also implies that receipt of more reactive Navy fuel would cause the criticality accident to increase. Because fuel is more reactive does not necessarily increase the occurrence of an inadvertent criticality. ICPP imposes additional administrative controls on reactive fuel (e.g., when such fuel is being handled, only one module is allowed to be open at a time). Thus, the frequency of occurrence of an inadvertent criticality for handling in ICPP-666 remains on the order of  $1E-04$  per year.

The commentor states that 1) ICPP has not performed a detailed assessment of nuclear fuel and ICPP-666 fuel-handling operations; 2) ICPP has not conducted comprehensive accident analyses of planned operations; and 3) ICPP has not developed and implemented a fuel control system. The commentor is incorrect. All of these actions were completed in ICPP-666.

The commentor further alleges that if SNF is consolidated at the Idaho National Engineering Laboratory "there will be a much higher probability that an accidental nuclear criticality will occur by the EIS." The results in the EIS for ICPP-603 represent the bounding inadvertent frequency of this event does not change for various alternatives, because movement of fuel would take place under all alternatives. If other fuels are consolidated at ICPP, used for storing that fuel. The frequency of occurrence of an inadvertent criticality somewhat in another facility, either existing or yet to be built, for storage of the fuel. For example, the frequency of an inadvertent criticality in ICPP-666 may increase from  $1E-04$  per year if all the consolidated fuel were handled there. Nevertheless, the bounding event frequency for all alternatives is expected to be an event in ICPP-603 as stated in the EIS.

## II COMMENT

The commentor states that the location selected for the potential spent nuclear fuel storage at the Oak Ridge Reservation will be next to the Y-12 "walk-in pits," which contain highly pyrophoric chemicals.

### RESPONSE

The Y-12 pits are actually 4 miles from the West Bear Creek Valley site selected for management activities at ORR. The distance is accounted for in accident impacts analysis in the EIS, and no significant adverse environmental or health and safety impacts are foreseen as a result of the proximity of the Y-12 pits.

## II COMMENT

The commentor asks for a description of the cask drop accident mentioned in Volume 1 of the EIS.

RESPONSE  
The cask drop accident mentioned is a postulated scenario in which a cask holding spent fuel rods overturned in the fuel transfer area of the 105-KE or 105-KW basins at the Hanford site. Broken spent fuel rods might spill out of the cask and onto the floor of the building. This accident is described in detail in Volume 1, Appendix A, section 5.1 of the EIS has been changed to correctly reference the cask drop accident.

## II COMMENT

The commentor recommends clarifying how the estimated frequency of a fuel-handling accident at Idaho National Engineering Laboratory, and the impacts associated with it, would change for the alternatives.

### RESPONSE

The characteristics of accidents analyzed under each of the alternatives are adjusted to reflect the differences in fuel handling between the alternatives.

scaling factors developed for both frequency and consequences (see Accident Assessment National Engineering Laboratory Facilities). For example, the expected frequency of accident involving SNF would be greater in the 1992/1993 Planning Basis alternative because of the increased number of handling events in the 1992/1993 Plan compared with the No Action alternative. But no adjustments to the consequences were made because the same type and amount of "material at risk" would be involved.

## II COMMENT

The commentor states that the accident impacts would decrease for Oak Ridge under the alternative due to storage upgrades not included in the No Action alternative.

### RESPONSE

Volume 1, section 5.1 has been modified as identified by the commentor.

## II COMMENT

The commentor notes that no liquid releases are planned for normal operations and to address whether these plans are subject to change; and if so, analyses should be made.

RESPONSE  
No current plans exist to change the operating scenario (i.e., no liquid releases in the environment, as stated in Volume 1, Appendix F, Part Two, section 5.8.1). Nevertheless, the release scenario was evaluated for this EIS, which represents a maximum amount of 1 could be released under operating conditions. This evaluation should be sufficient for operations releases.

## II COMMENT

Commentors indicate the EIS failed to analyze transportation accidents while transporting fuel through inland waters of the United States.

### RESPONSE

Volume 1, Appendix I has been expanded to include three additional shipping scenarios: N-Reactor SNF from the Hanford Site to Sellafield, England, for processing. The scenario and U.S. territorial water barge transport of SNF and transoceanic shipment of SNF. Accident consequences are included for port activities as well as during ocean transport. The public from these activities has been shown to be very small. This evaluation is an example of reasonably foreseeable impacts. Analyses, impacts, and consequences of research reactor (FRR) SNF on the open seas to the United States is addressed in the Proposed Nuclear Weapons Nonproliferation Policy Concerning Foreign Research Reactor Nuclear Fuel.

## II COMMENT

Commentors suggest that the EIS describe the historical spent nuclear fuel accident in 1971 and 1993 to determine if any had occurred in urban or suburban areas where the accident was noted by the EIS to be very low (less than  $1 \times 10^{-7}$  per year).

### RESPONSE

The  $1 \times 10^{-7}$  per year probability cited by the commentors does not refer solely to SNF accident; rather, it refers to the probability of an SNF accident accompanied by radioactivity. Based on the historical record, no SNF accidents in any areas (rural or urban) have resulted in the release of large amounts of radioactivity.

## II COMMENT

The commentor notes that the EIS does not address the potential for shipboard fires

contamination as a result of those fires, or the impact to emergency response personnel should a shipboard fire occur.

#### RESPONSE

The analysis of accidents, including shipboard fires, in ports and on ships, and the emergency response personnel for FRR SNF is beyond the scope of this EIS. However, accidents and their impacts are being addressed in a separate EIS entitled Proposed Nonproliferation Policy Concerning Foreign Research Reactor Spent Nuclear Fuel (Draft) as well as a decision as to whether the United States will receive such SNF.

The criteria used to choose the ports of entry are outlined in the Notice of Intent Register Vol. 58, No. 202, October 21, 1993, pages 54336-54340). These criteria include (a) harbor and dock characteristics to satisfy the cask-carrying ship requirements, and secure lag storage, (c) adequacy of overland transportation systems from ports of entry, (d) experience in safe and secure handling of hazardous cargo; (e) emergency preparedness and nearby communities; and (f) proximity to the proposed storage sites. A range of other factors will also be analyzed in the FRR EIS. The decision regarding port selection will not be completed until the FRR EIS is completed.

An analysis of a shipboard fire involving Naval SNF is included in Volume 1, Appendix Attachment F.

## II COMMENT

The commentor requests inclusion of a shipboard fire accident scenario in the EIS.

#### RESPONSE

Shipboard transport and handling of SNF is beyond the scope of this EIS. Policy alternatives for States origin foreign research reactor SNF, and for its transport, receipt, handling and storage are addressed in a separate environmental impact statement (58 FR 54336). The FRR SNF impacts of marine transport and receipt of FRR SNF at six or more ports of entry, and potential accidents, including a shipboard fire, will be evaluated.

An analysis of a shipboard fire involving Naval SNF is included in Volume 1, Appendix Attachment F.

## II COMMENT

Commentors raise the issue that transportation-accident health impacts to Tribal members living along Interstate-15 through the Shoshone-Bannock Reservation are not included in the EIS.

#### RESPONSE

As discussed in Volume I, section 5.11.2, radiological impacts for incident-free transport are determined for (1) crewmen (drivers) and (2) members of the public. The crewmen category includes drivers of the shipments, and the members of the public category includes Tribal members. For incident-free transportation, the radiological effects a shipment inspector might experience are encompassed within the effects to a crewman or driver of shipments based on the time the inspector interacts with a shipment compared to the interaction time of the driver. The effects to the driver are based on the driver receiving radiological exposure, while in the cab of the vehicle and during detailed inspections of the cargo and the radioactive material.

Incident-free radiological impacts to Tribal members for SNF and radioactive waste storage at the reservation are encompassed in the existing EIS analyses for members of the public living along a generic transport route.

A reservation-specific accident analysis would not provide information additional to that provided in Volume 1, Appendices D and I for the programmatic alternatives. The probability of an accident occurring along a specific 20-mile segment of interstate highway is so small that it is beyond the range of analysis required for a programmatic EIS.

## II II COMMENT

The commentor states that previous releases and accidents at DOE sites were intentional. The commentor also discusses previous and potential releases of radioactivity from government sites.

**RESPONSE**

It is DOE policy to identify and correct any inadequate practices concerning safety operation of its facilities. In this regard, accidents and accidental releases are releases from DOE facilities under all operating conditions are included in annual Detailed accounts of the events related to prior accidents or releases are outside EIS addresses the impacts of a number of reasonably foreseeable accidents related to with no significant risk of health effects or environmental impacts identified. DO current, and reasonably foreseeable future activities in assessing the cumulative impact small.

The environmental impact analyses are designed to produce a reasonable projection of potential environmental consequences. This requires the use of appropriately conservative and analytical approaches. In this context, "conservative" means that an assumption to overpredict, rather than underpredict, any adverse impacts. However, overly conservative not provide a useful basis for comparison among alternatives.

**II COMMENT**

Commentors, when referring to the transportation discipline, state they are confused by the phrase "maximum reasonably foreseeable accident." For example, commentors state they wonder if this worst-case accident and whether the EIS has evaluated such an accident. Commentors state that the maximum reasonably foreseeable accident, and commentors state they would deal with such an accident if it occurred.

**RESPONSE**

The EIS evaluates two complementary aspects of the impacts from transportation accidents: the risk associated with transporting radioactive material; and the probabilities and consequences of a complete spectrum of transportation accidents, from accidents with high probabilities and low consequences, to accidents with low probabilities and high consequences. The second aspect is the consequence associated with a bad transportation accident. This is too subjective and statistically, has virtually no probability of occurring. In an accident that better represents an accident that could occur, but one which has not occurred. This kind of accident is termed the "maximum reasonably foreseeable accident." In guidelines for accident analyses in EISs, this accident was chosen based on having a risk of  $10^{-7}$  per year or about one in 10 million per year. This kind of accident is roughly used to be called a worst-case accident, except that it is chosen based on a specific risk of  $10^{-7}$ .

For most alternatives, an accident involving a rail shipping container containing spent nuclear fuel is a maximum reasonably foreseeable accident. The precise accident scenario that leads to the maximum reasonably foreseeable accident is not described because there are different combinations of factors that could lead to the accident conditions. For example, a high-speed train collision with a tank car followed by a high-temperature fire that lasts 2 to 3 hours could lead to these conditions. Other combinations of fire and impact that could lead to the same conditions. Appropriate mitigation measures for various combinations.

The mitigation of transportation accidents may come either before or after the accident. Mitigation measures used before the accident include shipping the radioactive material in approved containers that contain large amounts of radioactive material, such as SNF, only containers that are designed to withstand hypothetical accident conditions are used. In addition, transportation measures are chosen to minimize the risk associated with transporting radioactive material. Measures after a transportation accident include emergency response and EPA protective action guidelines to limit doses.

The EIS Summary was changed to clarify this concept.

**II COMMENT**

The commentor asks about the impacts to the Idaho agricultural industry resulting from releases of hazardous materials to the air or to groundwater.

**RESPONSE**

Volume 1, Chapters 3 and 5 and Appendices A through F, and Volume 2, Chapters 3 and 5, discuss risks to the public, workers, and the environment due to facility accidents. Impacts from accidents would be small for all of the alternatives considered.

The maximum reasonably foreseeable accident considered with a potential impact to the environment.

aquifer was a release of the entire contents of a high-level waste tank at the ICPP 2, section 5.14. The assessment shows that even without taking credit for mitigation the aquifer would be small; for example, drinking water standards would not be exceeded. As shown in Volume 2, Table 5.14-4, for any accident involving an airborne radioactive or hazardous material at INEL, there is a potential for limited economic 1-year restrictions to public lands or up to a 1-year agricultural land withdrawal immediately adjacent to INEL (up to an estimated 10,000 acres).

## II COMMENT

The commentor notes that it is inconsistent to say no cases were found where an accident could cause an accident in a collocated facility when an earthquake could cause multiple facility and across the entire site.

### RESPONSE

Qualitative assessments of accidents associated with existing and proposed operations for causing accidents in another facility were part of the accident evaluation. No accident in one facility would cause an accident in another facility great accidents already considered in the EIS. The potential for simultaneous accidents seismic initiator is described in Volume 2, section 5.14. DOE's analysis shows that would be bounded by those resulting from the postulated accidents at the Argonne National Laboratory. Volume 1, Appendix B, the consequences and risks associated with multiple facility accidents eliminated from further consideration because they do not represent the maximum realistic accidents within the frequency categories defined in Volume 1, Appendix B, Table 5.

## II COMMENT

Commentors state that the effects of a large earthquake at the Nevada Test Site show high consequence, low probability event.

### RESPONSE

In the EIS, the accident yielding the largest radiation dose (i.e., the bounding event) into the dry cell facility scenario. This accident scenario assumes a breach of the subsequent airplane fuel fire resulting in a plume of contaminants. The results of the analysis are provided in Volume 1, Appendix F, Part Two, Tables 5.15.1 through 5.15.6.

A large-earthquake scenario was considered in the EIS. It was determined that the scenario differs from the airplane crash scenario in that there is limited combustible material spilled airplane fuel is not present during an earthquake, and ignition sources are limited. Impact of subsequent fires and resultant contaminant plumes was found to be less in the earthquake scenario than for the airplane-crash scenario. As a result, a more detailed analysis was not warranted.

## II COMMENT

The commentor expresses disbelief that impacts from accidents such as Three Mile Island would not cause damage if they occurred at the Idaho National Engineering Laboratory.

### RESPONSE

The nature of potential accidents associated with storing SNF, as well as treating wastes, at INEL differs from the types of accidents the commentor mentions. Nuclear accidents cited were so intensely radioactive that the heat they generated internally or burned the fuels in the absence of cooling. For SNF in long-term storage at INEL, radioactivity has occurred long enough that the heat the fuel generates would be much required for fuel melting. The fraction of radionuclides available to be released is smaller for nonmelted fuel than for reactor fuel that could melt by internally generated heat. This EIS shows that the risk to workers and the public from INEL facility accidents is less than of the alternatives considered.

## II COMMENT

The commentor notes that flooding could occur at the Idaho National Engineering Lab impacts to water resources should be addressed.

#### RESPONSE

The INEL accident analyses, summarized in Volume 1, Appendices B and D, and Volume 2, considers flooding and other natural phenomena as potential causes of accidents. Scenarios were selected for detailed analysis because they were comparatively likely, and scenarios for detailed analysis because of their large potential consequences. The consequences of the high-level waste tanks was selected for detailed analysis instead of flooding inventory in the high-level waste tanks has a larger potential for consequences to flood. The high-level waste tank failure accident is reported in Volume 2, section 2.1.1. The aquifer would be small under all the alternatives that were analyzed.

## II COMMENT

The commentor states that risks associated with Idaho National Engineering Laboratory storage, waste management, and reburial of wastes for the Pit 9 Retrieval project have been characterized in the EIS. The commentor further asks that if the Pit 9 waste is not in the ground, what is the case with the safety of the tons of high-level waste in storage?

#### RESPONSE

The Pit 9 Retrieval Project is an on-going project initiated under INEL FFA/CO and approved alternatives. Simply stated, the project will excavate previously buried wastes, separate components, and rebury the remaining waste. The separated components would be placed and stored in the Transuranic Storage Area of the RWMC. While the Project has separate documentation, the Pit 9 Retrieval Project impacts were included in this EIS as part of a summary of Pit 9 Retrieval Project is given in Volume 2, Appendix C. Risks, including those associated with the Pit 9 Retrieval Project are part of the baseline impacts summary in Chapter 5. Post-treatment low-level waste from Pit 9 could be stored safely above ground, returned to shallow land burial. The section in the EIS Summary entitled Public Works notes that the risk from facility accidents would be small for the alternatives considered.

## II COMMENT

The commentor states that collocation issues are not discussed, and that there is a potential for secondary impacts from an accident in one facility on other operating facilities at the Engineering Laboratory.

#### RESPONSE

Volume 2, Chapters 3 and 5 and Appendix F discuss risks to the public, workers, and the environment from facility accidents at INEL. As indicated in the EIS, the discussion is a summary of detailed Accident Assessments for Idaho National Engineering Laboratory Facilities. The assessment includes evaluations and analyses of accidents that were extensively reviewed. Qualitative assessments associated with existing and proposed operations, and their potential for secondary impacts in another facility, were part of the accident evaluation. No case was made that an accident in one facility would cause an accident in another facility greater than that already considered in the EIS. Secondary impacts to other facilities were limited to increased costs. No other collocation issues were identified.

## II COMMENT

Commentors suggest that particles released from the main stack at the Idaho Chemical Processing Plant on April 2, 1992, could be dispersed by wind and that a single 3-millirem release could cause an exposure of 10 millirem in about 3 1/2 hours. Commentors suggest that the release of such particles was not analyzed because of the assumption of interdiction measures.

#### RESPONSE

In the incident at the ICPP main stack, a release of quarter-sized flakes of ammonium nitrate at an elevation of about 250 feet. All detectable material was found within an area 2,560 feet in diameter.

yards long, about 12 acres. Thus, it is unlikely that any detectable radioactivity the INEL boundary. A subsequent cleanup effort with high efficiency particulate air equipment returned the contaminated area to levels below those for noncontaminated with DOE Order 5480.11, Radiation Protection for Occupational Workers. Resuspension of radioactive materials from the ground by wind is acknowledged as a mechanism. Windborne resuspension reduces the amount of exposure at any given distance of releases, but increases the area in which some exposure occurs. The commentor is that direct contact with a 3-millirem-per-hour particle for about 3 1/2 hours would whole body dose of 10 millirem. Rather, only that part of the body in contact with receive a localized dose of 10 millirem. Depending on the exposure pathway, it may such particles to result in an effective whole body dose of 10 millirem. For the I with the maximum reasonably foreseeable consequences, and with the most unfavorable conditions, some restrictions on use of agricultural products might be implemented established protective action guides.

## II COMMENT

Commentors raise the issue of health risks involved should there be an accidental water table at the Idaho National Engineering Laboratory.

### RESPONSE

Volume 1, Chapters 3 and 5 and site-specific Appendices A through F; and Volume 2, and Appendix F discuss risks to the public, workers, and the environment due to a facility accidents. The maximum reasonably foreseeable accident considered with a Snake River aquifer was the release of the entire contents of a high-level waste to Processing Plant. This accident is discussed in Volume 2, section 5.14. The assessment without taking credit for mitigation measures, impacts to the aquifer would be small at the site boundary would be within requirements of the safe drinking water standards.

## II COMMENT

The commentor states that the EIS fails to fully assess the Idaho Chemical Processing waste tanks and vaults, including structural constituents, seismic (risks), leakage and service line leaks.

### RESPONSE

A maximum reasonably foreseeable accident associated with the high-level waste tank the EIS, as reported in Volume 2, section 5.14. A more detailed description of the Accident Assessments for Idaho National Engineering Laboratory Facilities. The analysis of a seismic event of sufficient magnitude to cause one or more tanks to fail, and 300,000 waste to be released to the soils beneath the tank farm. Modeling of migration of Snake River Plain aquifer showed that even without any mitigation measures, the maximum of radionuclides at the nearest site boundary would be within requirements of safe standards.

## II COMMENT

Commentors express disbelief that a criticality would occur only once in 10,000 years fuel storage pool; risk methods used to estimate number of latent cancers are also not believable to commentors.

### RESPONSE

DOE acknowledges a typographical error in Volume 1, Chapter 5. The estimated probability of a criticality accident at the ICPP is 1 chance in 1,000 per year of operation, not 1 in 10,000. While DOE recognizes the potential for a criticality accident in an SNF storage pool, a nuclear criticality in an SNF storage pool in the history of the DOE complex or in the experience base represented by the commercial nuclear power industry. The evaluations in this EIS of the probability of an inadvertent criticality consider including facility design controls, administrative controls, fuel inventories, fuel of some fuels, and fuel-handling frequencies. In addition to the estimated probability, risk depends on the consequences of a criticality, which were conservatively calculated.



The risk factors for cancer induction used in the EIS were taken from the most recent Commission on Radiological Protection recommendations (1990 Recommendations of the Commission of Radiological Protection), which reflect the most recent and most wide of all currently available data. The authors of ICRP 60 reviewed all available studies. Appendix A provides a useful primer on radioactivity and radiation dose. Volume 2, provides a discussion of how radiation doses were calculated and how cancer risks were calculated. Volume 1, Appendix D, section F.1.3.3 and Volume 2, Appendix F-4 discuss the terms and factors used by the International Commission on Radiological Protection and how they are applied in calculating the effects of radiation on human health in this EIS. Cancer fatalities were used in the EIS to summarize and compare the results, since they are of the greatest interest to the most people. The typographical error in Volume 2 has been corrected.

## II COMMENT

The commentor asks DOE to clarify whether the "accident scenario with the highest risk" in the Summary is equivalent to the "maximum credible accident" or "maximum conceivable accident" or "maximum foreseeable accident" or "maximum reasonably foreseeable accident" as reported in Volume 2.

### RESPONSE

The accident scenario with the highest risk as reported in the Summary is not necessarily "maximum credible" or "maximum conceivable" or "maximum foreseeable" or "maximum reasonably foreseeable" accident. The evaluation of facility accidents in Volume 1, Appendix E 5.15; and Volume 2, section 5.14 consider a range of accidents, from relatively common handling accidents, to very rare events, such as an aircraft crash into a facility. "Maximum reasonably foreseeable" accidents. For NEPA purposes, they are accidents with catastrophic consequences even if their probability of occurrence is low, provided the impacts are supported by credible scientific evidence, is not based on pure conjecture of reason" [40 CFR section 1502.22(b)]. In many cases, these accidents were beyond the facilities and more severe than the maximum reasonably foreseeable accident for which accident risks were determined by multiplying accident consequences by accident probability. The accidents with the highest risk are reported in the Summary because they bound the risks from

## II COMMENT

The commentor suggests that because of the potential for causing contamination in the event of a seismically initiated Mackay Dam failure, a dynamic analysis of the dam structure is warranted to determine its level of seismic resistance.

### RESPONSE

DOE considered the failure of the Mackay Dam in its analysis and found that the consequences of the potential event would be much less than the maximum reasonably foreseeable accident. As a result, a dynamic analysis of the dam structure to determine its level of seismic resistance is unwarranted.

Mackay Dam is an earthenfill structure completed in 1917 and has a storage capacity of 1.5 million acre-feet. The dam was not built to conform to seismic or hydrologic design criteria. In 1978 it was classified as a high-hazard dam by the State of Idaho, based on inspections by the Idaho Department of Water Resources and the U.S. Army Corps of Engineers (Phase 1 Inspection Report). The dam is located 10 miles northwest of the epicenter of the 1983 Borah Peak earthquake. Following the earthquake, the dam was inspected and there was no structural damage to the dam or the outlet works. The dam's ability to withstand severe seismic activity is unknown, but the performance during the Borah Peak earthquake demonstrated the stability of the embankment during moderate ground motion (Flood Routing Analysis for a Failure of Mackay Dam). Following the earthquake, stabilization work was completed on the right abutment of the dam and it was cleared of rock debris. The dam was inspected by the Idaho Department of Water Resources and a certificate was issued for continued operation of the dam and storage (Letter, Department of Water Resources to Mr. J. Doyle Jensen, Big Lost River Irrigation District, April 20, 1983). In spite of the good record for the dam, various postulated dam failure scenarios have been considered in regard to flooding of INEL facilities. These postulated failures include piping failure, dam collapse, and overtopping of the dam structure during the hypothetical probable maximum flood. In all cases, the reservoir was assumed to be full at the start of the initiating event.

failure, the failure was assumed to occur during the 25-year return period flood with full reservoir of 4,030 cubic feet per second (Flood Routing Analysis for a Failure). These conditions bound any additional water that could be impounded by ice dams above because the Big Lost River plain is relatively flat and the depth of the river is small (feet), making the storage of significant bodies of water behind ice dams beyond realistic. In all the above cases, it is assumed that the Big Lost River diversion dam would be floodwaters, with the probable maximum flood being by far the worst case (Flood Routing Analysis for a Failure of Mackay Dam). The probability of a probable maximum flood leading to dam failure is estimated to be less than  $1.0 \times 10^{-6}$  per year [Flood Evaluation Study; Radioactive Waste Complex (Draft)]. Although the probability for a seismically induced failure of the dam was not calculated, the probability of seismic failure causing total collapse, coupled with the dam would overtop dikes at the RWMC (Safety Analysis Report for the Radioactive Waste Complex at the Idaho National Engineering Laboratory), although there would be some contamination in the Reactor Area, ICPP, Expanded Core Facility, and Test Area North areas (Flood Routing Analysis for a Failure of Mackay Dam). Even for probable maximum flood conditions, the flood water transported contamination would be contained within the boundaries of INEL (Flood Routing Analysis for a Failure of Mackay Dam). Groundwater contamination could be introduced during an accident has been bounded by the assessment of a seismic failure of the high-level tanks which is assumed to rupture one or more tanks, releasing 300,000 gallons of high-level waste beneath the ICPP tank farm. The maximum reasonably foreseeable event would be a seismic event more severe than flood-induced contamination over a large surface area.

## II COMMENT

The commentor expresses the opinion that DOE and the Department of Defense should be more cautious in disposing of radioactive waste because the area at the Idaho National Engineering Laboratory is seismically and volcanically active and could cause a radioactive release to the Snake River Plain. RESPONSE

Seismic hazards and geologic analyses can be found in Volume 1, section 4.2 and Appendix F-2, and Volume 2, section 4.6 and Appendix F-2. Seismically induced accidents are discussed in section 5.14 and Appendix F-5. DOE takes seismic hazards very seriously, and INEL has reviewed analyses to support the enforcement and implementation of DOE Orders and standards. Extensive effort has occurred over the past several years to upgrade DOE Orders and standards for natural phenomena hazards. Studies have been under way for many years and are continuing to ensure that seismic hazard characterization is based on up-to-date information and methods. New geologic information on seismic hazard characterization is continually being determined. It is not clear if additional geologic studies are needed.

DOE has analyzed the effects of a hypothetical lava flow event at INEL. The geologic flow is discussed in Volume 2, section 4.6.4, and the estimated consequences of such various alternatives are shown in Volume 2, section 5.14, Tables 5.14-3, -5, -6, -8. The methodology used for performing these analyses is documented in Volume 2, Appendix F-2. Accident Assessments for the Idaho National Engineering Laboratory Facilities. As part of these analyses, the DOE used conservative assumptions to account for the uncertainty in modeling an accident involving molten lava coming into contact with radioactive materials. The potential impacts on the public are well below DOE's Nuclear Safety Policy.

DOE has considered the potential for a volcanic ashfall event at INEL in Volume 2, section 4.6.4, and Appendix F-2.1.2. As stated in section 4.6.4, potential ashfall events are bounded by the site. The risk associated with an ashfall event is bounded by the accidents evaluated in section 5.14. The impacts on the Hanford Site resulting from the Mount St. Helens eruption were small. The Volcanism Working Group (Assessment of Potential Volcanic Hazards at the Production Reactor Site at the Idaho National Engineering Laboratory) determined that volcanic events are small for INEL. Therefore, a silicic ash-flow hazard at the INEL is not a reasonably foreseeable significant adverse impact on the human environment.

A hypothetical accident involving the instantaneous release of the contents of a high-level waste tank represents the situation with the most potential impact on the Snake River Plain aquifer under geologic conditions at the INEL and is discussed in Volume 2, sections 5.14 and Vol 2. Under this scenario, maximum radionuclide concentrations are predicted to reach the Snake River Plain aquifer years after the accident and predicted concentrations will be less than EPA MCLs or DOE Order 5480.28, Natural Phenomena Hazards Mitigation, sets forth DOE procedures to assess, and operate DOE facilities so that workers, the general public, and the environment are protected from the impacts of natural phenomena hazards on DOE facilities. This Order specifies that the impacts of natural phenomena hazards on DOE facilities be re-evaluated upon any change in design and construction standards. Existing facilities

undergone continual safety analysis and seismic design review. Several of the projects in Volume 2, Appendix C of the EIS are proposed by DOE to replace or upgrade facilities. Likewise, actions such as the transfer of fuels from potentially vulnerable facilities resulted from the ongoing safety analysis and seismic design reviews.

No new analyses are required for DOE Idaho Operations Office-managed facilities because the EIS summarizes existing credible scientific evidence relevant to understanding the existing environment, identifying reasonably foreseeable impacts, and evaluating potential consequences. The EIS is based on methods generally accepted by the scientific community. The EIS evaluates the potential consequences including direct, indirect, cumulative, and secondary effects and long-term productivity losses. See also the responses to comments 05.0 05.08.01 (030).

General discussions of waste management procedures and plans are covered in Volume 2. Therein it is noted that the DOE is committed to a strategy emphasizing waste minimization, with the goal being that most newly generated radioactive waste will be managed through necessary cleanup activities and decommissioning of contaminated facilities that no longer have a mission. The DOE complex-wide management and cleanup of wastes associated with these activities is outside the scope of this EIS. However, they are currently being addressed in the Waste Management Programmatic EIS.

With respect to cleaning up INEL, the INEL Environmental Restoration Program, including remediation and decontamination and decommissioning, is discussed in Volume 2, section 02.04 (047). The significant progress already made in this program at INEL, see text 02.04 (047).

The generation and storage of SNF is discussed in Volume 1, section 1.1. Therein it is noted that DOE SNF was generated in DOE production and experimental reactors that have ceased operations. Considerable source reduction has already occurred. See Volume 1, Appendix E for further discussion of experimental reactors. In addition, the Navy is developing longer-lived Naval reactors that will reduce the amount of SNF that will be generated. Completely eliminating the source of SNF is outside the scope of this EIS.

## II COMMENT

The commentor suggests that an additional failure scenario of the Mackay Dam be evaluated. RESPONSE

The Mackay Dam failure scenarios analysis in Flood Routing Analysis for a Failure or Breach cited in the EIS includes a probable maximum flood scenario considered to be the most reasonably possible using NRC siting criteria for commercial nuclear reactors. The study includes sensitivity analyses that indicate significant changes in parameters variations in flooding at INEL. Therefore, DOE believes the Mackay Dam failure model assesses reasonably foreseeable INEL flooding hazards that could occur as a result of a dam failure. The combination of probable maximum flood estimated frequency and addition of their probabilities would result in flooding hazards with probabilities lower than those reasonably foreseeable.

No new analyses are required for DOE Idaho Operations Office-managed facilities because the EIS summarizes credible scientific evidence relevant to understanding the existing environment, identifying reasonably foreseeable impacts, and evaluating potential consequences. This information is provided in Volume 2, section 4.8 and Volume 2, Appendix F-2.

The results of accident analyses (including beyond reasonably foreseeable accidents) indicate that the risk to the public from this EIS would be small. Therefore, additional information or characterization of seismic events with lesser potential impact would have no effect on the decision-making process.

## II COMMENT

Commentors state that nuclear waste, spent nuclear fuel, and other dangerous materials could be released in the event of an accident. RESPONSE

Volume 1, Chapters 3 and 5 and site-specific Appendices A through F, and Volume 2, Appendix F discuss risks to the public, workers, and the environment, and secondarily from a range of potential accidents. The discussions include evaluations and analyses of the risks from a range of potential accidents. Although DOE cannot guarantee that no accidents will occur, the results of evaluations and analyses indicate that the risks are small.

this EIS indicate that risks to workers, the public, and the environment would be s alternatives considered. (See the EIS Summary, Public and Worker Health Effects.)

## II COMMENT

The commentor suggests that the EIS discuss an accident at the Idaho National Engin involving up to 6,000 gallons of hydrofluoric acid.

### RESPONSE

An accidental release of hydrofluoric acid is discussed in Volume 1, Appendix B, se Hydrofluoric acid is stored outside in the ICPP facility area in a 30,290-liter (8, Although there are only about 11,356 liters (3,000 gallons) in the tank, the accide assuming a full storage tank. The tank is over a catch basin that would contain th the tank ruptures or if there is a piping failure. All the tank's contents were as The amount of hydrofluoric acid released and the surface area of the acid in the ca considered in the analysis. Downwind concentrations of acid are independent of the spilled and depend only on the evaporation rate from the catch basin. The evaporat depends on the surface area of the catch basin, as well as other factors. The dura however, depends on the total amount of acid spilled.

The EIS shows that the consequence of this potential event at the nearest site boun per cubic meter of hydrofluoric acid. As to the impact to the maximally exposed in concentration represents 0.2 percent of the Emergency Response Planning Guide Level hydrofluoric acid. For reference purposes, 100 percent of the ERPG-3 level is the of the specific toxic material from which a person not wearing a respirator could e without having his ability to escape impaired or experiencing irreversible side eff

## II COMMENT

The commentor questions whether the maximally exposed individual is the person at t recommends that further analysis be done to show that this individual has indeed re individual dose.

### RESPONSE

The accident analyses in the EIS were performed with the plume rise going to the lo maximum dose is received. See Volume 1, section 5.1.

## II COMMENT

The commentor suggests that after an accident, communication with members of the pu consume contaminated vegetables and other food produced in the vicinity is not well Idaho National Engineering Laboratory.

### RESPONSE

Volume 1, section 5.7 and Volume 2, section 5.19 discuss accident mitigation. DOE Orders specifying the requirements for emergency preparedness, and each DOE site ha emergency management program. These programs are developed and maintained to ensur response for most accident conditions and to provide the framework to readily exten accidents not specifically considered.

The emergency management program incorporates activities associated with planning, response, including simulated emergency exercises with states, counties, and other preparedness requirements for the facilities would be part of the planning that wou Command, control, and communication are key parts of these emergency management pro the details of such planning are beyond the scope of the EIS.

For the off-site population, the need for any protective action would be based on t the protective action guides developed by EPA. Interdiction activities by INEL acc personnel are expected to take place following an accident to limit doses to off-si This interdiction can limit ingestion exposure to the public.

For accidents with maximum reasonably foreseeable consequences at INEL, interdictio with protective action guides was assumed in the EIS analyses. Doses resulting fro were calculated assuming contaminated foods comprised 10 percent of the person's 1- the accident. More information on the parameters used in the accident analysis an

regarding ingestion of contaminated food can be found in Accident Assessments for I Engineering Laboratory Facilities, sections 2.1.2. and 2.1.3.

## II COMMENT

Commentors raise the issue of impacts a nuclear accident could have on the State of on tourism and the economy.

### RESPONSE

Volume 1, Chapter 5 and Appendices A through F, and Volume 2, Chapters 3 and 5 and the EIS discuss reasonably foreseeable accidents and their impacts. Although DOE c accidents will be prevented or that contamination will not occur, for all alternati the risk to workers and the public from facility accidents would be small. DOE exp reasonably foreseeable accidents on tourism and the economy would be limited and of noted in Volume 2, section 5.14, there would be a potential for limited economic im

## II COMMENT

The commentor wants to better understand the assumptions used to determine risk acc constitutes acceptable risk, and who is responsible for this determination.

### RESPONSE

Risks are presented in the EIS without a determination of acceptability. Acceptabl determined only by the individual.

As used in this EIS, risk is defined as the product of the probability of an event that event. Volume 1, Appendices A through F, and Volume 2, Appendix F provide the risk analyses for this EIS were performed.

## II COMMENT

Commentors state that accidents, accidental releases, and long-term effects of acci RESPONSE

DOE cannot guarantee that accidents will not occur. Given that Volumes 1 and 2, Ch the results of analyses of reasonably foreseeable accidents. Volumes 1 and 2, Chap impact avoidance and mitigation measures. These analyses show that the risks of re accidents under all the alternatives considered would be small.

## II COMMENT

The commentor states that assumed ground-level releases from a facility accident ma impacts to the off-site population, because the modeling assumptions bias the model conclusions of the accident analysis. An example provided is that a small number o release point receive a higher dose than the large numbers of members of the public perimeter.

### RESPONSE

The environmental impact analyses are designed to produce a reasonable projection o potential environmental consequences. This requires the use of appropriately conse and analytical approaches. In this context, "conservative" means that an assumptio to overproduced, rather than underpredict, any adverse impacts. However, unreasona analyses do not provide a useful basis for comparing alternatives. Therefore, the unreasonable conservatism and base the environmental impact analyses on realistic, information whenever possible. Facility accidents were modeled using a release ele the specific accident scenario. For example, some scenarios would have an elevated through a stack, and others would have a ground-level release point. Each alternat using comparable methods and levels of conservatism so that the relative impacts of assessed accurately.

Volume 2, Appendix F-5.3.1 has been revised to state that the methods used in the a higher estimates of radiation exposures near the point of release.

## II COMMENT

The commentor suggests that following an accident, certain roadways could be inaccessibility due to direction or weather conditions, and that this should be acknowledged.

### RESPONSE

The EIS has been changed to acknowledge that under certain conditions, the ability to use designated evacuation routes could be impeded.

## II COMMENT

The commentor requests clarification of what is meant by "not credible" with respect to the Hanford Site.

### RESPONSE

The EIS has been revised to explain that if an event has a probability of occurring within a million years, additional analyses were not performed.

## II COMMENT

The commentor states that there could be a considerable error in the assumption that the portion of contaminated foods consumed in the year following an accident for a person at the Engineering Laboratory's nearest site boundary would be 10 percent of their diet.

### RESPONSE

For the purposes of this EIS, accident assessments were performed using realistic, conservative assumptions. As part of the health impact analysis to the maximum extent practicable following a potential accident, that individual's total dose received comprises four pathways: immersion, inhalation, ingestion, and direct ground-surface exposure.

That portion of the dose resulting from the ingestion pathway was calculated assuming that it comprised 10 percent of the person's 1-year diet following the accident and that there were no protective actions unless EPA protective action guides were projected to be exceeded. The assumption is based on an engineering judgment of what is reasonable for most of the people living near the site, as well as to try to make the scenario realistic, but generally conserving the percentage to a greater value would represent an unwarranted overconservatism in the health impact analysis.

The environmental impact analyses are designed to provide a reasonable projection of potential environmental consequences. This requires the use of appropriate conservative and analytical approaches. In this context, "conservative" means that the analyses tend to overpredict, rather than underpredict any adverse impacts. However, overconservative analyses do not provide a useful basis for comparison among alternatives.

More information on the parameters used in the accident analyses and the assumption of contaminated food can be found in Accident Assessments for Idaho National Engineered Facilities, sections 2.1.2 and 2.1.3.

## II COMMENT

The commentor suggests that the degrading structural integrity of spent nuclear fuel is a significant factor and that the EIS should include this prominent factor in the discussion of the No Action alternative. As an example, the commentor states that the degraded fuel at the Hanford Site is contributing to elevated radionuclide activities, which contaminates the groundwater in the Columbia River.

### RESPONSE

The accident risks presented in the EIS for the No Action alternative reflect an assessment of accident probabilities associated with SNF management, including the probabilities of degraded (vulnerable) fuels and facilities. Under the No Action alternative, DOE will ensure the minimum necessary for safe and secure management of SNF at the generation site location.

Volume 2, section 5.1.2 has been modified to state: "Consequences would be bounded accident assessments, but likelihood may increase."

## II COMMENT

The commentor states that the cumulative impacts from more than one accident initiating a major earthquake must be evaluated in the EIS.

### RESPONSE

As discussed in Volume 2, section 4.6.3, seismic events were found to be the most likely initiators with the potential to cause releases at more than one facility and involve multiple facilities. Further, the potential for simultaneous accidents caused by a single seismic event is discussed in Volume 2, section 5.14.2. DOE's analysis shows that potential multiple-facility releases resulting from the postulated accidents at the Argonne National Laboratory-West Hot Facility. Consistent with the accident selection methodology described in Volume 1, section 5.15.3, the consequences and risks associated with multiple facility releases were considered because they do not represent the maximum reasonably foreseeable accident frequency categories defined in Volume 1, Appendix B, Table 5.15-5.

## II COMMENT

The commentor states that nonradiological health effects resulting from an accident involving materials through a groundwater or surface water pathway at the Idaho National Engineering and Environmental Laboratory have been overlooked.

### RESPONSE

Such events are summarized in Volume 2, section 5.8. Under all of the alternatives possible future sources of contamination would be small compared with previous practices. This section DOE concludes that (a) only contaminant concentrations below EPA MCLs would migrate beyond the site boundary, resulting in small impact to the quality of the INEL site; (b) adverse effects to groundwater quality have occurred in localized areas (contaminant plumes), but these plumes have not affected the regional quality of groundwater resources (through source reduction and reduction of contaminants existing in water resources); (c) computer modeling of vadose zone contaminant transport indicates that contaminant plumes with concentrations above MCLs would continue to decrease at least through 2030 and the overall quality of the aquifer would be improving; and (d) water use at the INEL site for any alternative would have a minimum quantity of water in the Snake River Plain aquifer.

## II COMMENT

The commentor suggests a seismically induced accident associated with the 100-K basins included in the Hanford Site accident assessments since an "unreviewed safety question" was raised on May 5, 1994.

### RESPONSE

A discussion of the seismic effect on the 100-K basins has been added to Volume 1, section 5.15.

## II COMMENT

The commentor states that thousands of cancers could result from one mistake that could involve transportation or a criticality in an inversion layer.

### RESPONSE

Volumes 1 and 2, Chapter 5 discuss the probabilities and consequences of transportation accidents, including those caused by human error. These discussions and their supporting data include extensive evaluations of accident consequences using generally accepted engineering practices including analysis under various meteorological conditions. The EIS shows that the consequences of public from facility and transportation accidents would be small for the alternative.

## II COMMENT

The commentor states that a dam failure, rather than flooding at the Hanford Site, inundate spent nuclear fuel facilities. A reference to the dam failure discussion  
RESPONSE

Volume 1, Appendix A, section 4.8 discusses natural flooding at the Hanford Site be potential for collapse of the shoreline along the riverbank in the White Bluffs are dam failure in Appendix A has been added. Neither the probable maximum flood, nor collapse of the shoreline in the White Bluffs area would impact SNF operations at t Flooding from a 50 percent failure of Grand Coulee Dam would inundate the K-basins.

## II COMMENT

The commentor states that only "worst case" accidents should be the basis for a dec  
RESPONSE

CEQ regulations no longer require analysis of worst-case accidents. Rather, CEQ re assessment of effects of reasonably foreseeable accidents. In accordance with CEQ guidance, the evaluation of reasonably foreseeable accidents in the EIS considers b section 5.15 and Volume 2, section 5.14.) The high-risk and high-consequence accid because they produce effects that are very unlikely to be exceeded by severe accide they could potentially represent a higher risk (risk = probability x consequence) probability accidents with higher consequences. The EIS shows that the risk to wor from all accidents analyzed would be small for all alternatives considered.

## II COMMENT

The commentor notes that spent nuclear fuel is dangerous, but that so is gasoline i If gasoline had the same handling requirements as spent nuclear fuel, it would be t  
RESPONSE

DOE agrees that potential consequences from accidents involving some hazardous mate greater than those from SNF management.

## II COMMENT

The commentor states that DOE has not considered impacts from shipboard fires and e  
RESPONSE

The EIS addresses seismicity in Volume 1, section 5.2.4, accidents in Volume 1, sec accidents involving shipboard fires in Volume 1, Appendix D, section F-1.4.4. Loca SNF management have emergency action plans and equipment to respond to accidents an emergencies. Shipboard fires would be included as one of the types of accidents, i location. The plans would be updated to cover any new SNF facilities and activitie coordinate activities with state and local agencies to establish and implement an a response training program for potential accidents for the location. The details of the scope of the EIS.

## II II II II COMMENT

The commentor states that the source terms in Volume 2, Table 4.7.1 are constants a range of values over a 10-year period. Additionally the commentor requests project under postulated abnormal conditions involving several facilities.

RESPONSE

The projection requested by the commentor is provided in Volume 2, Chapter 5, which impacts of the alternatives, including impacts under abnormal and accident conditio



## II 5.12 Transportation Issues

### II COMMENT

Commentors object to the shipment of radioactive material because the risk is perceived. Commentors state that an adequate study of the worst-case accident is needed and a publicly funded response team training, and that some longshoremen may refuse to handle shipments.

#### RESPONSE

DOE complies with the DOT regulations for transporting radioactive material. These regulations are designed to protect workers and the public by minimizing the risks associated with the transport of radioactive material. The EIS analyzes a full range of alternatives, from no action, which involves no transport of radioactive material, to centralization, which involves extensive transport of radioactive material. For all alternatives, the potential risks from transportation would be the same risks associated with maximum reasonably foreseeable accidents. The probabilities of maximum reasonably foreseeable transportation accidents are discussed and evaluated in Appendices D and I. Although the consequences of an accident of this type might be of such an accident having high consequences is on the order of one chance in 10 million, the consequences of most accidents, including those with a probability of occurring more than once in 10,000 years, are less than those of the accidents analyzed.

With more than 50 years of radioactive material transportation in the commercial sector, there have been few transportation accidents involving radioactive materials, and a very small amount of release of radioactivity. Nonetheless, emergency response teams are in place throughout the United States to respond quickly in the event of a transportation accident. The importance of preparedness for potential accidents involving SNF transportation is emphasized. FEMA provides training and materials to local emergency responders to prepare them to respond properly. DOE provides for Radiological Assistance Program teams, which consist of personnel equipped and prepared to quickly respond to an accident, and assist local emergency responders. This response network, along with other preventive measures, such as safety reviews and testing, and adherence to stringent regulations, supports the continued safe shipment of SNF shipping containers that could be handled by longshore workers are designed to meet international standards for safety, including radiation levels at the outside of the containers. This EIS analyzes transportation from ports of entry. The potential for radiological accidents involving workers is within the scope of the EIS entitled Proposed Nuclear Weapons Nonproliferation Concerning Foreign Research Reactor Spent Nuclear Fuel.

### II COMMENT

One commentor states a definition of the term "general transportation" in Appendix I. RESPONSE

The term "general transportation" is discussed in Volume 1, Appendix I, section I-9 "transportation activities that take place that are unrelated to the alternatives evaluated in the EIS." Examples of these activities are shipments of radiologically hazardous materials to nuclear medicine laboratories and shipments of commercial low-level radioactive waste to disposal facilities. The activities described by general transportation activities are independent of DOE work and over which DOE has no control.

### II COMMENT

The commentor states that the EIS should address the condition of the transportation infrastructure (rail lines, crossings, bridges, and tunnels).

#### RESPONSE

Adequate rail lines, crossings, bridges, and tunnels exist to support the SNF transportation. SNF requires no special transportation infrastructure that is not also necessary for other commodities in the United States today. DOT is the regulatory agency responsible for enforcing the standards for the transportation infrastructure.

## II COMMENT

The commentor states that DOE should halt shipments of spent nuclear fuel during in  
RESPONSE

Although the comment is not specifically related to the effects of weather on SNF t  
response applies for radioactive material transportation. DOT requirements for con  
modeling codes used to analyze potential impacts of transportation account for such  
accidents, natural phenomenon, etc.

## II COMMENT

The commentor states that the EIS used a generic approach to the mitigation of impa  
secondary route comparison factors discussed in the mitigation section are critical  
Idaho. The commentor also notes that TRANSAX-92 demonstrated that state corridor em  
responders are not prepared for radiological incidents.

RESPONSE

The primary and secondary route comparison factors discussed in the mitigation sect  
DOT; DOE and the Navy believe them to be accurate. Pursuant to 49 CFR 397.67, moto  
transporting hazardous material required to be placarded or marked in accordance wi  
and not subject to a nonradioactive hazardous material routing designation, shall o  
routes that do not go through heavily populated areas, places where crowds are asse  
streets, or alleys, except where the motor carrier determines that: (1) there is  
(2) a reasonable deviation is necessary to reach terminals, points of loading and  
food, fuel, repairs, rest, or a safe haven; or (3) a reasonable deviation is requir  
such as a detour that has been established by a highway authority, or a situation e  
enforcement official requires the drivers to take an alternate route.

DOE participates with other Federal, state, and local authorities to sponsor and fu  
response training courses throughout the United States. These courses are provided  
and local authorities responsible for public safety and emergency response to natur  
equipped state and Federal emergency response teams that are quickly available to a  
in the event of an emergency.

## II COMMENT

The commentor states that DOE does not have a good record with respect to building  
nuclear fuel casks and waste repositories, and getting the cooperation of the state  
period of time.

RESPONSE

The commentor is referring to lengthy delays in the construction and opening of the  
Waste Isolation Pilot Plant sites, as well as the 5- to-10-year time period for des  
radioactive material shipping casks.

DOE operates within the framework of Federal regulations and DOE policy, which are  
and stakeholder involvement when procuring shipping casks or constructing new facil  
such a process is costly and time consuming; however, DOE feels it is a process tha  
opportunity to obtain facilities or apparatus designed with the highest standards o  
public/stakeholder input into the process.

## II COMMENT

The commentor states that DOE did not address the environmental impacts of moving s  
RESPONSE

Volume 1, Appendices D and I analyze the transportation of SNF. NEPA, 42 USC Secti  
and CEQ regulations at 40 CFR Part 1500 et. seq. require that an EIS describe the p  
the proposed action; alternatives, including no action; the affected environment; a  
consequences associated with the proposed action and alternatives. Volumes 1 and

these requirements. In each volume, Chapter 2 describes the purpose and need for t Chapter 3 describes the alternatives being considered; Chapter 4 describes the affe Chapter 5 describes the environmental consequences.

Input was solicited from the public during a 90-day public comment period, which al send written comments, give oral comments and send a facsimile by a toll-free telep one or more of the 33 public hearings held in 20 locations around the United States All supporting documents referenced in the EIS are on file and are available to the

## II COMMENT

The commentor requests specific information on the number of 40-year-period spent n shipments, highway routes affected, and populations exposed to risks.

### RESPONSE

Specific information on the number of SNF shipments is in Volume 1, Appendices D an The HIGHWAY computer code predicts highway routes for transporting radioactive mate United States. The HIGHWAY code currently describes approximately 240,000 miles of complete description of the interstate highway system, United States highways, most highways, and a number of local and community highways are identified in the databa HIGHWAY computer code calculates routes that maximize the use of interstate highway allows the user to predict routes for shipping radioactive materials that conform t specified in 49 CFR Part 177). The routes calculated conform to applicable guideli therefore, they represent routes that could be used.

The impacts of transportation for all programmatic alternatives considered in this

## II COMMENT

The commentor questions the need for cross-country shipments under the Regionalizat alternative.

### RESPONSE

For the Regionalization by geography alternative, all existing and future SNF would destination site without crossing the Mississippi River. However, there would be c of Naval SNF. To examine all Naval SNF in a cost effective manner, examination wou location. Because the Navy defuels and refuels ships at shipyards on the east and the alternatives analyzed in the EIS are found in Volume 1, Chapter 3.

## II COMMENT

The commentor states that a history of the movement of spent nuclear fuel is not in specific example that gives the understanding that all previous shipments of spent the Savannah River Site from Newport News/Hampton Roads have been transported by tr many hundreds of shipments. Yet, the discussion of movements out of the Newport Ne area in Volume 1, section 4.6.2 mentions only 10 shipments, each conducted by rail.

### RESPONSE

The EIS conducted a comprehensive transportation cumulative impacts analysis, evalu present, and future or projected shipments of radioactive material, which includes SNF. Dose information is contained in Volume 1, Appendix I. The transportation cu analyses includes historical shipments of SNF and is found in Volume 1, Appendix D Appendix I for non-Naval SNF.

The example given by the commentor refers to Naval SNF shipments, which travel by r references provided in Table I-58 contain the historical data for non-Naval SNF shi predominantly travel by truck.

## II COMMENT

The commentor suggests specific information regarding Fort St. Vrain fuel, number o destination facility, and inventory be added to the Final EIS.

**RESPONSE**

The EIS already contains this information in either Volume 1 or Volume 2. Volume 1 1994 letter to distribution from T.L. Wichmann, Spent Nuclear Fuel Inventory Data,, information regarding quantity of Fort St. Vrain fuel currently stored at INEL and be received in the future. The quantity that could be received could be stored at may be managed in other facilities and in other ways. The EIS has bounded the info assumptions and methodologies used in calculating the individual and cumulative imp EIS is considered to bound the information suggested by the commentor, the EIS has

**II COMMENT**

The commentor states that the EIS concentrates on the radiological impacts of trans exclusion of the other hazardous materials.

**RESPONSE**

Volume 2, section 5.11 discusses the transportation of both hazardous and radioacti incident-free and accident cases. In incident-free transportation, there are no em being transported, so the only hazardous materials emissions considered were those sulfur dioxide present in urban population zones. The methodologies for determinin impacts associated with hazardous materials transportation accidents are discussed 5.11.1. The analysis of the maximum reasonably foreseeable case truck accident sce alternatives is in Volume 2, Table 5.11-15. The impacts of a hazardous material tr are low under all alternatives.

**II COMMENT**

The commentor states that the EIS should discuss the impacts of the increase in hig associated roadway congestion, as well as the impacts of increased rail traffic.

**RESPONSE**

A discussion of highway and rail transportation impacts and potential accident impa the EIS entitled Traffic and Transportation, Transportation, and Offsite Transporta public and agency comments, DOE has modified descriptions of on-site traffic patter DOE complies with the DOT requirements for off-site transportation of SNF, includin shipping containers that meet DOT performance requirements. As a result, the poten public to radiation hazards is extremely low. DOE further minimizes accident risks and route-selection guidelines and uses other procedural controls for hazardous and In the unlikely event of an accident, DOE and local governmental authorities will i response measures. As described in the EIS Summary, Public and Worker Health Effec overall risk from transportation would be small. See also the response to comment 05.12 (003).

**II COMMENT**

The commentor expresses concern that the EIS inadequately addresses the nonradiolog transportation activities, and questions the adequacy of the 1982 reference documen

**RESPONSE**

Incident-free nonradiological fatalities were estimated using unit risk factors. T account for the fatalities associated with exhaust emissions, but the distances use must be doubled to reflect the round-trip distance, because these impacts occur whe shipment contains radioactive material. Two sets of data were evaluated: 1) data Impacts of Transporting Radioactive Material and 2) data from the Motor Vehicle-Rel Study. In Non-radiological Impacts of Transporting Radioactive Material, the nonra factor for trucks was  $1.0 \times 10^{-7}$  fatalities per kilometer, and the nonradiological was  $1.3 \times 10^{-7}$  fatalities per kilometer. These unit risk factors are applicable on Vehicle-Related Air Toxics Study the unit risk factor was calculated to be  $7.2 \times 10^{-7}$  per kilometer; this unit risk factor is applicable in all areas (i.e., rural, suburban, routes analyzed in this EIS, the unit risk factors from Non-radiological Impacts of Radioactive Material were found to overestimate impacts by about 20 or 30 times rel factors from Motor Vehicle-Related Air Toxics Study. Therefore, the unit risk fact

Non-radiological Impacts of Transporting Radioactive Material were used as a conser the incident-free nonradiological fatalities presented in this EIS. Unit risk fact Impacts of Transporting Radioactive Material account for all fatalities, not just c effects of chronic exposure to diesel exhaust emissions have been followed in occup workers, but these data are not sufficient to make a correlation between the effec experienced (Motor Vehicle-Related Air Toxics Study). Therefore, these impacts wer EIS.

## II II II COMMENT

The commentor states that the Mackay Branch has been abandoned by the Union Pacific there is an application before the Interstate Commerce Commission to abandon the Sc Arco, Idaho, to Mile Post 43.

### RESPONSE

The map showing the Mackay Branch will be corrected to reflect abandonments by the Railroad.

## II II COMMENT

The commentor states that purpose-built ships would greatly add to the safety of ha reactor spent nuclear fuel shipped to ports in the United States.

### RESPONSE

The risks associated with the transport by ship of FRR SNF and its handling at U.S. purpose-built ships, are being evaluated in the EIS entitled Proposed Nuclear Weapo Policy Concerning Foreign Research Reactor Spent Nuclear Fuel (Draft).

## II COMMENT

Commentors question the choice of ports of entry to the United States that are anal state that the EIS does not consider transportation or radioactive material handlin shipboard fires, at port facilities.

### RESPONSE

The analysis of impacts at port facilities and nearby communities, the specific por the overseas transportation of FRR SNF to United States ports is being addressed in Proposed Nuclear Weapons Nonproliferation Policy Concerning Foreign Research Reacto Nuclear Fuel (Draft) (FRR EIS). Only the impacts of transportation of SNF from the DOE facilities are analyzed in this EIS.

The criteria used to choose the ports of entry are outlined in the Notice of Intent Register Vol. 58, No. 202, October 21, 1993, pages 54336-54340). These criteria in of harbor and dock characteristics to satisfy the cask-carrying ship requirements, secure lag storage, (c) adequacy of overland transportation systems from ports to t experience in safe and secure handling of hazardous cargo, (e) emergency preparedne and nearby communities, and (f) proximity to the proposed storage sites. A range o also be analyzed in the FRR EIS. The decision regarding port selection will not b EIS and the FRR EIS are completed. In addition, in response to public comments, thi that results in a shipboard fire approximately 2 miles from Seattle (Volume 1, Appe

## II COMMENT

The commentor is concerned that Puget Sound will be a possible point of entry for h of radioactive material and that the DOE fails to recognize the danger for this urb  
RESPONSE

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In addition, in response to public comments, this EIS discusses the consequences of that results in a shipboard fire approximately 2 miles from Seattle (Volume 1, Appendix

## II II COMMENT

A commentor raises the issue of the proposed movement of nuclear waste from Washington to Tennessee in what his sources indicate are leaky containers.

### RESPONSE

DOE is not proposing to ship nuclear waste from Washington, DC, and believes that it will have Washington state or other states with DOE facilities in mind. DOE is evaluating alternatives that would entail transporting SNF to ORR for storage. Any transportation conducted in compliance with DOT regulations using NRC transportation standards.

## II COMMENT

A commentor provides recommendations for the packaging of radioactive materials for transportation.

DOE complies with the applicable requirements of DOT regulations covering the packaging of radioactive materials. DOE has conducted analyses using representative packaging for radioactive materials. If an alternative is chosen that requires transportation of radioactive materials, by the commentor will be considered. These analyses are adequate for comparison of consideration in this programmatic EIS.

## II COMMENT

The commentor believes the EIS does not adequately address the potential health effects from radiation from spent nuclear fuel casks.

### RESPONSE

Volume 1, Appendices D and I provide analyses of potential health effects from exposure associated with SNF transportation. These analyses show that the health effects from exposure under all alternatives considered in the programmatic EIS would be small.

## II II COMMENT

The commentor states that the EIS should analyze a more realistic scenario of transportation shipments by truck or all shipments by rail. A combination of the two forms of transportation would be analyzed.

### RESPONSE

The assumption of all shipments by truck or all shipments by rail serves to produce extremes representing the limits of potential transportation impacts; any combination of the two would have impacts between these extremes. Therefore, additional analyses are not needed. If of transport by truck or rail, the potential impacts would be small.

## II COMMENT

The commentor states that the description of the regional transportation infrastructure Site implied that Interstate 90 would be used for shipping campaigns, and that the northern Idaho are not considered in the EIS.

#### RESPONSE

The description of the regional transportation infrastructure is a discussion of the environment at and around the Hanford Site; it is not meant to imply that Interstate shipping campaigns. The analysis of transportation risks is provided in Volume 1, These analyses cover all appropriate shipping routes and show that the risks for all alternatives considered would be small.

## II II COMMENT

One commentor questions the regulatory status of on-site shipments in noncertified RESPONSE

The Hazardous Materials Transportation Act applies only to hazardous material shipment commerce." A letter written in 1991 from the U.S. Department of Transportation, Regulatory Programs Administration, addresses the definition of the term "in commerce" and the Hazardous Materials Transportation Act to shipments conducted on DOE sites. The letter states that shipments conducted in areas to which the general public does not have access are not "in commerce" and as such, need not meet the requirements of 49 CFR. The above notwithstanding, DOE has implemented specific procedures, as required by DOE Order Requirements for the Packaging and Transportation of Hazardous Materials, Hazardous and Hazardous Wastes, which ensures the health and safety of the public and workers on-site shipments. These procedures include (but are not limited to) speed restrict vehicles, and prior notification of appropriate emergency response personnel that take place.

## II COMMENT

Commentors question the adequacy of transportation regulations, including radiation accident safety requirements, and routing. For example, commentors question the extent associated with the shipping containers, the ability of a shipping container to withstand radioactive material shipments.

#### RESPONSE

A brief discussion of transportation regulations is in Appendix I of the EIS. DOE has regulations for shipping radioactive material, which include requirements for external radiation container to withstand hypothetical accident conditions (including fire), and transportation requirements were established by DOT to protect workers and the public and are designed to minimize risks associated with transporting radioactive material. DOE has no reason to question DOT regulations. As discussed in the EIS, the risk from transportation would be very low. The criteria used to choose the ports of entry are outlined in the Notice of Intent Proposed Nuclear Weapons Nonproliferation Policy Concerning Foreign Reactor Spent Nuclear Fuel (FRR EIS). These criteria included: (a) adequacy of harbor and dock characteristics, (b) transportation systems from ports to the storage sites, (c) experience in safe and handling of hazardous cargo, (d) emergency preparedness status at the port and nearby communities, and (e) proximity to the proposed storage sites. A range of alternative ports will also be analyzed. The final decision regarding port selection will not be made until both this EIS and the FRR EIS are completed.

## II COMMENT

Commentors ask about notification and inspection of radioactive materials shipments. Commentors question the inspection of foreign research reactor spent nuclear fuel.

#### RESPONSE

The DOE complies with all DOT regulations regarding notification and inspection of shipments. The inspection of FRR SNF before it reaches the United States would be done by the shipper, who must certify that the radioactive material is in proper condition for shipment.

includes compliance with external radiation and contamination requirements.

## II COMMENT

The commentor states that the EIS has not acknowledged the right of Indian Tribes to transportation of spent nuclear fuel and other hazardous materials across Tribal lands. The Materials Transportation Act.

## RESPONSE

RESPONSE  
DOE is and always has been committed to safe and secure transportation of SNF to ap  
for storage or other management activities. Consistent with this commitment, DOE w  
applicable requirements promulgated by a state, a political subdivision, or an Indi  
authorized and has not been preempted by the Hazardous Materials Transportation Act  
Act, or other applicable Federal law.

## II II II COMMENT

Commentors state that the consequences of the maximum reasonably foreseeable transp are provided only for a rural population zone. The commentor asks about the conseq accident occurred in an urban population zone.

## RESPONSE

NEPA requires that an EIS evaluate reasonably foreseeable impacts from proposed act an accident is considered reasonably foreseeable if it has a probability of at least chance in 10 million years. Factors that affect accident probability include state accidents per kilometer; the fraction of accidents that occur in rural, suburban, a zones; the probability that an accident will be of a certain severity; and the annual rural, suburban, and urban population zones. Weather conditions also affect the probabilities consequences because stable, worst-case, weather conditions are only about one-tenth average weather conditions.

Volume 1, Appendix I, Table I-41 summarizes the maximum reasonably foreseeable tran accident for the Regionalization by geography alternative, in which all SNF is sent SRS. The footnotes to the table state that the maximum reasonably foreseeable acci suburban population zone, not a rural zone. If this same accident were postulated population zone, the accident probability would be less than  $1 \times 10^{-7}$  per year, whi that the scenario was not analyzed.

Volume 1, Appendix I, Table I-31 summarizes the maximum reasonably foreseeable transportation accident for the Decentralization alternative. Footnote "a" to the table states that the foreseeable accident occurs in a rural population zone. If an accident of equal severity occurred in an urban or suburban population zone, the accident probability has been calculated as less than  $1 \times 10^{-7}$  per year, which makes it so unlikely that the scenario was not analyzed. The scenario used to calculate the probability of rail transportation accidents is summarized in Volume 1, Appendix I, Table I-55, which summarizes the maximum reasonably foreseeable transportation accident for the Centralization alternative at ORR. The table shows that under neutral weather conditions the maximum reasonably foreseeable transportation accident could occur in an urban area with a probability of  $1 \times 10^{-7}$  per year. If the accident occurred under stable weather conditions, the probability would be one-tenth of the probability under neutral weather, or  $1 \times 10^{-8}$  per year, which is a chance in 10 million per year. Calculations documented in the references also show that an accident of equal severity in a suburban area also has a probability of less than  $1 \times 10^{-7}$  per year. The probability in a rural population zone, because most of the distance traveled by the shipments would be in rural areas, is a probability greater than  $1 \times 10^{-7}$  per year for an accident of maximum severity to occur. In the worst-case, weather conditions, other less severe accidents would have a probability of less than a chance in 10 million per year in urban and suburban areas under stable, worst-case, weather conditions, but their consequences would be less than the results shown in Table I-55.

The consequences of transportation accidents in rural areas include ingestion doses predominantly agricultural area where residents most likely eat what they produce in contrast to the consequences for transportation accidents in urban and suburban areas include ingestion doses. Residents of these areas are most likely not involved in produce what they eat at their resident location. Therefore, the consequences of t in rural areas may be greater than the consequences in suburban or urban areas, even population densities in the later areas are higher.



## II COMMENT

A commentor states no emergency response systems are set up to respond to transport involving spent nuclear fuel.

### RESPONSE

DOE has developed and implemented emergency response systems to respond to transport involving DOE radioactive materials and SNF. This is discussed in Volume 1, Appendix A, Table 1. Accidents involving SNF have been rare. In the event of an accident involving transit, local fire and police organizations are first to respond. DOE, DOT, and F and training materials to local emergency responders to prepare them to handle accidents. DOE provides Radiological Assistance Program teams, which consist of trained experts equipped to quickly respond to an accident and assist local emergency response personnel if needed. Response network, along with preventive measures, such as shipping container design adherence to stringent regulations, supports the continued safe shipping of SNF. DOE uses the Transcom satellite tracking system for each of its SNF shipments. This transponder located on the trailer with the shipment that relays continuous position data to a satellite to computer terminals at DOE facilities around the country. In the unlikely event of an accident with a shipment, the exact position of the shipment can be immediately determined in order to dispatch response teams and aid in assessing the situation.

## II II COMMENT

Commentors note that the future selection of a national central repository would require the transport of spent nuclear fuel and that analyses of these shipments should be included in the EIS. Commentors state that the public has not been properly sensitized to the full transportation impacts.

### RESPONSE

Further shipments of SNF might be needed when a decision is made regarding ultimate permanent repository. Assessment of the impacts of these shipments is not included in the EIS. A method for ultimate disposition has not been selected and such analyses would be prepared in the final decision. The EIS describes the alternatives for SNF management until 2035. This amount of time may be required to make and implement a decision for ultimate disposition of SNF. DOE has a range of reasonable alternatives for safely managing SNF during the period 1995 to 2035. To inform the public concerning SNF transportation issues, this EIS evaluates the transportation alternatives for a reasonable range of alternatives. The alternatives vary from no action, involving transport of radioactive material, to centralization, which involves extensive transport of radioactive material. Analyses in the EIS show that the potential risks from transportation of SNF would be minimized for the alternatives considered. Minimizing transportation is one of the factors that has been considered in the DOE decision-making process.

## II COMMENT

The commentor questions why and how these waste products must be moved.

### RESPONSE

Transportation of SNF and radioactive wastes may be necessary to implement alternatives for effectively managing these materials during the period evaluated by the EIS. The alternatives are discussed in Volume 1, Chapters 1 and 2, and in Volume 2, Chapters 1 and 2. Most SNF would be transported by truck or train using shipping containers that satisfy the requirements of DOT and NRC. DOE follows DOT regulations for the shipment of radioactive materials, which include requirements for routing, external radiation limits, and the ability to withstand hypothetical accident conditions, including fire. A brief discussion of the alternatives is in Volume 1, Appendix I.

## II II COMMENT

The commentor notes that the increased shipments required to centralize spent nuclear fuel at the Test Site matter because of the low risk of transportation and the eventual need to

to Nevada for ultimate disposition at Yucca Mountain.

#### RESPONSE

The commentor is correct that for all alternatives, the potential risks from transp small. It is true that centralization at NTS could provide interim storage of SNF potential site of ultimate disposition. DOE has considered these, as well as other identification of a preferred alternative and the ROD. See also the response to co DOE's preferred alternative.

## II 5.13 Emergency Preparedness

### II COMMENT

Commentors state that DOE has not agreed to pay for monitoring, training, and equip responders at ports of entry and along shipping routes. One commentor states that DOE responsibility for training emergency responders to DOE.

#### RESPONSE

As a shipper of radioactive materials, DOE is responsible for complying with the re the safety of its shipments. This includes assisting state, tribal, and local emer accident occurs. DOE's Transportation Emergency Preparedness Program includes init and training, exercises, and technical assistance to state, tribal, and local gover DOE participates with other Federal, state, and local authorities to sponsor and fu response training courses throughout the United States. These courses are usually of local, state, and tribal authorities responsible for public safety and emergency disasters or man-made accidents. The funds for these training sessions come from F allocations of state tax dollars. Trainees provide their own transportation to the provide their own emergency response equipment; however, Federal assistance is prov times. The Federal Government has organized, trained, and equipped state and Feder response teams, which are quickly available to assist local authorities in the even

### II II COMMENT

The commentor wants to know the mechanics of dissemination of information to the pu occur at the Idaho National Engineering Laboratory.

#### RESPONSE

The DOE Idaho Operations Office maintains a Warning Communications Center (WCC) tha hours a day, 7 days a week. WCC personnel operate in four teams, with each team on time. Incident information is immediately passed to the WCC by INEL personnel and on the nature of the incident, different media are informed. Incidents such as car traffic are sent to local radio stations only. With radioactive materials releases however, information is immediately sent to not only local radio stations, but to a major state radio stations, newspapers, and public officials. Information is updat an incident, additional personnel are brought in to answer questions from public of the general public.

### II COMMENT

Commentors propose that DOE inform all those living within a 500-mile radius of nuc sites of the wastes generated and stored nearby and the significant danger these wa

#### RESPONSE

The action proposed by the commentors is being accomplished by the preparation and EIS and other site-specific EISs that will be prepared to assess the environmental radioactive waste management at DOE sites. SNF and radioactive waste management po be understood and minimized. This EIS evaluates these hazards and the engineered s management practices designed to reduce or eliminate the hazards. Sites have emerg equipment to respond to accidents and other emergencies. DOE requirements for emer preparedness are contained in DOE Orders 5500.1B, 2B, and 3A (Emergency Management Emergency Categories, Classes, and Notification and Reporting Requirements; and Pla

Preparedness for Operational Emergencies, respectively). DOE emergency notification based on the Emergency Response Planning Zone determined for each facility based on for the facilities. DOE notifies out to the distance required by the Emergency Res applicable state and local requirements.

## II COMMENT

The commentor points out that, in the event of an incident involving spent nuclear National Engineering Laboratory, large numbers of highly trained personnel are always the effect of any incidents.

### RESPONSE

The commentor is correct. INEL's highly trained work force includes a broad range and skills; this expertise, knowledge of plant systems and procedures, and training actions and priorities are key elements in the control of emergency situations and

## II COMMENT

The commentor questions whether statements related to the evacuation time for motor public highway to the Idaho National Engineering Laboratory are substantiated.

### RESPONSE

The commentor is referring to a statement in Volume 2 that a motorist at the nearest could be evacuated in 2 hours. In the event of an accident at an INEL facility that release to the environment, normal precautionary actions include establishment of road portions of public highways traversing the site. The road blocks prevent members of entering the affected area; site security personnel would also patrol the affected ensure no motorists remained after the road blocks were established. Evaluations of times indicate that these actions can be accomplished well within the 2-hour period Volume 2 accident analysis.

## II II COMMENT

The commentor suggests that switching from truck to train for transportation of spent result in inadequate emergency preparedness along the new routes.

### RESPONSE

The EIS addresses accidents in Volume 1, section 5.7.12. Locations considered for emergency action plans and equipment to respond to accidents and other emergencies. be updated to cover any new SNF facilities and activities. DOE would coordinate with local agencies to establish and implement an appropriate emergency response training potential accidents. The details of such planning are beyond the scope of the EIS.

## II COMMENT

The commentor states that the Shoshone-Bannock Tribes have limited emergency response lack any equipment in the event of an accident on the Fort Hall Reservation.

### RESPONSE

In the event of an accident involving a hazardous or radioactive material shipment Reservation, local fire and police organizations are first to respond. DOE, DOT, a training and training materials to local emergency responders to prepare them to handle. If the accident involves a release of hazardous or radioactive material, assistance from the State Hazardous Materials Team located 15 minutes away in Pocatello. DOE Radiological Assistance Program teams consisting of trained experts equipped and prepared to respond to a radiological accident and assist local emergency response personnel, if response team could respond to a request for assistance from the Tribes in much less on documented response times to other locations such as Dubois, Idaho, and the State. Although the accident analysis presented in the EIS takes no credit for emergency response impacts of the potential accidents would be small.

## II COMMENT

The commentors state that emergency response systems are not set up to respond to accidents involving spent nuclear fuel.

### RESPONSE

To date, accidents involving SNF have been rare, but they do occur; however, no significant accidents resulted from any of the accidents during SNF transportation. In the event of an accident, a shipment in transit, local fire and police organizations are first to respond. DOE provides training and training materials to local emergency responders to prepare them to handle accidents. DOE provides for Radiological Assistance Program teams, which consist of trained experts prepared to quickly respond to an accident and assist local emergency response personnel if requested. This response network, along with preventive measures, such as design and testing and adherence to stringent regulations, supports the continued safe

## II COMMENT

The commentor states that DOE needs to define a position regarding the funding of emergency response in states along spent nuclear fuel transportation corridors.

### RESPONSE

DOE recognizes the importance of preparedness for potential accidents involving transportation. Currently, training is available on a limited basis at the awareness level for first responders working with state and local officials through the Transportation External Coordination Program. DOE is developing a national approach for training and technical assistance.

## II II II COMMENT

Commentors question the adequacy of notification of civil agencies and inspection of radioactive materials. In particular, some commentors express concern about the inspection of research reactor spent nuclear fuel.

### RESPONSE

DOE complies with DOT regulations and, when applicable, the International Atomic Energy Agency regulations regarding notification and inspection of radioactive material shipments transporting material to ultimate destinations within the United States are also regulated by DOT regulations. Inspection of SNF before it reaches the United States is the responsibility of the person who must certify that the radioactive material is in proper condition for transport in compliance with external radiation and contamination requirements.

The Naval Nuclear Propulsion Program does not announce the times or routes of shipments more difficult for terrorists, saboteurs, or hijackers to plan and execute an attack. This is in accordance with Federal Government policy and regulations governing such shipments. Navy's policy on notification is also in full compliance with the applicable state regulations for such shipments containing highly enriched weapons-grade uranium.

## II COMMENT

The commentor requests that DOE consider Governor Campbell's request for assistance with North Carolina's emergency response capability because of the shipment of foreign research reactor fuel within the state.

### RESPONSE

DOE responded to former Governor Campbell's request by providing funds to assist with North Carolina's emergency response capability.

## II 5.14 Not used

## II 5.15 Socioeconomics

### II COMMENT

The commentor states that the negative public perception of spent nuclear fuel stor Ridge Reservation could lead to rejection by certain persons or businesses of the n suitable place to live or conduct business. That rejection would have a correspond impact on the community.

#### RESPONSE

Volume 1, Appendix F, Part Three, section 5.3 discusses the socioeconomic impacts o on the region of influence around ORR. Because the actual environmental impacts as management under all alternatives considered in the EIS would be small, there is no storage or examination of SNF at any of the locations evaluated would have any adve economy.

### II COMMENT

Commentors state they are concerned about the loss of spent nuclear fuel management the alternatives.

#### RESPONSE

Employment and job issues are discussed in Volume 1, Chapter 5 and site-specific Ap F, and in Volume 2, Chapters 4 and 5. These sections discuss direct and indirect j on the labor force of affected communities. The EIS Summary section Spent Nuclear Employment concludes that employment-related impacts would be small for all the alt considered.

### II COMMENT

Commentors suggest looking at clean energy sources and toward alternative jobs that RESPONSE

The development of clean energy sources and the associated new jobs and employment not within the scope of this EIS.

### II COMMENT

Commentors state that the EIS socioeconomic analysis should include effects on loca subsequent effects on the tax base, and the effects on the effort to diversify the RESPONSE

Because the environmental impacts associated with SNF management under all alternat small, there is no reason to believe that storage or examination of SNF at any loca have a discernible effect on local property values, as described where appropriate A through F, and Volume 2, section 5.3. Changes in the economic conditions under a considered would be small relative to the local economies of the potential sites an diversify local economies would be small.

### II COMMENT

The commentor notes that in addition to the four county school districts, there are Oak Ridge and Harriman, Tennessee.

#### RESPONSE

The average daily memberships for city school districts, such as Oak Ridge and Harr the total average daily membership presented for the four county school districts i Part Three, section 4.3.3.

## II COMMENT

The commentor states that the EIS should include a more detailed socioeconomic analysis of Clark Counties in Nevada, including consideration of the impact of this project in activities planned for the Nevada Test Site.

### RESPONSE

The EIS, Volume 1, Appendix F, Part Two, section 5.16, presents the potential cumulative impacts of the proposed SNF management facilities. The approach for analysis in Volume 1, Appendix section 5.3, is adequate for comparing alternatives in a programmatic EIS.

## II COMMENT

The commentor states that the environmental and health risks associated with nuclear power outweigh any economic benefit.

### RESPONSE

Volume 1, section 5.3 and Volume 2, section 5.15 of the EIS evaluate potential impacts on the public from both radiological and nonradiological hazards. The analyses show that impacts from alternatives would be small.

## II COMMENT

The commentor is concerned that the unique situation of the Shoshone-Bannock Tribes Hall Reservation is not discussed. The assumed migration rates fail to consider the impact on the Reservation, and greater household sizes on the Reservation must be addressed in socioeconomic analysis. The Reservation should be treated as a separate entity due to the high unemployment rate on the Reservation and because 70 percent of the food is acquired by hunting and gathering.

### RESPONSE

The purpose of this EIS is to analyze the potential impacts related to the alternative management changes in baseline conditions are addressed in general to support the impact analysis. There would be no significant impacts to the socioeconomic resources of the region of interest from changes in regional economic, transportation, health, accidents, or environmental conditions from the SNF management alternatives at the potential sites or environmental restoration program alternatives at the INEL. Therefore, it was not considered necessary to separately evaluate potential impacts to the Shoshone-Bannock Tribes or the Fort Hall Reservation. Impacts from implementation of any of the EIS alternatives are expected to be small.

With respect to INEL, employees represent less than 2 percent of employed persons in the Fort Hall Reservation (25 out of 1,544). Employment changes at INEL as a result of the project are expected to disproportionately affect the Tribes or the Reservation; therefore, separate analysis is performed.

The migration assumptions do account for a proportion of the population remaining in the reservation. If the commentor is concerned that residents of the Reservation would not be adequately reflected in the migration assumptions contained in the EIS, household size assumptions could be used to determine estimates of migrating population. Because it is unlikely that any alternative management changes would migrate, the difference in household size does not impact the population. Transportation and accident analyses do not indicate that Reservation lands would be impacted by no impact to agricultural production or hunting or gathering are expected. The reservation is not expected to be impacted.

## II COMMENT

The commentor observes that there is no discussion on the adequacy of public facilities in the region of influence around the Idaho National Engineering Laboratory.

### RESPONSE

Data regarding community resources are presented in Volume 2, section 4.3.3. The data

any remarkable excesses or deficiencies in levels of service; therefore, their adequacy was specifically evaluated. The data-collection process did not reveal outstanding problems.

## II COMMENT

The commentor disagrees with the use of current employment figures rather than more projections for the Idaho National Engineering Laboratory and states that the analysis did not adequately evaluate impacts and reasonably foreseeable actions.

### RESPONSE

The EIS has been revised to reflect current projections of employment, including contractor consolidation including program changes at Argonne National Laboratory-Washington. Cumulative employment impacts are presented in Volume 2, section 5.15. The cumulative figures include the effects of (1) baseline changes at INEL, (2) alternative impact (non-DOE) project impacts. The cumulative employment impacts are based on the best time of the analyses. The projected INEL employment figures are bounding for the alternatives. With the announced INEL employment reductions, employment estimates for any of the alternatives are easily accommodated within the existing site and region of influence. The Final EIS and ROD will be issued in 1995; therefore, fiscal year 1995 would be used for analyzing potential impacts that could result from implementation of the SNF management and waste management alternatives. The analysis in Volume 2, section 5.15 provides potential impacts under each alternative relative to conditions in 1995. However, data are provided beginning with fiscal year 1990 (Volume 2, section 4.4.3 and Figure 4.4.3). The reader may compare the projected impacts to employment levels during years prior to 1990 raised in the comment regards baseline employment only. The absolute impacts of the alternatives are the same regardless of which baseline year is chosen. It is the "relative" impact that the analysis conducted estimates the impacts of the alternatives, not of changes in baseline employment is not an alternative, and therefore, is not analyzed as such.

## II COMMENT

The commentor states that the socioeconomic analyses should have identified local impacts surrounding the Idaho National Engineering Laboratory and discussed the fiscal health of the alternatives on those specific areas.

### RESPONSE

Community resources were analyzed, and the results are presented in Volume 2, section 4.3-4. Existing economic, social, and community profiles for affected communities are provided. The potential socioeconomic impacts associated with the alternatives are so small that a detailed analysis of local jurisdictions is not needed. Most INEL employees live in Bonneville County (Idaho). Therefore, it could be expected that any potential impacts would be focused in that area.

## II COMMENT

The commentor states that the higher wage rate of Idaho National Engineering Laboratory compared to the average wage rate in the region of influence, was not considered in the analysis.

### RESPONSE

It is true that INEL jobs on average are higher paying than the average private-sector jobs. However, job losses (under the Ten-Year Plan and Minimum Treatment, Storage, and Disposal alternatives) and job gains (under the Ten-Year Plan and Maximum Treatment, Storage, and Disposal alternatives), as discussed in Volume 2, section 5.3, are not expected to be sufficient to offset impacts with or without wage differentials taken into account. Volume 2, section 5.15 was used to analyze impacts, including total employment and earnings impacts that were calculated using the Regional Input-Output Modeling System multipliers. As described in Volume 2, section 5.15, in the year 1990, INEL directly employed approximately 11,100 persons, while the population supported by INEL employment was estimated to be approximately 38,000 persons.

## II COMMENT

Commentors object to shipment and storage, and potential sabotage of nuclear waste Engineering Laboratory, because it would seriously affect the tourist industry and western Wyoming.

### RESPONSE

Because the actual environmental impacts associated with SNF management under all a considered in the EIS would be small, there is no reason to believe that storage or any of the locations evaluated would have any significant effect on tourism. Even hypothetical accidents are limited in extent and small enough that there should be

## II COMMENT

The commentor raises an issue about the lack of quantitative analysis of the socioe would result from a 1-year restriction of agricultural use of land surrounding the Engineering Laboratory that has been contaminated following an accident and release material.

### RESPONSE

The impacts have been addressed in Volume 2, section 5.14 in a qualitative manner. that the major part of the land that would be restricted following an accident at I there is a potential for existing agricultural land near INEL to become contaminate from use. More likely, however, is the possibility of a temporary restriction of l completion of surveys to ascertain whether contamination has occurred under allowab temporary restriction would be of short duration.

Although the economic value of any contaminated land is highly subjective, in the e incurred as a result of contamination and restriction of land use, persons injured their losses in accordance with applicable laws and regulations.

## II COMMENT

The commentor requests that the socioeconomic portion of the EIS address DOE's stra improve U.S. competitiveness in a world economy and to transfer technology from the sector. Specifically, the commentor asks what the impacts of each alternative are technology transfer.

### RESPONSE

DOE is in the process of identifying technologies for transfer from the public to t its facilities and has ongoing programs targeting improving U.S. competitiveness in The activities associated with SNF management use existing technologies and do not opportunities for technology transfer.

## II COMMENT

The commentor is of the opinion that managing spent nuclear fuel at the Savannah Ri projected employment declines, will impede economic development in the region and h impact on the quality of public education in Aiken County, South Carolina.

### RESPONSE

As noted in Volume 1, Appendix C, section 5.3, DOE believes that the projected decl SRS would be offset, in part, by the creation of operations jobs to support SNF man DOE does not anticipate any adverse impacts to the public education system under an alternatives being considered.

In terms of economic development in Aiken County and the region, DOE believes that development activities and opportunities that may accompany SNF management activiti economic development in the region.



## II COMMENT

The commentor notes the importance of maintaining the pool of experts.

### RESPONSE

The commentor is correct in noting the importance of maintaining a pool of expert p it is necessary to maintain the existing infrastructure and skilled resources neces as other nuclear materials and waste. One of the factors considered in identifying management was maximizing the use of existing expertise and overall SNF infrastru environment, safety, and health; waste management safeguards and security; and emer capabilities.

## II COMMENT

The commentor raises an issue about adverse employment impacts to the Shoshone-Bann asks whether DOE will mitigate those impacts.

### RESPONSE

Volume 1, section 5.7.2 states that DOE will minimize impacts by coordinating with planning agencies to address impacts on community services, housing, infrastructure transportation.

## II COMMENT

The commentor states that the number used for the population located within 50 mile Site is too low and that workers from the Nevada Test Site are not considered in th  
RESPONSE

Volume 1, Appendix F, Part Two, section 5.7 states that a population of 15,100 pers be within 50 miles of the proposed SNF facilities at NTS in 1995. This population 1990 census data extrapolated to 1995 using county growth rates. Volume 1, Append section 4.3, considers Nye and Clark counties, where most of the NTS work force res

## II COMMENT

The commentor states that DOE needs to make firm commitments to mitigate adverse em that could occur, ranging from retraining displaced workers to providing support fo  
RESPONSE

As stated in Volume 1, Chapter 5, DOE will coordinate its planning efforts with loc county planning agencies to address impacts on community services, housing, infrast transportation, and employment. In the past, DOE has worked to retrain and refocus changes in mission, such as the transition from past emphasis on defense-related ac War to current environmental restoration activities. Also, as in the case of the C working with community leaders to help diversify the economic base away from a larg DOE activities at INEL.

## II 5.16 Safeguards and Security

## II COMMENT

The commentor states that this EIS addresses nothing new in establishing a viable w moving nuclear wastes around only delays the problem to the next generation.

### RESPONSE

DOE is committed not only to developing Federal geologic repositories for permanent but to providing safe interim storage pending availability of permanent disposal fa SNF is necessary to varying degrees under the alternatives DOE is analyzing for pro

storage and management of SNF. The alternatives have definite purposes for relocating and storing similar fuel types within a single secure facility. Thus, the alternatives transportation concerns with other worthy considerations, including nonproliferation and cost effectiveness.

The potential impacts from storing radioactive materials associated with SNF are discussed in Chapter 5. Environmental consequences of SNF management for all alternatives are discussed in Volume 1, section 5.1, and mitigation measures are discussed in Volume 1, section 5.7. DOE safety managing and storing SNF and other radioactive materials at each of the site. It is DOE policy to design, construct, and operate its facilities in a way that provides safety assurance that meets applicable Federal, state, local, and DOE requirements and regulations. DOE will manage SNF in accordance with applicable Federal and DOE requirements and regulations in a manner that ensures protection of the environment, health and safety of the public and site employees.

## II COMMENT

The commentor states that there should be "a lot more" security associated with the alternatives described in Volume 1, and these alternatives should all be comparable with the Centralization alternative.

### RESPONSE

DOE has security systems in place at all facilities that handle nuclear materials. Security systems established for the various alternatives would be appropriate for the alternatives. Security precautions are routine for all shipments of DOE nuclear material. Security measures more than 40 years, resulted in no known theft of DOE nuclear materials. See also the response to comment 05.16 (001).

## II COMMENT

Commentors request declassification of environmental, safety, and health documentation establishing historical Idaho National Engineering Laboratory source terms (radioactive) unavailability of this previously classified documentation has prevented an accurate assessment of impacts.

### RESPONSE

This comment relates to DOE's dose reconstruction project, which is outside the scope of the U.S. Department of Health and Human Services (HHS) and DOE have two Memoranda of Understanding (MOUs) for public health responsibilities around DOE sites. Under the MOUs signed in December 1990, DOE transferred the responsibility for managing and conducting analytic epidemiologic research to HHS. HHS has delegated responsibility to the Centers for Disease Control and Prevention (CDC). Baseline health effects studies for both DOE workers and the surrounding public are either under way or planned at all facilities. To support the dose reconstruction project, DOE is responding to all CDC requests for declassification of materials be released. DOE is responding to all CDC requests for declassification of the dose reconstruction project. All studies will be made available to the public community. For more information on this matter, contact the DOE Office of Public Affairs. In recent years, DOE has released significant amounts of previously classified data and release additional information as it becomes declassified. Although most environmental data are not classified, other data on DOE activities are very sensitive and will remain classified by the Secretary of Energy.

## II COMMENT

The commentor asks about the consequences of terrorist attacks, and states that storage facilities should be where the least damage could occur.

### RESPONSE

The EIS evaluates 10 sites as reasonable alternatives for some level of SNF management. The analysis in the EIS includes a number of factors including the potential risks to the operations and reasonably foreseeable accident conditions. Discussions on public health are found in the Occupational Public Health and Safety sections in Volume 1 (and its appendices).

Appendices A through F), and in the Health and Safety section in Volume 2. The EIS would be no significant risks to the public or the environment due to SNF management the 10 sites being considered.

The consequences of postulated terrorist acts are expected to be bounded by the res analysis. SNF is not attractive to terrorists due to the bulk of the fuel and tran the high radiation fields surrounding unshielded SNF.

DOE and the Navy have extensive security systems at all facilities handling nuclear precautions are routine for all shipments of government-owned nuclear material. Fo security precautions have successfully prevented the theft of government-owned nucl

## II COMMENT

The commentor is opposed to alternatives that centralize spent nuclear fuel at a si attack on a nuclear fuel storage facility could release large quantities of radioac cause significant loss of human life.

### RESPONSE

DOE has extensive security systems in place at all facilities that handle nuclear m precautions, including emergency response team notification, are routine for all sh material. Even in the event of a successful attack on a DOE nuclear facility, the in the EIS, which bound any credible terrorist attack scenario, describe consequenc "the extinction of mankind" mentioned by the commentor. However, scenarios involvi weapons are outside the scope of this EIS. Volume 2, section 5.14 has been changed terrorism as an initiating event.

## II COMMENT

The commentor is opposed to nuclear power because of the concern about nuclear mate "the wrong hands."

### RESPONSE

DOE has extensive security systems in place at all facilities that handle nuclear m precautions, including emergency response team notification, are routine for all sh material. Security precautions have, for more than 40 years, successfully prevente nuclear materials. Questions and concerns regarding nuclear nonproliferation are o EIS. However, Volume 1, sections 1.2.3 and 1.2.4 refer the reader to other DOE-spo reviews. Nuclear nonproliferation policies will be addressed in two future DOE pub Proposed Nuclear Weapons Nonproliferation Policy Concerning Foreign Research Reacto Nuclear Fuel and Programmatic EIS for the Management and Disposition of Excess Nu (Draft).

## II 5.17 Monitoring

## II COMMENT

The commentor states that adequate funds must be available to support environmental at Idaho National Engineering Laboratory.

### RESPONSE

INEL has adequate funds to support environmental monitoring activities per DOE Orde Environmental Protection Program, which implements the established environmental pr at INEL.

## II COMMENT

The commentor states that the EIS does not evaluate the potential need for addition monitoring of new Idaho National Engineering Laboratory facilities described under  
RESPONSE

The purpose of the EIS is to evaluate the potential environmental impacts from prop  
EIS is not intended to substitute for the assessments required by regulations. Any  
requirements for monitoring emissions from facilities and surveillance of the surro

## II COMMENT

The commentor has requested documentation of the results of the environmental monit  
particularly those of the Environmental Protection Agency, in the Volume 1 site des  
RESPONSE

DOE has added references to the environmental monitoring results at the various sit  
1, Chapter 4.

## II COMMENT

The commentor requests that the EIS contain a detailed monitoring plan for the pref  
describe the feedback mechanisms by which the monitoring results are used to modify  
based on changing information.

RESPONSE

The Idaho National Engineering Laboratory Environmental Monitoring Plan has been pr  
reference for the EIS. For existing facilities, it is independent of the alternati  
new facilities, more specific information, such as specific locations and facility  
required before an appropriate monitoring plan could be prepared. The facility-sp  
would be prepared after final issuance of an EIS. DOE believes that inclusion of a  
plan in this EIS would not provide useful information to decisionmakers, because it  
discriminator for comparison of the alternatives.

## II 5.18 General Operations

## II COMMENT

The commentor questions what techniques are being developed to ensure safe, long-te  
waste, and that this is not dangerous material and ways of safely storing it really  
RESPONSE

Numerous technologies are already available for managing radioactive materials, and  
considered for this purpose. Technological options for SNF management are describe  
section 1.1.3 and Appendix J. Current management practices for all types of radioa  
discussed in Volume 2, section 2.2.7, and technology development activities are des  
section 3.1. (Volume 2 is specific to INEL, but waste management technologies also  
other DOE sites.) DOE has established a policy of compliance with all applicable F  
regulations and DOE Orders. All radioactive materials will be managed to protect t  
health and safety of the public and site employees.

## II COMMENT

The commentor believes that technologies for safe, long-term storage of nuclear was  
not exist because the material being stored has a long half-life and will outlast t  
RESPONSE

DOE has a program to safely manage and store radioactive materials (including both  
SNF) at each of the sites considered in the EIS. The potential impacts of storing  
mitigation measures are discussed in Volume 1, Chapter 5. Supporting information o  
storage options for them is provided in Volume 1, Appendix J. Management and stora  
materials at INEL are described in Volume 2, Chapters 1 and 2. It is DOE's policy  
applicable Federal, state, and local regulations and DOE Orders. All radioactive m  
to ensure protection of the environment and the health and safety of the public and  
One of the concerns that must be addressed prior to ultimate disposition is the con

commentor that the waste may outlast some storage methods. While ultimate disposition scope of this EIS, DOE is researching and developing disposition technologies that the longevity of the waste and ensure that the public and environment are protected. General long-term solutions proposed for managing SNF at INEL are discussed in Volume 1 and 2. The alternatives for safe SNF management in the interim are discussed in Volume 1.

## II COMMENT

Commentors raise an issue about the disposing of hazardous and radioactive wastes using unacceptable methods.

### RESPONSE

DOE accepts the responsibility to operate its hazardous and radioactive waste management in compliance with applicable requirements. DOE continues to improve the procedures associated with waste management. Accordingly, the lessons learned from past waste management practices and the knowledge being gained from current research and development programs are incorporated into future waste management programs. One purpose of this EIS is to achieve these objectives.

Volume 1 is intended to provide the public and decisionmakers with a programmatic view of actions and alternatives for managing SNF. For all alternatives analyzed, DOE is considering applicable Federal, state, local, and DOE requirements to ensure that SNF is safely managed in the environment and health and safety of the public and site employees are protected. For each alternative, only the minimum actions necessary for continued safe management of SNF are implemented.

Volume 2 is a site-specific assessment of SNF and environmental restoration and was developed for alternatives at INEL. Again, the intent of Volume 2 is to provide the public and decisionmakers with information necessary to select the best alternative for these activities at INEL. The programmatic EIS for waste management, which will provide a broader view of complete waste management programs similar to the way Volume 1 of this EIS addresses the programmatic management of SNF.

## II COMMENT

The commentor states that for Volume 1, high-level, transuranic, and mixed waste are treated with different risks, and should be dealt with separately in the EIS. The commentor also questions the definition of mixed waste.

### RESPONSE

DOE agrees with the comment that these three wastes are of different types, with different disposal requirements. While it would be necessary in a site-specific EIS to treat separate entities, for this programmatic EIS, they were lumped together (and separated) for two reasons: (1) the volumes of high-level, transuranic, and mixed waste generated from SNF management under the No Action alternative are uniformly small compared to the volumes of these wastes already at DOE sites, and (2) high-level, transuranic, and mixed waste will eventually be disposed of offsite, whereas low-level wastes can be disposed of on-site. Mixed waste has been added to Appendix H.

## II COMMENT

The commentor indicates that Figure 5-2 and the text on page 5-25 do not agree.

### RESPONSE

The text in Volume 1, section 5.1.3.3 indicates that the Hanford Site would generate a certain amount of high-level, transuranic, and mixed waste due to processing. Volume 1, Figure 5-2 shows the volumes of waste that would be generated from the Decentralization alternative.

## II COMMENT

The commentor has questions about safe temporary storage and ultimate disposal of r  
RESPONSE

DOE has a program to safely manage and store radioactive materials, including SNF, considered in this EIS. It is DOE's policy to design, construct, and operate its f provides a level of safety and safety assurance that is in accordance with applicab local regulations and DOE Orders. DOE will manage radioactive materials and wastes ensures protection of the environment and the health and safety of the public and s Management and disposal of radioactive wastes are discussed in Volume 2, Chapter 1. management practices for each type of radioactive waste (which are improvements on given in Volume 2, section 2.2.7. The potential impacts of storing radioactive mat SNF are discussed in Volume 1, Chapter 5. Specific supporting information on types options for them are presented in Volume 1, Appendix J.

## II COMMENT

The commentor asks about three waste treatment facilities under development by the Group, Inc. at the Oak Ridge Reservation site.

RESPONSE

Scientific Ecology Group, Inc., has three commercial waste treatment facilities und are not located at ORR. It has recently completed construction of a Carlsbad, New recently purchased property for a Hanford, Washington, site; and is in the planning Falls, Idaho, site. As stated in Volume 1, Appendix F-4, Scientific Ecology Group, considered, as stated in Volume 1, Appendix F, Part Three, section 5.16. While som this facility will be made, it will remain within the property boundaries of the si incremental impacts from the addition of a second radioactive incinerator are asses manner in the EIS.

## II COMMENT

The commentor questions the meaning of off-site disposal as a waste management acti Test Site.

RESPONSE

Off-site disposal in the context of Volume 1, section 4.4 means disposal off of the DOE facility or permitted and licensed commercial disposal facility. The destinati depend on the type of waste. The text in the Final EIS has been changed to clarify manage wastes offsite.

## II COMMENT

Commentors want all wastes disposed of in Resource Conservation and Recovery Act-pe waste and/or Environmental Protection Agency/Nuclear Regulatory Commission-permitte waste disposal sites as appropriate.

RESPONSE

DOE waste management policies and practices embrace numerous laws and regulations g hazardous and radioactive wastes. A comprehensive list of these requirements is pr Chapter 7, and associated environmental permits are also discussed there. Current for radioactive and nonradioactive wastes are described in Volume 2, section 2.2.7, INEL, but also generally applies to wastes at other DOE sites. DOE has established with all applicable Federal, state, and local regulations and DOE Orders, including establishing disposal requirements, including RCRA disposal of wastes in hazardous appropriate, EPA/NRC-permitted radioactive waste disposal sites. All radioactive materials will be managed to protect the environment and the health and safety of t employees.

## II COMMENT

The commentor states that several types of low-level radioactive waste should be co  
RESPONSE

DOE radioactive wastes are specifically managed according to DOE Order 5820.2A, Rad Management, which classifies radioactive wastes somewhat differently than regulation NRC for commercial radioactive wastes. In particular, DOE has only one category for which encompasses the A, B, C, and greater-than-Class-C distinctions made by NRC. management measures may still be prescribed for DOE low-level wastes according to the of radionuclides present, analogous to standards for disposal of commercial radioactive example, DOE low-level waste analogous to NRC greater-than-Class-C waste is required 5820.2A, Radioactive Waste Management, to be handled as a special case, and is not buried in the RWMC. Additional information on special-case waste at INEL has been section 3.1.3.

## II COMMENT

The commentor urges that until we can eliminate the generation of nuclear waste, keep it and monitor it, and people have an interest in seeing that the generation is even substantially curtailed.

RESPONSE

Under the No Action alternative, DOE would limit actions to the minimum necessary for management of SNF at the generation sites or current storage locations. Most DOE SNF production and experimental reactors that have ceased to operate, so considerable has already occurred. SNF management plans are presented for all alternatives in Volume 2 and mitigation measures are discussed in section 5.7.

## II COMMENT

The commentor expresses an opinion that all waste should be stored in a retrievable technologies available.

RESPONSE

Descriptions of how wastes would be managed under the proposed alternative actions and 2, section 3.1. These alternative actions also consider the best technologies development activities, including stabilization technologies, aimed at advancing technologies available for waste management are described in Volume 2, section 3.1.

## II COMMENT

The commentor wants to know if the statement on Volume 1, page 5-72 stating "but will approximately 2 cubic meters per year (3 cubic meters per year) of high-level waste process or a reprocessing activity at the Savannah River Site."

RESPONSE

The statement refers to "processing," as shown in Volume 1, Appendix C, section 3.1

## II COMMENT

The commentor suggests a wording change in Volume 1, Appendix A, section 2.3 to better characteristics of the Hanford Spent Nuclear Fuel Management Plan.

RESPONSE

The suggested wording change has been incorporated into the EIS.

## II COMMENT

The commentor states that the EIS should reconsider the procedures for burial at the Engineering Laboratory Radioactive Waste Management Complex of the material removed fuel modules during examination at the Expanded Core Facility, and that the EIS does changes to this procedure.

## RESPONSE

The Navy and DOE rely on definitions and classifications of nuclear materials set forth in the Waste Policy Act, as amended, and regulations issued by EPA (40 CFR 261) and NRC (10 CFR 20.2001). Categories set forth in these regulations are "Spent Nuclear Fuel," "High-Level Waste," "Low-Level Waste," "Low-Level Mixed Waste," "Greater-than-Class-C Waste," and "Decommissioning Waste."

Volume 1, Appendix H sets forth the definition of SNF used in this EIS as "fuel that has been removed from a nuclear reactor following irradiation, the constituent elements of which have been separated from the fuel. The definition of high-level waste in Volume 1, Appendix H is "highly radioactive waste that results from the reprocessing of spent nuclear fuel, including liquid waste and solid waste produced from the reprocessing of spent nuclear fuel, including liquid waste produced from the reprocessing of spent nuclear fuel..." Transuranic waste is defined as "waste containing alpha-emitting transuranic isotopes, with half-lives greater than 20 years..." Low-level waste is defined as "waste that contains radioactivity and is not high-level waste, transuranic waste, or spent nuclear fuel."

The ends removed from Naval SNF modules at the Expanded Core Facility are structural and provide support and direct the flow of cooling water during operation. The material ends of the fuel modules does not contain any fuel or fission products from fuel and are considered SNF. It does not contain transuranic elements or fission products and is not considered high-level waste or transuranic waste. The amounts of radioactivity in the material ends are not sufficient to be classified as low-level waste. Consequently, the material removed from the ends of the fuel modules at the Expanded Core Facility is categorized as low-level waste due to the amount of radioactivity. Their disposal at the RWMC at INEL is accomplished in accordance with applicable regulations. As indicated in Volume 1, Appendix D, section 5.2.15, the amount of low-level waste generated from the Expanded Core Facility is 425 cubic meters. The radioactive isotopes, which represent the activity in the material removed from the ends of fuel modules, are identified in Table 5.2.15-1.

ISOTOPE	HALF LIFE
Fe55	2.73 years
Co60	5.271 years
Ni59	76,000 years
Ni63	100 years

A description of the composition of material removed from the ends of fuel modules has been added to Volume 1, Appendix D, Attachment B.

## II II COMMENT

The commentor states that he was unaware that spent fuel storage generates transurans concerned that this may be due to extensive fuel leakage.

## RESPONSE

As reported in Volume 1, section 5.1.1 and site-specific Appendices A through F, tr generated in small quantities by the routine operations associated with transporting managing SNF (from filters, ion exchange columns, etc., particularly during examination activities) rather than extensive leakage.

## II II II COMMENT

The commentor points out that the vulnerability assessment states that canned fuel transferred to ICPP-666 could lead to contamination and additional vulnerabilities, address this issue.

## RESPONSE

DOE is aware of the potential for contamination if transfers are not conducted in a manner. All fuels to be transferred from ICPP-603 at the Idaho Chemical Processing Plant have been inspected for corrosion and other potential breaches. Potentially breached fuel elements will be placed in suitable containers to prevent release of radioactive material. Fuel elements are stored in shielded transfer casks. ICPP-666 has extensive monitoring and water purification systems. If a leaking container or fuel element would be identified and necessary corrective action taken.



additional vulnerabilities are anticipated.

The EIS discusses the Spent Fuel Working Group Report on Inventory and Storage of Spent Nuclear Fuel and Other Irradiated Nuclear Materials and Their Environmental, Health Vulnerabilities (known as the vulnerability assessment) and associated actions identified vulnerabilities in Volume 1, section 1.1.2 and Appendix I-2 and in Volume 2, section 1.1.2 and Appendix I-2.

## II COMMENT

Many commentors raise issues about DOE's past record of waste-handling practices at Hanford, Oak Ridge, and Idaho National Engineering Laboratory, resulting in release of radioactive materials.

DOE has identified, or is currently evaluating many of the problems that exist with infrastructure, or that have resulted from past releases of contaminants to the environment. Management strategies are continually evolving to meet current regulatory requirements of technology advancements. Many facilities across the DOE complex are either under construction or planning upgrades or replacements to come into compliance with applicable regulatory requirements. Contaminant releases are addressed by DOE's Environmental Restoration Program. Each on EPA's National Priorities List must negotiate an agreement with the appropriate state to prioritize work and develop enforceable schedules for cleanup of contaminated areas. INEL's FFA/CO, which is signed by DOE, EPA Region X, and the State of Idaho. As discussed in Volume 1, Chapter 1, DOE is committed to complying with all applicable state laws and regulations, DOE Orders, and interagency agreements governing SNF management and restoration and waste management.

As discussed in Volume 1, Chapter 3, safe management of SNF requires that many factors including site security, presence of skilled workers, safety, and the affected environment be considered in reaching a decision regarding in which state or states SNF will be stored. Analysis of potential storage locations were included in the EIS. As part of the public comment process, public input regarding the eventual location of SNF storage facilities was sought. The public input was part of the process used in arriving at the preferred alternative. The EIS, as well as other factors, will be considered in the ROD for the proposed action. Volume 1, section 5.1.1 summarizes potential impacts from waste management activities. The SNF management alternatives. Site-specific details are discussed in Volume 1, Hanford Site, Volume 1, Appendix F, Part Three for ORR, and Volume 1, Appendix B for

## II II COMMENT

The commentor wants mitigations measured for their effectiveness and addressed in the discussion of proposed mitigation for direct, indirect, and cumulative impacts should be included in the Council on Environmental Quality regulation states that an EIS should include the measures to avoid, minimize, and compensate for adverse environmental effects.

### RESPONSE

As discussed in Volume 1, Chapter 5, the EIS evaluated impacts to socioeconomic, environmental, waste management, occupational health and safety, public health and safety, and transportation. In all cases, the results indicate that impacts to the environment and to humans would be less than or comparable to the impacts of the proposed action. This level of detail is consistent with the programmatic EIS. Follow-on-site-specific NEPA analyses would address specific mitigation measures for identified impacts. Comparison of specific impacts by alternatives is provided in Table 3.3-1, with an indication of proposed mitigation measures. Possible mitigation measures are further discussed in Volume 2, Chapter 5. Specific mitigation measures to be under consideration for the ROD, and if necessary, a formal mitigation action plan will be developed for the ROD.

## II 5.19 Miscellaneous

## II COMMENT

Several commentors state preferences for truck, rail, barge, or air as modes of transportation. Reasons were provided for favoring one mode of transportation over another.

**RESPONSE**

The EIS evaluates truck, rail, barge, and ship transportation because they are believed in terms of risk and cost. Other modes of transportation were not evaluated. Truck transport of radioactive material is a legal and viable option and the potential transportation are very small. Rail transport of radioactive material is also a legal EIS evaluates both truck and rail transportation for DOE shipments. Navy SNF has been barge rail, except for transportation by ship from Pearl Harbor Naval Shipyard to Puget Sound where the containers are transferred to railcars and heavy-lift transporters move to the nearest rail access at the Kesselring Site. Transport of SNF or radioactive waste under any alternative being considered in this EIS. An analysis of barge transport analysis has been added to the EIS.

**II COMMENT**

The commentor identifies errors or omissions in the text and suggests alternative wording meaning of the text.

**RESPONSE**

The errors or omissions identified by the commentor have been corrected in the Final EIS.

**II COMMENT**

The commentor expresses support for DOE ecological activities and research at the Idaho Engineering Laboratory, which are not specific to this EIS.

**RESPONSE**

The comment is noted.

**II COMMENT**

Commentors express fear of moral impacts and obligations, catastrophic events, radiological materials, and emotional concerns over the management of nuclear material such as spent nuclear fuel.

DOE has attempted in this EIS to develop reasonably foreseeable, quantifiable environmental impacts to the proposed action(s), including operations and accident consequences. Other potential impacts as moral, emotional, and psychological (including fear, dread, mental anguish, negative attitudes, etc.) issues are beyond the scope of required NEPA evaluations. The U.S. Supreme Court in *Metropolitan Edison v. People Against Nuclear Energy*, 103 S. Ct. 1556 (1983), clarified the aforementioned NEPA evaluative requirements.

**II COMMENT**

Many commentors state they are concerned about errors and inconsistent use of information in the document, while others express concern about misleading discussions that need to be corrected.

**RESPONSE**

The EIS has been reviewed for errors and inconsistencies, including those identified by commentors. Changes have been made to the EIS to correct errors or clarify misleading information.

**II COMMENT**

Commentors express reservation and/or discontent about residing near nuclear waste management facilities.

**RESPONSE**

DOE is aware of general public fears regarding radiation and radioactivity, which arise from a basic unfamiliarity with such risks. The EIS analyzes the cumulative impacts of operations at the 10 candidate sites for management activities involving SNF. The EIS concludes that there would be no significant risk due to either operations or credible accidents involving SNF.

SNF, including transportation, at any of the candidate sites.

## II COMMENT

The commentor questions the existence or effectiveness of quality assurance or quality control at its facilities.

### RESPONSE

DOE and its contractors implement quality assurance/quality control requirements for all DOE and facility operations. Formal quality program requirements are derived and implemented in DOE Order 5700.6C, Quality Assurance, which defines the interrelations of criteria and procedures for managing, achieving, and assessing quality that result in improved safety and reliability of products and services. In accordance with these requirements, approved quality programs are implemented at the project/program level. These quality programs are tailored to meet the specific needs of the projects/programs and apply the appropriate industry standard criteria unique to each project, such as NQA-1 for nuclear reactor operations, EPA environmental quality assurance management system for remediation activities, etc. In recent years, DOE has adopted the Total Quality Management approach whereby employees at all levels are encouraged to take ownership in applying quality management aspects of their respective duties and interactions, resulting in more immediate and effective quality control.

## II COMMENT

The commentor asks why the value for the State of Idaho appears to be omitted from Table 5.15-1 of the EIS.

### RESPONSE

This error has been corrected.

## II COMMENT

Commentors raise the issue of the potential impacts to the environment and the people of Idaho.

### RESPONSE

Descriptions of the existing environment at INEL and the potential impacts to the environment from implementation of the alternative actions are in Volumes 1 and 2, Chapters 4 and 5. Chapters 4 and 5 discuss the current environmental situation and the expected consequences of the alternative actions on the environment and show that the impacts would be small for measures that DOE could implement to control or reduce impacts to the environment. As described in Volume 1, section 5.7 and Volume 2, section 5.19. As described in these sections, DOE is operating its facilities in compliance with all applicable laws and regulations protecting natural resources to ensure that the impacts of DOE activities on those resources are small.

## II COMMENT

The commentor notes that the EIS identifies irreversible and irretrievable commitments of resources likely to occur due to the proposed action and notes "the assertion that the EIS states that surface and groundwater resources already have been irretrievably impacted." The commentor states that DOE has an obligation to protect natural resources under its jurisdiction and to reevaluate the impacts of the proposed action on those resources.

### RESPONSE

The identification of irreversible and irretrievable commitments of resources is a key element of the EIS. Irreversible and irretrievable commitment of resources refers to the process of committing resources unavailable for use as a result of past, present, or proposed actions. Irreversible commitment of resources does not imply adverse environmental impacts. The discussion of impacts in Volume 2, section 5.15 shows that the impacts from past, present, and proposed actions would be small.

## II COMMENT

The commentor suggests specific deletions, corrections, or additions to the EIS.

### RESPONSE

If the suggested change was considered editorial or significant to the decision-making, appropriate change has been incorporated into the EIS.

## II COMMENT

The commentor states that a discussion of Oak Ridge spent fuel inventories in Volume 1 incorrectly refers the reader to a section that does not exist.

### RESPONSE

Volume 1, Appendix F, Part Three, section 2.3.7 has been modified to correct this error.

## II COMMENT

The commentor expresses the opinion that all facets of DOE's nuclear program are for the protection of bureaucrats.

### RESPONSE

This EIS addresses the programmatic management of SNF in the interim to ultimate disposition, environmental restoration and waste management activities at INEL over the next 10 years. Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environmental impacts of all alternatives considered in this EIS. The analyses show that the impacts of all alternatives are acceptable. Although vulnerabilities exist, DOE has the management skill, scientific capability, and mandate to safely manage SNF and INEL waste management and environmental restoration during the period covered by this EIS. See also the response to comment 03.07 (004).

## II COMMENT

The commentor states that a description of the amount of radiation expected to be released from this project is a necessary item in the EIS.

### RESPONSE

This information is provided for all alternatives and all sites considered in the EIS. Chapter 5 summarizes information on potential releases to the environment. Additional information is provided in Volume 1, Appendices A through D and K, and Volume 2, Appendix F.

## 05.19 (017) Miscellaneous

### COMMENT

The commentor identifies sections of Volume 2 of the EIS that require clarification and requests additional information to more completely address the material in appropriate sections.

### RESPONSE

The EIS has been modified to include the additional information requested by the commentor in Chapter 4.

## 05.19 (018) Miscellaneous

### COMMENT

The commentor requests a specific change to the EIS.

### RESPONSE

The commentor's suggested language has been incorporated in Volume 1, section 5.1.1.

**05.19 (019) Miscellaneous****COMMENT**

The commentor is unclear what the term "estimated population dose" means and states Volume 1 refers to Figure 5-1 as representing the estimated population dose, but th contain that term.

**RESPONSE**

The statement should have referred to estimated annual latent cancer fatalities. T the commentor has been revised in the EIS.

**VOLUME III Part B**

Department of Energy Programmatic  
Spent Nuclear Fuel Management  
and  
Idaho National Engineering Laboratory  
Environmental Restoration and  
Waste Management Programs  
Final Environmental Impact Statement  
Volume 3  
Part B  
April 1995  
U.S. Department of Energy  
Office of Environmental Management  
Idaho Operations Office

**VOLUME 3, PART B: CONTENTS**

SEE PART A FOR INTRODUCTION, ACRONYMS, AND CHAPTERS 1 THROUGH 5.

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## 6. SPENT NUCLEAR FUEL MANAGEMENT SPECIFIC

### 6.1 Vulnerabilities

#### 06.01 (002) Vulnerabilities

##### COMMENT

Commentors express the opinion that spent nuclear fuel is currently stored in poor DOE will not take responsibility for the waste currently existing. Commentors also trust DOE to manage additional spent nuclear fuel any better in the future, because learned from the last 40 years of spent nuclear fuel management.

##### RESPONSE

DOE acknowledges its responsibility to safely manage spent nuclear fuel (SNF). The has publicly affirmed that current DOE policy and practice emphasize safety and env considerations above other program goals. DOE is formally committed to protecting of its workers, the public, and the environment. Furthermore, DOE intends to desig operate facilities in a safe manner, relying on lessons learned from the last 40 ye DOE is working to rectify and eliminate any adverse environmental impacts from past Problems at existing storage facilities have been identified in the Spent Fuel Work Inventory and Storage of the Department's Spent Nuclear Fuel and Other Reactor Irra Materials and Their Environmental, Safety and Health Vulnerabilities. This report, nuclear fuel vulnerability assessment, and associated action plans to resolve ident acknowledged in Volume 1, section 1.1.2 and Appendix J-2, and Volume 2, section 2.2 site-specific information is in Volume 1, Appendices A through F. Environmental co management are presented for all alternatives in Volume 1, section 5.1, and mitigat discussed in section 5.7. For all alternatives analyzed, the impacts of SNF manage small.

#### 06.01 (005) Vulnerabilities

##### COMMENT

The commentor states that the EIS fails to acknowledge current problems with spent storage, and that these problems will continue to be ignored if DOE begins its mass concentration program.

##### RESPONSE

Problems at existing storage facilities have been identified in the Spent Fuel Work Inventory and Storage of the Department's Spent Nuclear Fuel and Other Reactor Irra Materials and Their Environmental, Safety and Health Vulnerabilities. This report, to as the spent nuclear fuel vulnerability assessment, and associated action plans vulnerabilities are acknowledged in Volume 1, sections 1.1.2 and J-2, and Volume 2, Additional site-specific information is in Volume 1, Appendices A through F. Envir consequences of SNF management are presented for all alternatives in Volume 1, sect mitigation measures are discussed in section 5.7. For all alternatives analyzed, D complying with applicable Federal, state, and local regulations and DOE Orders to e environment and the health and safety of the public and site employees. See also th 03.05 (024).

#### 06.01 (006) Vulnerabilities

##### COMMENT

The commentor raises two issues: (1) the ability of the K-basins to withstand a cr the chemical breakdown of the spent nuclear fuel in the basins.

##### RESPONSE

The continued management, storage, and chemistry of spent nuclear fuel currently at Site will be evaluated in the Hanford Spent Nuclear Fuel Management EIS (tiered from the EIS for the Management of Spent Nuclear Fuel from the K-Basins at the Hanford Site, Washington). The EISs will consider both the seismic conditions and the chemistry of fuel.

## 06.01 (008) Vulnerabilities

### COMMENT

The commentor states that existing storage conditions for N-Reactor fuels in the Hanford must be corrected immediately because of degradation from corrosion and hydriding.

### RESPONSE

Descriptions of SNF stored at the Hanford Site and technologies for managing SNF are given in Volume 1, section 1.1. Hanford-specific information on N-Reactor fuel and conditions given in Volume 1, Appendix A, section 2.3. More general information on management of degraded SNF and associated management technologies is in Volume 1, Appendix J, section 2.3. DOE agrees that it is necessary to deal with spent N-Reactor fuel, especially in the situation that was prominently identified in the Spent Fuel Working Group Report on Inventory of the Department's Spent Nuclear Fuel and Other Reactor Irradiated Nuclear Material Environmental, Safety and Health Vulnerabilities (called the spent nuclear fuel vulnerability assessment). These concerns also are reflected in a June 1994 Tri-Party Agreement on the encapsulation of uncontainerized fuel. This agreement between the State of Washington, Environmental Protection Agency (EPA), and DOE has a target date of December 2002 for SNF and sludge from the 105-K basins. See also the response to comment 06.01 (006).

## 06.01 (009) Vulnerabilities

### COMMENT

The commentor requests that the EIS address cleanup of Idaho National Engineering Laboratory contamination and the safety of existing spent nuclear fuel storage facilities, particularly at the INEL.

### RESPONSE

The evaluation in Volume 2 of this EIS bounds environmental impacts from environmental cleanup activities at the Idaho National Engineering Laboratory (INEL). However, cleanup activities at INEL are generally addressed under an enforceable agreement with the Environmental Protection Agency (EPA) Region X, and the State of Idaho on December 1994 Federal Facility Agreement and Consent Order (FFA/CO). The FFA/CO establishes a cooperative process that integrates the remediation requirements of Comprehensive Environmental Response, Compensation, and Liability Act (CERCLA), the corrective action requirements of the Resource Conservation and Recovery Act (RCRA), and the State of Idaho's Hazardous Waste Management and Cleanup activities are conducted under the process and schedule established in the Decision (RODs) under the FFA/CO process are signed by all three entities and represent a determination that environmental protection will be achieved through implementation of the remedy. The FFA/CO's role in INEL's environmental restoration program is discussed in Volume 2, sections 2.2.6 and 7.2.5.

DOE has a program for safely managing and storing SNF and other radioactive materials considered in the EIS. It is DOE's policy to design, construct, and operate its facilities to provide a level of safety and safety assurance that is in accord with applicable Federal regulations and DOE Orders. DOE will manage SNF in a manner that ensures protection of the environment and the health and safety of the public and site employees. The potential consequences of SNF management are discussed in Volume 1, Chapters 5. Environmental consequences of SNF management are presented for all alternatives in Volume 1, section 5.7. Mitigation measures are discussed in section 5.7.

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environmental impacts of all the alternatives considered in this EIS. The analyses show that the impacts of SNF management are small. Problems at existing storage facilities have been identified in Spent Fuel Inventory and Storage of the Department's Spent Nuclear Fuel and Other Reactor Irradiated Nuclear Material and Their Environmental, Safety and Health Vulnerabilities. This report, SNF vulnerability assessment, and associated action plans to resolve identified vulnerabilities are acknowledged in Volume 1, sections 1.1.2 and J-2 of the EIS. Additional information is in Volume 2, section 2.2.2.



## 06.01 (011) Vulnerabilities

### COMMENT

The commentor asserts that DOE failed to acknowledge the urgency of the vulnerability existing storage facilities at the Hanford Site, as highlighted in the Spent Fuel Working Group Inventory and Storage of the Department's Spent Nuclear Fuel and Other Reactor Irradiated Material and Their Environmental, Safety and Health Vulnerabilities, and that the EIS was modified to include the three specific actions included in the Hanford Federal Facility Consent Order (Tri-Party Agreement) in January 1994.

### RESPONSE

DOE concurs that action is necessary to deal with spent N-Reactor fuel, especially this situation was prominently identified in the spent nuclear fuel vulnerability assessment. This situation also is reflected in a June 1994 Tri-Party Agreement to initiate encapsulation of fuel. This agreement between the State of Washington, EPA, and DOE has a target date for removal of SNF and sludge from the 105-K Basins. A fourth amendment to the Tri-Party Agreement (January 1994) contains specific milestones related to managing SNF at the Hanford Site. SNF stored at Hanford and technologies for managing SNF are in Volume 1, section 1.1.2. Information on N-Reactor fuel and K-basin conditions is given in Volume 1, Appendix J. More general information on management options for degraded SNF and associated management technologies is in Volume 1, Appendices J-3 and J-4, respectively.

## 06.01 (013) Vulnerabilities

### COMMENT

The commentor states that the EIS is based on an unjustified presumption that spent nuclear fuel moved to be stored.

### RESPONSE

Volume 1, section 1.1.2 discusses the corrective actions for the SNF vulnerability assessment. DOE. Volume 1, section 1.1 of the EIS presents a comprehensive discussion on the management of SNF, including storing, stabilizing, transporting, and preparing it for final disposition. Technologies to accomplish these options are discussed in Volume 1, Appendix J. These technologies are incorporated to varying degrees in all of the alternatives, as described in Volume 1, Figures 3-1 and 3-6 show the number of shipments expected for each alternative and compares estimated shipments among all of the alternatives. The wide range in shipment estimates reflects DOE's desire to consider all realistic transportation possibilities and the related costs. In addition, the alternatives have definite purposes for relocating SNF, such as storing within a single secure facility. Thus, the alternatives attempt to balance transportation and other worthy considerations, including nonproliferation, worker safety, and cost effectiveness.

## 06.01 (014) Vulnerabilities

### COMMENT

Commentors note that descriptions of alternatives in Volume 1, Tables 3-1 through 3-3 identify storage problems at Test Area North but not at other storage facilities at the Idaho National Laboratory that were identified as not meeting current standards, such as the fuel storage facilities at ICPP-603. The commentor adds that no matter what alternative is selected, fuel storage facilities that do not meet current standards.

### RESPONSE

DOE is currently taking steps to correct outdated and potentially unsafe facilities. Storage facilities have been identified in the Spent Fuel Working Group Report on the Environmental, Safety and Health Vulnerabilities of the Department's Spent Nuclear Fuel and Other Reactor Irradiated Nuclear Material. This report, commonly called the SNF Vulnerability Assessment, and associated action plans to resolve identified vulnerabilities are in sections 1.1.2 and Appendix J-2 of the EIS. Additional site-specific information is in Appendices A through F. INEL plans to move SNF from ICPP-603 by December 31, 2000, and the impacts of the action are included in the No Action alternative. Volumes 1, Appendix K, and Volume 2, Chapter 5 summarize the environmental impacts of all the alternatives.

considered in this EIS. The analyses show that the impacts of all alternatives would be differences among the alternatives, these differences by themselves are not sufficient between alternatives. For all alternatives analyzed, DOE is committed to complying with applicable Federal, state, and local regulations and DOE Orders to protect the environment and the health and safety of the public and site employees.

## 06.01 (016) Vulnerabilities

### COMMENT

The commentor states that the condition of the spent nuclear fuel and the spent nuclear facilities is not adequately covered in the EIS, and cites specific problems with the K-basins at the Hanford Site. In addition, the commentor believes that, based on the Working Group Report on Inventory and Storage of the Department's Spent Nuclear Fuel Reactor Irradiated Nuclear Material and Their Environmental, Safety and Health Vulnerabilities, it is apparent that none of the facilities in the DOE complex is acceptable for the continuing inventories of spent nuclear fuel, much less additional fuel from another location.

### RESPONSE

DOE believes that the condition of SNF and existing storage facilities, as well as vulnerabilities, are adequately represented in the EIS. Problems with the K-basins facilities identified in the Spent Fuel Working Group Report on Inventory and Storage of the Department's Spent Nuclear Fuel and Other Reactor Irradiated Nuclear Material and Their Environmental, Safety and Health Vulnerabilities are being addressed by corrective actions proceeding independently of this EIS.

## 06.01 (017) Vulnerabilities

### COMMENT

The commentor states that it is urgent to address the vulnerabilities at the Hanford Spent Fuel Working Group Report.

### RESPONSE

Descriptions of SNF stored at the Hanford Site and technologies for managing SNF are in section 1.1. Hanford-specific information on N-Reactor fuel and K-basin conditions are in Volume 1, Appendix A, section 2.3. More general information on management options and associated management technologies is discussed in Volume 1, Appendix J, section 2.3. DOE agrees that it is necessary to deal with spent N-Reactor fuel, especially in the 105-K basin, which was prominently identified in the spent nuclear fuel vulnerability assessment. The assessment is reflected in a June 1994 Tri-Party Agreement to initiate encapsulation of uncontained SNF and sludge from the 105-K basins. The agreement between the State of Washington, EPA, and DOE has a target date of December 1993 for removal of SNF and sludge from the 105-K basins.

Under all alternatives except No Action, production reactor SNF would be removed from the 105-K basin and stored at an alternative storage location. Volume 1, Appendix A, section 2.3 has been revised to provide additional information on the Hanford Site vulnerabilities described in the spent nuclear fuel vulnerability assessment in December 1993.

## 6.1.1 Working Group Report and Action Plans

### 06.01.01 (001) Working Group Report and Action Plans

#### COMMENT

Some commentors state that the safety and health vulnerabilities, some of which have been identified in the Spent Fuel Working Group Report, have been ignored or are not acknowledged in the EIS. Others ask if the No Action alternative is an excuse to avoid its responsibilities for spent nuclear fuel vulnerabilities, this as a reason for supporting the No Action alternative.

#### RESPONSE

The Spent Fuel Working Group Report on Inventory and Storage of the Department's Spent Nuclear Fuel and Other Reactor Irradiated Nuclear Material and Their Environmental, Safety and Health Vulnerabilities is being addressed by corrective actions proceeding independently of this EIS.

Vulnerabilities (spent nuclear fuel vulnerability assessment) and associated actions identified vulnerabilities are acknowledged in Volume 1, section 1.1.2 and Appendix section 2.2.5 for INEL. These sections note that Phase I, Phase II, and Phase III released for public comment.

Additional site-specific information is presented in Volume 1, Appendices A through environmental consequences of SNF management are presented for all alternatives in 5.1, and mitigation measures are discussed in Volume 1, section 5.7. For all alternatives committed to comply with applicable Federal, state, and local regulations and DOE Order protection of the environment and the health and safety of the public and site employee exception of the No Action alternative, all alternatives fully address the identified vulnerabilities. Although the No Action alternative includes actions for safe and secure SNF management, Volume 1, section 5.1.2, the minimal actions allowed by the No Action alternative may not resolve all the long-term vulnerabilities at all existing facilities identified in the assessment, particularly for degraded SNF. As a result of public comments, additional vulnerability assessment and corrective action plans and their relationship to this assessment are presented in Volume 1, Appendices A, C, and F.

For additional discussion on the No Action alternative as it relates to SNF storage, see comment 06.05 (016).

## 6.2 Existing Facilities

### 06.02 (002) Existing Facilities

#### COMMENT

The commentor states that the use of a hypothetical spent nuclear fuel processing plant is misleading because DOE started constructing the fuel processing facility at Idaho Chemical Processing Plant in 1991.

#### RESPONSE

The hypothetical facility is described in Volume 2, Appendix C, SNF6. This facility is the basis for estimating the impacts of constructing the facility at other sites considered. The data sheet states that the existing Fluorine Dissolution Process (Idaho Chemical Process 601) and Fuel Processing Restoration Project were considered as part of the structural project.

## II COMMENT

Commentors question where spent nuclear fuel from foreign research reactors would be stored in the United States, and express concern about the ability of existing spent nuclear fuel storage facilities at the Savannah River Site to safely store foreign research reactor spent nuclear fuel, given the limited capacity of spent nuclear fuel storage areas.

#### RESPONSE

Foreign research reactor (FRR) SNF is discussed in Volume 1, sections 1.1 and 1.2. DOE is accepting up to 409 SNF elements, which will be stored temporarily at the Savannah River Site while preparing a separate EIS entitled Proposed Nuclear Weapons Nonproliferation Policy. The Foreign Research Reactor Spent Nuclear Fuel (Draft), which will determine whether to accept 25,000 additional elements and which port(s) of entry would be used. Volume 1 of the assessment discusses the cumulative environmental impacts of managing all DOE FRR SNF through 2035, including 25,000 FRR elements.

Consequently, the impacts associated with FRR SNF are evaluated under the management alternatives analyzed in Volume 1, along with the DOE SNF generated in the United States. Problems with existing storage facilities have been identified in the Spent Fuel Working Group Report on behalf of the Department's Spent Nuclear Fuel and Other Reactor Irradiated Nuclear Material Management, Safety and Health Vulnerabilities (spent nuclear fuel vulnerability assessment) report and associated action plans to resolve identified vulnerabilities are acknowledged in section 1.1.2 and Appendix J-2. Additional site-specific information is presented in Appendices A through F. As noted in the summary section of Volume 1, on a nationwide or site-specific implementation of any of the alternatives would not significantly contribute to cumulative impacts. The Preferred Alternative language for all DOE SNF, including

research reactors if the decision is to accept such fuel into this country, will be this EIS. For locations of foreign research reactors SNF management under DOE's see the response to comment 04.04 (008).

## II COMMENT

The commentor notes that the EIS emphasizes the use of new facilities and believes more effective use of existing and currently uncompleted facilities.

### RESPONSE

DOE did consider facility usage in its decision-making process for the identification alternative for SNF management. The alternatives considered in this EIS include the existing facilities and those based on using new facilities. To the extent practicable, modifying existing facilities when safety and environmental considerations would not when such modifications and operations are relatively cost effective. Although not a separate cost report on SNF management has been prepared and identifies the cost of using existing facilities and building new facilities.

## II COMMENT

The commentor asks when more effective storage facilities will be available.

### RESPONSE

Following the ROD for this EIS, if the alternative selected requires new storage facilities, upgrading existing facilities or building new facilities as soon as possible after the Environmental Policy Act (NEPA) reviews are performed and funding is available. A schedule for new nuclear facilities is 7 to 10 years. In the meantime, DOE will continue safely managing and storing SNF and other radioactive materials at each of the sites considered in the EIS. DOE manages SNF in accordance with applicable Federal, state regulations and DOE Orders in a manner that ensures protection of the environment and safety of the public and site employees.

## II COMMENT

The commentor states that DOE failed to analyze what it will do when ICPP-666 is full long before 2002.

### RESPONSE

DOE analyzed various methods of expanding the storage capacity at INEL to accommodate receipts under each of the alternatives. The capacity of Building 666 at the Idaho Plant (ICPP) can be increased by implementing the Increased Rack Capacity for ICPP-666 described in Volume 2, Appendix C, SNF2. This project would extend the ability to ICPP-666 by several years. In addition, depending on the alternative selected, additional capacity at INEL could be provided by additional reracking at ICPP-666 [see Volume 2, Appendix Additional Increased Rack Capacity (ICPP-666)] or by constructing Project SNF4, Dr Facility: Fuel Receiving, Canning/Characterization, and Shipping (see Volume 2, Appendix

## II COMMENT

The commentor asks if there are sufficient glass containers available at the Savannah River Site for all of the wastes that might be shipped to that site under some of the alternatives.

### RESPONSE

DOE believes that the commentor is referring to the vitrified (glass) high-level waste at the Savannah River Site Defense Waste Processing Facility. This facility does not rather it adds inert materials to the waste materials and melts them into a vitrified interim storage or disposition. This technology is not directly applicable to SNF, high-level waste produced from processing SNF. Detailed information on this activity is in Supplemental EIS - Defense Waste Processing Facility, Savannah River Site.

## II COMMENT

The commentor notes that, while the EIS states that spent nuclear fuel is stored in above-grade dry storage, there is some spent nuclear fuel in below-grade dry storage at the Idaho National Engineering Laboratory.

### RESPONSE

Some SNF is stored in below-grade dry storage at ICPP. Volume 1, section 5.8.3 has clearly include isolation from the environment in below-grade dry storage.

## II COMMENT

The commentor identifies specific safety issues associated with the continued management chemistry of spent nuclear fuel, principally N-Reactor fuel, currently stored at the continued wet storage for N-Reactor fuel.

### RESPONSE

DOE has fully evaluated the safety issues associated with SNF management at the Hanford sites and reported the results of this evaluation in the Spent Fuel Working Group Inventory and Storage of the Department's Spent Nuclear Fuel and Other Reactor Irradiated Materials and Their Environmental, Safety and Health Vulnerabilities. This EIS evaluation associated with SNF management at Hanford, including normal emissions and accidents, found that they would be small. The continued management, storage, and chemistry of SNF stored at Hanford will be evaluated in the Hanford-specific NEPA reviews. One such review being prepared for the transfer of fuel from the K-basins. Appendix A has been revised to reflect the proposed path forward for the K-basins.

## II COMMENT

The commentor expresses an opinion that DOE is currently storing "other spent fuel waste" at the Hanford Site in illegal "once-through" cooling pools that discharge waste contaminated soils and flush contaminants into the Columbia River.

### RESPONSE

Storing SNF in basins is not illegal; the SNF storage basins do not use "once-through" cooling. The SNF vulnerability assessment fully describes SNF at Hanford and identifies the vulnerabilities associated with this storage. Eight of the Hanford production reactors, all of which use "once-through" cooling systems. Direct discharges to the Columbia River from the basins in accordance with an existing, legal National Pollutant Discharge Elimination System permit. Measures are being taken to isolate from the rest of the basin a known area in the basins where leaks to the ground have occurred.

## II COMMENT

The commentor raises an issue about the numerical designation of a specific spent nuclear fuel facility.

### RESPONSE

The numerical designation of the facility in question is of no intended significance.

## II COMMENT

The commentor states that the spent fuel capacity, which could be provided by the Advanced Test Reactor at the Oak Ridge Reservation, is not addressed in the EIS.

### RESPONSE

A discussion of the SNF management program at the Oak Ridge Reservation (ORR) is in Appendix F, Part Three, section 2.3.

## II COMMENT

Commentors suggest that the discussion of transuranic waste in Volume 1, Appendix F not explain how or why the waste is generated. Commentors also question whether the Reservation has facilities capable of handling more waste than is obligated by the RESPONSE

Small quantities of radioactive wastes, including transuranic wastes, would usually be some fuel stabilization activities. The quantities identified in Volume 1, Appendix F estimates of the maximum amount that could be generated from these activities. Activities generated are likely to be smaller. A discussion of transuranic wastes is presented in Volume 1, section 3.3.4. Additional information about waste generation can be found in the F-Team Final Report. This report is available in reading rooms and information locations. EIS.

SNF management generates low-level waste. For ORR, this is described in Volume 1, Appendix F, section 5.14.2, which states that low-level waste generated by SNF management using the wet-storage option is estimated at 7,800 cubic meters, and the dry storage option would be significantly less. This section shows that ORR has a capacity for storing low-level waste of 98,300 cubic meters. The addition of 7,800 cubic meters would not significantly impact the decisions associated with low-level waste management at ORR.

SNF management also generates small amounts of transuranic waste, although transuranic waste is shipped to ORR, as commentors' remarks imply. This is discussed in Volume 1, Appendix F, section 5.14.2, which states that transuranic waste generated by SNF management is shipped directly to the Waste Isolation Pilot Plant, if it is available. If the Waste Isolation Pilot Plant is available, ORR transuranic management capacity would be evaluated, including options for storage.

## II COMMENT

Commentors state that reactor-irradiated nuclear materials are currently stored in some examples of the vulnerabilities at Idaho National Engineering Laboratory from the Working Group Report and express the opinion that DOE may not manage additional spent nuclear fuel in the future.

### RESPONSE

Problems at existing SNF storage facilities have been identified in the Spent Nuclear Fuel Inventory and Storage of the Department's Spent Nuclear Fuel and Other Reactor I Materials and their Environmental, Safety and Health Vulnerabilities. This DOE report vulnerability assessment, and associated action plans to resolve identified vulnerabilities are in Volume 1, sections 1.1.2 and Appendix J-2, and Volume 2, section 2.2.5. Additional information is in Volume 1, Appendices A through F. Environmental consequences of the alternatives are presented for all alternatives in Volume 1, section 5.1, and mitigation measures in Volume 1, section 5.7. For all alternatives analyzed, DOE is committed to comply with Federal, state, and local regulations and DOE Orders to ensure protection of the environment, health and safety of the public and site employees.

The Secretary of Energy has publicly affirmed that current DOE policy and practice give environmental considerations above other program goals. DOE is committed to protect the health of its workers, the public, and the environment. Furthermore, DOE intends to operate facilities in such a way as to provide a level of safety equal to or better than the operation of commercial facilities. DOE is working as quickly as possible to reduce adverse environmental impacts from past programs.

Immediate actions to correct any possible unsafe storage practices and to address SNF include transfer of SNF from the Underwater Fuel Storage Facility at ICPP-603 and to more modern facilities at ICPP-666. These actions also involve inspection and cannot be completed until the INEL. Volume 2, Table 2.2-1 lists specific corrective actions and schedules to address SNF at INEL.

## II COMMENT

The commentor asserts that DOE has not demonstrated that safe storage methods are available.

kept secret the fact that spent nuclear fuel storage areas at ICPP-603 had been uns nuclear fuel; that only after a whistleblower gave this information to the State wa that transfers of spent nuclear fuel from ICPP-603 at the Idaho National Engineerin be as safe or as quick as projected.

#### RESPONSE

DOE is taking the actions necessary to ensure safe SNF storage, including the SNF c transferred from ICPP-603. In the absence of substantiation of the commentor's ass reason to question the safety or speed of the transfers from ICPP-603 or the requir safety. DOE has aggressively identified and disseminated information about the vul deficiencies of its SNF management facilities.

See the response to comment 06.02 (015) for more information on vulnerabilities ass storage.

## II COMMENT

Commentors state that DOE should reconstruct all existing facilities at the Idaho N Laboratory that do not comply with the current design standards, such as those for confinement requirements.

#### RESPONSE

DOE Order 5480.28, Natural Phenomena Hazards Mitigation, sets forth DOE policy for constructing, and operating DOE facilities so that workers, the general public, and protected from the impacts of natural phenomena hazards on DOE facilities. This Or requires facilities to be reevaluated when there is any change in design and constr Additionally, in accordance with DOE Order 5480.23, Nuclear Safety Analysis Reports at INEL are required to undergo periodic safety analyses, including seismic and con reviews. When appropriate, design modifications are made. Besides design modifica emergency preparedness plans and administrative controls to minimize potential haza INEL reactors and confinement structures meet requirements as outlined in their saf safety evaluation reviews, and safety basis documents. These reviews show that pot site boundary are within regulatory limits. Engineered barriers are designed for o potentially high impact or high probability of occurrence.

The seismic analyses, which have been completed for some facilities at INEL, show t adequate to meet current earthquake standards. For facilities that do not meet cur actions have been identified and implemented. As an example, DOE is taking action nuclear fuels from potentially vulnerable facilities to modern facilities. Further described in Volume 2, Appendix C are proposed by DOE to replace or upgrade existin

## II COMMENT

The commentor states that long-term radiation exposure has compromised spent nuclea barriers in Savannah River Site canyons by changing the concrete into a spongy subs the commentor requests that DOE identify the canyon confinement boundaries and thos will ensure their safe maintenance.

#### RESPONSE

Periodic inspections of Savannah River Site canyons reveal some erosion of interior long-term exposure to acids and bases. This erosion, or etching, has left the surf appearance. Tests have shown this to be only a surface phenomenon that does not co structural integrity of the 4- to 5-foot-thick concrete confinement barriers.

## II COMMENT

Commentors state that DOE has failed to recognize in the EIS the problems with exis such as compliance with environmental laws and safety requirements, and question wh vulnerabilities identified in the Spent Fuel Working Group Report will be addressed commentor notes that the U.S. District Court ordered immediate action to mitigate u at the Idaho National Engineering Laboratory.

#### RESPONSE

As noted by the commentors, DOE prepared a report on vulnerabilities of the current

been directed by the Secretary of Energy to develop an integrated, long-term SNF pr vulnerability assessment and associated action plans to resolve identified vulnerab acknowledged in Volume 1, section 1.1.2 and Appendix J, and Volume 2, section 2.2.5 site-specific information is in Volume 1, Appendices A through F. Environmental co management for all alternatives are discussed in Volume 1, section 5.1, and mitigat discussed in Volume 1, section 5.7. For all alternatives analyzed, DOE is committe Federal, state, and local requirements and DOE Orders to ensure protection of the e health and safety of the public and site employees. However, under the No Action a minimum actions deemed necessary for the continued safe management of SNF would be Action has been taken at INEL to correct any possible unsafe storage practices, inc from the Underwater Fuel Storage Facility and other facilities into more modern fac inspection and canning of corroded fuel. However, the U.S. District Court did not action be taken to mitigate unsafe storage practices at INEL.

## II COMMENT

The commentor asks why Volume 1, Table 1-2 lists only 45 non-DOE facilities, while 55 such facilities.

### RESPONSE

There are a total of 57 non-DOE facilities representing domestic, licensed, small g 45 facilities identified in Volume 1, Table 1-2, Appendix E, Table 2.1-1 are non-DO contribute to projected SNF inventories during the next 40 years. The EIS has been distinction.

## II COMMENT

The commentor requests that the word "proposed" be deleted from Volume 1, Appendix with regard to the Fort St. Vrain Independent Spent Fuel Storage Installation, bec operation.

### RESPONSE

The requested changes have been made.

## II COMMENT

The commentor suggests that a reference to Las Vegas being 80 kilometers to the eas Site is incorrect and should be deleted.

### RESPONSE

The sentence intended to infer that the Union Pacific railroad is near Las Vegas, The EIS has been clarified.

## II COMMENT

The commentor identifies three issues related to the transfer of fuel between the K Site.

### RESPONSE

Volume 1, Appendix A, section 3.1 and Attachment A discuss basin storage at the Han potential for an inadvertent criticality related to this transfer has been consider Westinghouse Hanford Company, Richland, Washington, internal memorandum to J. P. Sc Westinghouse Hanford Company, Richland, Washington, Consequences Analyses of Hypot K-Basin Accident Scenarios, 105 KE/KW Accident Liquid Discharge, August 14; and Mon 1993, Engineering Study of the Transfer of Irradiated Fuels on the Hanford Site, WH Rev. 0, Westinghouse Hanford Company, Richland, Washington). The details provided Appendix A, section 3.1 and Attachment A are adequate for the evaluation and compar in this programmatic EIS.



## II COMMENT

The commentor requests more information concerning near-term plans for the Hanford  
RESPONSE

The EIS evaluates five alternatives for SNF interim management over a 40-year period. For any specific SNF management location will be the subject of site-specific NEPA. Volume 1, Appendix A, section 2.3 has been revised to provide additional information on the status of K-basin fuel. Additional information concerning foreign processing of SNF is in Volume 1, section 3.2.5 and Appendix A.

## II COMMENT

The commentor suggests that removal of all fuel from pool storage at ICPP-603 by the Idaho National Engineering Laboratory be added as a project in Volume 1, Appendix A (potential spent nuclear fuel projects for each alternative).

RESPONSE

On-site SNF movement is under-way. This activity is considered part of normal SNF operations and is not considered a potential activity.

## II COMMENT

The commentor requests that the EIS provide information on the capacity of ICPP-666 for the rerack projects and a comparison of the capacities to the amount of SNF expected at the Engineering Laboratory under the Ten-Year Plan alternative and the Maximum Treatment and Disposal alternative.

RESPONSE

The EIS has been modified to incorporate the requested information. Specific changes are in Volume 2, Appendix C, Projects SNF2, Increased Rack Capacity for ICPP-666, and SNF3 Increased Rack Capacity (ICPP-666). The comparison to the amount of SNF expected under the Ten-Year Plan alternative and the Maximum Treatment, Storage, and Disposal alternative is in section 3.1.1. The EIS has been changed to compare rack capacity and projected inventory for various alternatives.

## II COMMENT

The commentor states that the short-term impacts of the transfer of spent nuclear fuel from other facilities are not addressed in the EIS.

RESPONSE

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environmental impacts of all the alternatives considered in this EIS. The analyses show that the impacts are small.

## II COMMENT

The commentor notes that there has been nuclear waste stored at the Idaho National Engineering Laboratory for the past 40 years, which was not intended to be long-term storage, but in opinion that this does not justify adding additional waste, and that possibly all of the waste should be removed.  
RESPONSE

Volume 2, Chapter 5 and Volume 1, Appendix D discuss the impacts of waste management for SNF management, respectively. These impacts would be small under all the alternatives in the EIS.

## II COMMENT

The commentor states that the safety of existing technologies and facilities has been decades.

### RESPONSE

This comment is consistent with the EIS, which shows environmental impacts from all considered would be small.

## II COMMENT

The commentor states that the EIS does not adequately address spent nuclear fuel facilities that specific information such as layout plans, design standards, proposed safety monitoring plans be added. Additionally, the commentor states that this is unacceptable because the 40-year project lifetime exceeds the design lifetime of

### RESPONSE

This is a programmatic EIS to aid in making programmatic decisions for SNF management. It evaluates the general technologies and types of facilities required under the different specific information requested by the commentor will be developed as part of the implementation programmatic decisions. The information would be available in later facility-specific permit applications and design documents.

DOE recognizes the commentor's concern regarding facility design life. DOE is taking facility design lives and taking appropriate action to upgrade facilities to safely replace facilities that cannot be upgraded.

## II COMMENT

The commentor believes that releases due to degrading cladding have been calculated

### RESPONSE

Volume 1, Appendix A of the EIS has been modified to include an evaluation of the releases from the commentor.

## II COMMENT

The commentor questions the capacity of K-basins to combine 105-KE and 105-KW volumes

### RESPONSE

The footnote for Volume 1, Appendix A, Table 3-2 has been changed to clarify the capacity of K-basins with reracking.

## II 6.3 Inventories (amount and characteristics)

### II 06.03 (001) Inventories (amount and characteristics)

#### COMMENT

The commentor notes an apparent discrepancy between Volume 1, Table 1-1, which states existing foreign research reactor spent nuclear fuel in the United States, and the EIS Implementation Plan, which addresses 15,000 elements of foreign research reactor spent nuclear fuel.

#### RESPONSE

The Volume 1, Table 1-1 column headed "Existing" refers to SNF currently managed by facilities. Until received in the United States, FRR SNF is not managed by DOE as part of the existing inventory. Although the Implementation Plan for the EIS on the Weapons Nonproliferation Policy Concerning Foreign Research Reactor Spent Nuclear Fuel addresses 15,000 elements of FRR SNF that would be eligible for return to the United States

DOE in the event FRR SNF acceptance policy is renewed, the revised Implementation Plan 25,000 elements of FRR SNF eligible for return to the United States. These elements are managed by the United States or managed by DOE except for fuel shipments returned under the Environmental Assessment of Urgent-Relief Acceptance of Foreign Research Reactor Spent Nuclear Fuel.

## **II 06.03 (002) Inventories (amount and characteristics)**

### **COMMENT**

The commentor states that DOE refers to approximately 290 metric tons of heavy metal and approximately 1,200 metric tons of spent nuclear fuel at the Idaho National Engineering Experiment Station.

**RESPONSE**  
The 290 metric tons of heavy metal (MTHM) and 1,200 metric tons (total assembly weight) DOE currently manages at INEL refer to the same amount of material and are consistent with the heavy metal content of SNF as a common measure of the amount of fuel matrix in a fuel element. Fuel elements vary widely in structural material in relation to the amount of fuel matrix associated with them. Materials removed prior to storage may depend on fuel type. MTHM provides a uniform measuring of quantity, irrespective of the structure of the fuel element. DOE records structural material, either as an integral part of the fuel element, or as a separate

## **II 06.03 (003) Inventories (amount and characteristics)**

### **COMMENT**

The commentor states that the Kema Suspension Reactor Fuel is not addressed in the Environmental Assessment.

**RESPONSE**  
At the time the Draft EIS was prepared, the Kema Suspension Reactor Fuel was not on the inventory. Subsequently, the fuel was placed on the inventory. The EIS has been modified to include a discussion of the SNF management program at ORR, where the Kema Suspension Reactor Fuel can be found in Volume 1, Appendix F, Part Three, section 2.3.

## **II 06.03 (004) Inventories (amount and characteristics)**

### **COMMENT**

The commentor points out that Volume 1, Appendix B states that there is no DOE production fuel stored in Idaho. Volume 1, Appendix B, Table 2-2 indicates production fuel is stored at the Savannah River Chemical Processing Plant. The commentor suggests resolving this apparent discrepancy.

**RESPONSE**  
The ICPP does not store production reactor fuel, but only stores other aluminum-clad fuel. Savannah River Site, as referenced in Volume 1, Appendix B, Table 2-2.

## **II 06.03 (005) Inventories (amount and characteristics)**

### **COMMENT**

The commentor notes that in Volume 1, Table 1-8, the spent nuclear fuel inventory at the Reservation is 3.02 metric tons of heavy metal and in section 1.1 the inventory is of heavy metal and requests clarification.

### **RESPONSE**

The correct SNF inventory at ORR is 3.02 MTHM. The inventory number in Volume 1, section 1.1 has been corrected to 3.02 MTHM.

## **II 06.03 (006) Inventories (amount and characteristics)**

### **COMMENT**

The commentor notes that Volume 1, Appendix B, Table 2-2 indicates that graphite fuel is

underwater in ICPP-603 and believes that this is not correct.

**RESPONSE**

The commentor is correct. Graphite fuels, such as Fort St. Vrain SNF, are not stored in ICPP-603; they are stored dry in the Irradiated Fuel Storage Facility, a separate processing facility. This error has been corrected in the EIS.

## **II 06.03 (008) Inventories (amount and characteristics)**

**COMMENT**

The commentor requests spent nuclear fuel inventory information, including fuel types stored at National Engineering Laboratory storage facilities.

**RESPONSE**

The primary INEL SNF storage facilities, the types of fuel stored, and the stored quantities are presented in Volume 1, Appendix B, Table 2-2, and the relative properties of SNF at metric tons of heavy metal is presented in Volume 1, Appendix B, Figure 2-2. Additional information is available in Volume 1. This document, T. Wichmann letter, Subject Spent Nuclear Fuel, is available at any of the reading rooms and information locations listed in the Volume 1. Also see the response to comment 06.03 (013).

## **II 06.03 (009) Inventories (amount and characteristics)**

**COMMENT**

The commentor identifies an apparent discrepancy between Idaho National Engineering Laboratory nuclear fuel quantities in the EIS (288.68 metric tons of heavy metal) and those given by the INEL Site-Specific Advisory Board (266.55 metric tons of heavy metal). The comment requests clarification, particularly with respect to the fuel rods identified in the 1989 Radioactive Waste Management Information System as having been shipped to the Radioactive Waste Management Complex for disposal.

**RESPONSE**

Both numbers are correct in the context in which they are given. The presentation in the Site-Specific Advisory Board reflects the inventory as of approximately January 1994, and excludes fuel in the Experimental Breeder Reactor-II (EBR-II) at the time. (Nuclear fuel is not SNF until it has been discharged from a reactor after irradiation.) The number given in the EIS is a 1994 estimate that reflects the projected inventory as of July 1995. This projection includes EBR-II fuel, as well as approximately 4 MTHM of other fuels. These other fuels come from receipts from other INEL reactors and the 19 Naval shipments allowed under the modified 1989 Radioactive Waste Management Information System. The fuel rods identified in the 1989 Radioactive Waste Management Information System are not in this inventory because no positive determination has been made that they exist. The status of these materials will be resolved by the CERCLA action proposed for the Radioactive Waste Management Complex.

## **II 06.03 (010) Inventories (amount and characteristics)**

**COMMENT**

The commentor identifies specific inaccuracies in the EIS dealing with the number of elements that are in storage and that would be transferred to DOE under the agreement that the inaccuracies be corrected.

**RESPONSE**

DOE revised the EIS in response to this comment.

## **II 06.03 (011) Inventories (amount and characteristics)**

**COMMENT**

The commentor notes that the EIS incorrectly states that facilities on the Oak Ridge Reservation generate or manage spent nuclear fuel, high-level waste, or transuranic waste.

**RESPONSE**

ORR does generate and manage SNF and transuranic waste. A discussion of SNF generation is presented in the EIS in Volume 1, Appendix F, Part Three, section 2.2. Volume 1 has been modified to correct the statement about generation and storage of SNF and transuranic waste. Discussion of past generation activities at ORR is beyond the scope of this EIS.

## II 06.03 (013) Inventories (amount and characteristics)

### COMMENT

The commentor requests that considerably more detail be added to Volume 1, Table 1.1 to characterize the DOE spent nuclear fuel inventory.

### RESPONSE

The level of detail requested for SNF characterization is not available in all instances. Detail is not essential for a programmatic NEPA document. However, some additional SNF are provided in each site appendix. For example, see Volume 1, Appendix F, Part Three. Additional storage condition characterization can be found in the Spent Fuel Working Group's Inventory and Storage of the Department's Spent Nuclear Fuel and Other Reactor Irradiated Materials and Their Environmental, Safety and Health Vulnerabilities, which is referenced in the EIS.

## II 06.03 (014) Inventories (amount and characteristics)

### COMMENT

The commentor raises questions about complete reliance on high-efficiency particulate air (HEPA) filters for preventing emissions of radioactive particulates.

### RESPONSE

To minimize airborne releases, projects at INEL involving radioactive particulates are housed in a double-confinement structure. Conservative assumptions normally are used to estimate releases to the atmosphere, such as modeling only two filters in series when at least three are planned for operations. Also, although high-efficiency particulate air (HEPA) filters have estimated removal efficiencies of 99.97 percent (down to diameters of 0.3 micrometers), a conservative factor of only 99 percent typically is used for operational safety and accident analysis. A HEPA filter capable of removing particles as small as 0.001 micrometers from an airstream, but which performs the rating calibration at 0.3 micrometers using a standard aerosol generator, are tested annually and inspected daily to ensure that their efficiency is maintained. Safety analyses for forthcoming INEL facility operations will not assume perfect HEPA filter performance. Additional precautions will be taken to minimize airborne releases. The pressure drop across the filter is measured continuously to detect the formation of any holes or insecure filter seals. Filter temperature will be measured to promptly detect a filter fire. Finally, radiation sensors will be installed downstream of the filters to continuously monitor for radioactive releases. Detection of radioactive particulates above the natural background level will prompt shutdown of facility operations.

## II II COMMENT

The commentor emphasizes that spent nuclear fuel should be classified as waste.

### RESPONSE

The status of SNF is addressed in Volume 1, section 7.2.5. Historically, SNF was considered a valuable product and fissionable materials. As a recycled material, SNF is not covered under RCRA. Since April 1992, however, DOE's focus on most of its SNF has changed to ultimate disposition. This has created some uncertainty with regard to the regulation of SNF. DOE has initiated discussions with EPA, along with state regulators, about what should be designated as a waste, and about the potential applicability of RCRA to SNF. As decisions are made about which additional requirements might apply, SNF is still considered a material.

## II COMMENT

The commentor states that the EIS does not include substantial quantities of spent nuclear fuel managed and references the National Academy of Sciences recommendation to manage material generated from DOE's weapons-grade fissile materials program to an "SNF Standard." The commentor also states that the EIS does not include the recent purchase of 500 metric tons of spent nuclear fuel from dismantled nuclear weapons from the former USSR. The commentor is of the opinion that a programmatic EIS not including these potential sources of spent nuclear fuel increases the risks and other impacts and prevents meaningful evaluations of alternatives.

#### RESPONSE

The scope of the programmatic SNF portion of this EIS, described in Volume 1, section 1.1.1 of DOE SNF in a safe and environmentally sound manner until decisions regarding its management are made and implemented. For the purpose of this EIS (as described in the Volume 1 Summary), SNF is essentially defined as fuel that has been withdrawn from the reactor following irradiation, the constituent elements of which have not been separated. SNF includes uranium/neptunium target material, blanket subassemblies, pieces of fuel, and other materials, which are addressed in this EIS, include all current or reasonably expected SNF generated by DOE production, research, and development reactors; Naval reactors; and SNF (both domestic and possibly foreign of U.S. origin); and some special-case commercial SNF from the Three Mile Island Unit 2 reactor.

Materials that would be generated from the DOE weapons-usable fissile materials program, such as uranium from dismantled nuclear weapons, are outside the scope of this EIS. These materials, known as special nuclear materials, are being addressed in the Storage and Disposition of Weapons-Usable Fissile Materials Programmatic Environmental Impact Statement, which is currently being prepared. Special nuclear material is defined as (a) plutonium or uranium enriched in <sup>233</sup> or <sup>235</sup>, and any other material that the Nuclear Regulatory Commission, pursuant to the Atomic Energy Act of 1954, Section 51, determines to be special nuclear material artificially enriched by any of the foregoing, but which does not include source material. The above definitions are contained in Volume 1, Appendix H of the EIS. There are no differences between the management of SNF and special nuclear materials, such as the level of security required.

## II COMMENT

The commentor expresses the belief that Navy spent nuclear fuel is probably more stable than commercial nuclear fuel that DOE manages, such as graphite and aluminum spent nuclear fuel, and that Navy fuel represents a substantial portion of DOE's spent nuclear fuel, because the fuel presently in spent nuclear fuel will outlast the cladding and the temporary storage facility.

DOE agrees that because of the robust nature of Navy SNF, it is more stable and easier to handle than graphite and aluminum fuels. However, as described in Volume 1, section 1.1.3 and 1.1.4, Navy and DOE fuels will be stored safely until ultimate disposition is decided. Amounts and descriptions of SNF, refer to Volume 1, section 1.1.2 and site-specific information.

## II COMMENT

The commentor contends that the EIS is flawed because it does not indicate that transfer of spent fuel is a new venture by DOE, is untested, and may prove to be unsafe.

#### RESPONSE

DOE has substantial experience with handling and transferring SNF, including some commercial and otherwise damaged fuel. Transfers have been accomplished safely, and appropriate measures have been taken to similarly ensure the safety of future transfers, such as placing the fuel in corrosion-resistant containers. DOE also has safety analysis systems in place to ensure that the risks are understood and appropriately minimized. All transfers are conducted in accordance with regulatory analysis requirements.

## II COMMENT

The commentor identifies safety concerns related to handling metal fuels.

**RESPONSE**

The potential impacts from storage of radioactive materials associated with SNF are Volume 1, Chapter 5. The impacts of transporting SNF are also discussed in Volume environmental consequences of managing SNF, including metal fuels, are presented fo Volume 1, section 5.1, and mitigation measures are discussed in Volume 1, section 5 on the management of metallic SNF are provided in the site-specific appendices to V DOE has a program to safely manage, transport, and store all types of SNF and other at each of the sites considered in the EIS.

## II 6.4 Technologies

### II COMMENT

Commentors recommend several strategies for spent nuclear fuel management, as well technologies for the storing, stabilizing, and treating spent nuclear fuel.

**RESPONSE**

Some of the management strategies and technologies recommended by commentors are al actively pursued by DOE, while others are currently speculative and require additio determine their effectiveness for managing SNF. Volume 1, section 1.1 presents a c discussion of the options available for managing SNF, including storage, stabilizat preparation for final disposition. Specific technologies to accomplish these optio Volume 1, Appendix J.

### II COMMENT

The commentor questions what techniques are being developed to ensure safe, long-te waste so it cannot be used again.

**RESPONSE**

Numerous technologies are already available for managing radioactive materials, and actively developed for this purpose. Technological options for managing of SNF are 1, section 1.1.3 and Appendix J of the EIS. Current management practices for all t wastes are discussed in Volume 2, section 2.2.7, and technology development activit Volume 2, section 3.1. Volume 2 is specific to INEL, but waste management technolo apply to other DOE sites.

### II COMMENT

The commentor proposes on-site disposition of spent nuclear fuel using technology i

**RESPONSE**

DOE is unaware of any technology involving Tela coils that would be of practical be SNF. Technologies currently available are summarized in Volume 1, Appendix J. Dev technologies for managing SNF is beyond the scope of this EIS, and will be subjecte review prior to implementation.

### II COMMENT

The commentor supports technology development if it results in recycling fuel and e storage.

**RESPONSE**

Numerous technologies are available for managing radioactive materials, and others developed for this purpose. Technological options for managing SNF are described i 1.1.3 and Appendix J. Current management practices for all types of radioactive wa

Volume 2, section 2.2.7, and technology development activities are described in Vol  
Volume 2 is specific to INEL, but waste management technologies also generally appl  
DOE is committed to comply with all applicable Federal, state, and local regulation  
radioactive materials will be managed to protect the environment and the health and  
and site employees.

## II COMMENT

The commentor expresses the opinion that the "solution" to nuclear waste is the cer  
technology and that this technology should be at Hoquiam and Aberdeen, Washington.  
RESPONSE

Technologies for the encapsulation of high-level waste in ceramic or glass logs are  
immobilization of such wastes at a number of locations, including INEL's proposed  
Facility. DOE does not currently consider it reasonable to locate such facilities  
Location of similar facilities at sites other than INEL is beyond the scope of this  
1, Appendix J of the EIS describes technologies that are considered reasonable for  
present time.

## II COMMENT

The commentor states that the EIS merely describes potential technologies for proce  
fuel and does not analyze their potential consequences.

### RESPONSE

Volume 1, section 1.1 discusses DOE's plan to develop an integrated, long-term SNF  
discussion also points out a number of actions that prevent DOE from making all dec  
management for the next 40 years at this time. These actions include characterizat  
SNF and lack of acceptance criteria for ultimate disposition.

There are more than 90 types of DOE SNF, and any spent fuel that is to be disposed  
repository must first be characterized under an approved quality assurance program.  
database would be used to evaluate each fuel type to determine the extent of condit  
necessary to meet repository waste acceptance criteria.

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environme  
all the alternatives considered in this EIS. The analyses show that the impacts of  
small.

## II COMMENT

Commentors recommend a particular technology that they believe would be beneficial  
or disposition of spent nuclear fuel, specifically alternative fuel processing meth  
conducted at Argonne National Laboratories, and dry cask storage.

### RESPONSE

A summary of the technologies for SNF management, including some of the work being  
National Laboratories, as well as dry cask storage options, are presented in Volume  
of these technologies are already being actively pursued by DOE, while others are c  
and would require additional research to determine their effectiveness for managing  
that the technologies recommended are viable, they were considered, with many other  
the decision-making process for identification of the preferred alternative for SNF

## II COMMENT

The commentor states that no interim decision concerning DOE's metal fuel at the Ha  
made without considering the ultimate treatment measures necessary to prepare this  
disposition, and that some of the treatment options mentioned in the EIS are as yet  
RESPONSE

General technologies and practices for managing SNF, including metal fuels, are dis  
section 1.1.3 and Appendix J. Therein, it is noted that technologies for final dis



specified in advance of repository acceptance requirements. These requirements are completion and approval, but a combination of the technologies described in Volume satisfy the eventual acceptance criteria, even though some of them are as yet unproven. Furthermore, consideration is given by the alternatives analyzed in the EIS to providing processing flexibility that may prove necessary to meeting the acceptance requirements although the ultimate disposition of SNF is a high priority for DOE, the details of which have not been finalized and are beyond the scope of this EIS.

## II COMMENT

The commentor states that the Draft EIS does not adequately address the impacts of research and development of technologies to convert liquid high-level waste to a solid for disposal and proposals for any necessary interim storage.

### RESPONSE

The impacts of proposals regarding converting liquid high-level waste to a solid form and proposals for interim storage are discussed under the Project Summary entitled Facility (Technology Selection for Treatment of Sodium-Bearing and Calcined Wastes) Appendix C, HLW 2. DOE believes the analysis performed for this project is adequate.

## II COMMENT

The commentor recommends the quarter sections of land adjacent to the Zoo-East area "example site" the first time this site is mentioned in Volume 1, Appendix A, section 5.1.2 waiting until section 5.1.2 to make this statement.

### RESPONSE

The descriptions in Volume 1, Appendix A, sections 5.1 and 5.1.2 are very similar and clear. Thus, there is little chance for confusion about the wording, and no change is required.

## II COMMENT

The commentor wants Volume 1, Appendix A, Table 5.7-5 to indicate best available technology used in design.

### RESPONSE

Volume 1, Appendix A, section 2.2.2 has been changed to make it clear that DOE stands for all design/construction activities.

## II II COMMENT

One commentor states that the EIS alternatives specify either wet or dry storage without the environmental consequences or reasons for not splitting the two storage types. Commentors request information on the relative merits of wet storage versus dry storage for spent nuclear fuel. Commentors state that the EIS does not distinguish the consequences of repository wet storage. Information is also requested on spent nuclear fuel types, costs and short-term activities to fix storage problems, storage facility design, and work force.

### RESPONSE

The EIS discusses wet and dry storage in Volume 1, section 1.1.3 and Appendix J. Wet and dry storage type were included as input in modeling alternative's impact; therefore, the consequences related to a particular storage type (see Volume 1, Appendix I, for example). The analyses demonstrate that the impact considered would be small for both wet and dry storage. Separation into additional storage type is not likely to result in any significant difference in the consequences. Volume 1, Chapters 1, 4, and 5 and Appendix J, and Volume 2, Chapters 2 and 4 and A contain the requested information. Volume 1, section 3.3 summarizes the SNF cost evaluation. Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environmental impacts of all the alternatives considered in this EIS.

## II COMMENT

The commentor advocates long-term storage or disposal of spent nuclear fuel in a manner available to future generations to recover it and perhaps neutralize it, or otherwise treat it as available today.

### RESPONSE

The ultimate disposition of SNF is outside the scope of this EIS. This EIS addresses DOE SNF for a period of up to 40 years, until disposition decisions regarding DOE SNF

## II COMMENT

The commentor advocates considering monitored retrievable storage for spent nuclear fuel as a permanent disposal because of concern about criticality issues.

### RESPONSE

Although the final disposition of fissile materials and SNF is outside the scope of the Nuclear Waste Policy Act (NWPA), as amended, determined that these materials will be disposed of in a repository. DOE is aware of the criticality safety concerns and is considering several ways to address these concerns, including processing of some SNF to separate the fissile materials from the waste. If allowed by NWPA, DOE could use monitored retrievable storage for some SNF pending resolution of long-term criticality safety issues.

## II COMMENT

commentors suggest that highly enriched spent nuclear fuel and highly enriched spent nuclear fuel be stored in aboveground engineered storage facilities.

### RESPONSE

As stated in Volume 1, Chapter 1: "In 1992, the Secretary of Energy directed the Department of Energy to develop an integrated, long-term Spent Nuclear Fuel Management Program. This program is assessing the feasibility of fuel storage facilities, integrating DOE's many existing SNF activities into one program, and ensuring that SNF is managed in a safe and responsible manner of facility operation, and ensuring that SNF is managed in a safe and cost-effective manner." Solutions to the storage questions may require management strategies for these fuels, including such options as the construction of new facilities, including those suggested by the commentor, and stabilization of certain fuels. The Department has established a programmatic objective to define a management path and proceed toward the management of DOE SNF. Activities are currently in process to meet or address this objective. This EIS provides an overview of technologies for SNF management. Storage and disposition of SNF is beyond the scope of this EIS, but is being analyzed in the Programmatic Environmental Impact Statement for the Disposition of Weapons-Usable Fissile Materials (see Volume 1, section 1.2.3).

## II II COMMENT

The commentor states that the EIS should address the failure of science to turn nuclear waste into a material compatible with people and species on the planet.

### RESPONSE

Technology development addressed within this EIS, such as the Waste Immobilization technology (discussed in Volume 2, Appendix C, section HLW2), is focused on meeting waste acceptance criteria for waste disposition. These criteria will represent the best scientific consensus for a given technology.

## II 6.5 Ultimate Disposition

## II COMMENT

Some commentors request information on DOE's long-range plans for reprocessing or p commentors oppose reprocessing of spent nuclear fuel for reasons such as poor past nonproliferation issues, while others support it because they want the fissile mate resources to be recovered. Other commentors support processing for the purpose of long-term storage, but oppose reprocessing for purposes of separation and recovery

#### RESPONSE

Processing and reprocessing are defined in Volume 1, Appendix H. Processing means or physical process designed to alter the characteristics of the SNF matrix." Repr "processing of reactor-irradiated nuclear material (primarily SNF) to recover fissi order to recycle such materials primarily for defense programs." Thus, reprocessin processing. As discussed in Volume 1, Chapter 1, DOE made a policy decision in 19 of SNF for weapons production would be phased out. This policy is still in effect. Volume 1, Chapter 1 also indicates that several forms of SNF processing may still b certain types of SNF for safe storage. In addition, there are many different types differing characteristics that may require treatment for safe storage and final dis repository acceptance criteria for SNF and high-level waste for final disposition h therefore, the types of fuels that may require some type of treatment or processing Many of the treatments being studied do not separate fissile materials, although so repository acceptance criteria are not defined, it is not currently possible to det material will have to be separated from some fuels (such as fuels containing highly meet disposal criteria. Processing and use of existing reprocessing facilities are because these facilities could be utilized for short-term management of some fuels for extended underwater storage, but which are currently being stored underwater. for managing SNF are described in Volume 1, Appendix J.

## II COMMENT

Commentors express opinions that DOE is emphasizing transportation of spent nuclear considering the goals and consequences of these actions, and that DOE advocates rel fuel instead of addressing current storage problems and long-term spent nuclear fue including ultimate disposition. Commentors further note that there is no justifica nuclear fuel which is currently in storage before final disposition.

#### RESPONSE

Volume 1, section 1.1 of the EIS presents a comprehensive discussion of the options managing SNF, including storage, stabilization, transportation, and preparation for Specific technologies to accomplish these options are discussed in Volume 1, Append are incorporated to varying degrees in all of the alternatives, as described in Vol Volume 1, Figures 3-1 and 3-6 graphically indicate the number of shipments expecte and Figure 3-7 compares estimated shipments among all of the alternatives. The wid numbers reflects DOE's desire to consider all realistic transportation possibilitie stakeholder concerns. In addition, the alternatives have definite purposes for rel storing similar fuel types at a single site. In this way, the alternatives attempt concerns with other important considerations, including nonproliferation, worker sa effectiveness.

Problems at existing storage facilities have been identified in the Spent Fuel Work Inventory and Storage of the Department's Spent Nuclear Fuel and Other Reactor Irra Materials and Their Environmental, Safety and Health Vulnerabilities. This report, the SNF vulnerability assessment, and associated action plans to resolve identified acknowledged in Volume 1, section 1.1.2 and Appendix J-2. Additional site-specific presented in Volume 1, Appendices A through F.

Environmental consequences of SNF management are presented for all alternatives in 5.1, and mitigation measures are discussed in section 5.7. For all alternatives an to meeting applicable Federal, state, and local regulations and DOE Orders to ensur protection of the environment and the health and safety of the public and site empl alternatives, the environmental consequences would be small.

Volume 1, section 1.1.3 and Appendix J of the EIS notes that technologies for final cannot be specified in advance of repository waste acceptance criteria. These requ years from completion and approval, but a combination of the technologies described Appendix J may satisfy the eventual acceptance criteria. Furthermore, consideratio alternatives analyzed in the EIS to providing or maintaining processing flexibility to meeting the acceptance requirements.

Ultimate disposition of DOE SNF is a high priority. For planning purposes, DOE has SNF managed by DOE that is not otherwise dispositioned (e.g., chemically separated, waste being converted into a vitrified glass for repository disposal) is authorized repository. This authorization is subject to the physical and statutory limits of SNF meeting repository acceptance criteria, and payment of fees. As part of its SN DOE would (1) stabilize the SNF as needed to ensure safe interim storage, (2) characterize SNF inventory to assess compliance with the first repository's acceptance criteria, processing, if any, is required to meet the criteria. Decisions regarding the actual would follow appropriate review under NEPA and be subject to licensing by NRC. This would be implemented so as to minimize impacts on the first repository schedule.

## II COMMENT

Commentors express the opinion that disposal of spent nuclear fuel would result in resources, including some of the fission products, in addition to the uranium and plutonium DOE carefully guard and conserve these resources as well as pursue new and innovative technologies neutralizing the dangers of spent nuclear fuel and making use of its constituents. Commentors state that concentrating such resources in a particular location could encourage attempts to mine them.

### RESPONSE

Under the Nuclear Waste Policy Act, as amended (Section 122), disposing of SNF in a repository requires that the material be retrievable for recovery of economically valuable components in a short period of time. This requirement will be met by appropriate siting and design of the repository. In accordance with EPA's environmental standard (40 CFR 191), institutional provisions for safeguards and security will be implemented to address human intrusions.

## II COMMENT

The commentor states that DOE should stop trying to appease the public and create a new repository or stop generating nuclear energy.

### RESPONSE

Volume 1, Chapter 2 states the purpose and need for DOE action. DOE must deal in a sound manner with the SNF remaining in inventory, and with the small amounts to be generated by new programs. Neither disposal nor generation is within the scope of this EIS.

## II COMMENT

The commentor states that the EIS does not adequately integrate important information on final disposition (e.g. availability, cost, and acceptance criteria) with decisions on storage, and stabilization.

### RESPONSE

General solutions for managing SNF are discussed in Volume 1, section 1.1.3 and Appendix J. These sections note that technologies for final SNF disposition cannot be specified in advance of acceptance requirements. These requirements are several years from completion and a combination of the technologies described in Volume 1, Appendix J may satisfy the acceptance criteria. Furthermore, consideration is given by the various alternatives in both to providing or maintaining processing flexibility that may prove necessary to meet acceptance requirements. As stated in Volume 1, Chapter 2, activities related to the final disposition are beyond the scope of this EIS.

## II COMMENT

The commentor states that, because there is obviously no risk associated with any abandonment of construction of the Yucca Mountain repository and retain the site for use as a research and development facility using existing technologies, because they have been demonstrated to be so safe.

### RESPONSE

The EIS analyses indicate that the environmental impacts for all alternatives considered. Nevertheless, Congress has mandated in the Nuclear Waste Policy Act, as amended, that a geologic repository(s) for permanent disposal of SNF and high-level waste to ensure future generations are protected from the hazards of this material. Accordingly, DOE is presently characterizing the Yucca Mountain site to determine if it is a suitable site for a repository. If a repository is available, DOE will continue to store its SNF in accordance with the ROD.

## II COMMENT

Commentors state that criteria necessary for safe temporary or permanent disposal should be identified and the alternatives compared with them. Commentors give a list of issues that should be addressed, including transportation risks and accidents, human health risks, pollution prevention, and cost.

### RESPONSE

The ultimate disposition of SNF, including risks after disposition is beyond the scope of this EIS. Section 3.3 compares the impacts of the alternatives considered for managing SNF on topics identified by the commentor, including public health effects and risks from transportation, and accidents. Volume 1, Chapter 5 and supporting appendices and references discuss the potential environmental consequences and identify possible measures to be taken in the interim until disposal in the repository. Congress has mandated in the Nuclear Waste Policy Act, as amended, that the need for a repository and all alternatives to geologic disposal of SNF need not be considered by DOE.

In following the requirements of NEPA, this EIS includes a complete description of the impacts associated with all of the alternatives considered for SNF management. This EIS compares the alternatives and, as discussed in Volume 1, section 3.3.6, DOE evaluated the cost of the alternatives. This evaluation is available to the public.

Refer to Volume 1, Chapter 1 for an overview of DOE Spent Nuclear Fuel Management and

## II COMMENT

The commentor is of the opinion that spent nuclear fuel could be reprocessed at the INEL. Therefore, all spent nuclear fuel should be sent there for reprocessing.

### RESPONSE

DOE considered in this programmatic EIS the potential for processing SNF for stabilization. In 1992, DOE instituted a policy that phased out reprocessing for weapons production. SNF is not reprocessed at the Hanford Site. Because existing facilities at Hanford are not capable of managing many of the fuel types managed by DOE, and due to significant safety concerns of operating reprocessing facilities there, DOE has no plan to reprocess material at the Hanford Site.

## II COMMENT

The commentor expresses an opinion that the EIS does not address the problems associated with long-term storage of spent nuclear fuel.

### RESPONSE

The potential impacts from SNF storage over a 40-year period are fully discussed in this EIS and the Volume 1 site-specific appendices. Specific environmental consequences of SNF storage are presented for all alternatives analyzed in Volume 1, section 5.1, and mitigation measures are discussed in section 5.7. Further details are provided for each site in Volume 1, Appendices A through F.

DOE has a program to safely manage and store SNF at each of the sites considered in this EIS. Technological options for dealing with current and future inventories are discussed in Volume 1. In general, DOE has established a policy of designing, constructing, and operating facilities that meet applicable Federal, state, and local requirements and DOE Orders. All SNF is managed in a manner that ensures protection of the environment, the health and safety of site employees.

Ultimate disposition of DOE SNF is a high priority. For planning purposes, DOE has

SNF managed by DOE that is not otherwise dispositioned (e.g., chemically separated, waste being converted into a vitrified glass for repository disposal) is authorized repository. This authorization is subject to the physical and statutory limits of SNF meeting repository acceptance criteria, and payment of fees. As part of its SN DOE would (1) stabilize the SNF as needed to ensure safe interim storage, (2) chara SNF inventory to assess compliance with the first repository's acceptance criteria, processing, if any, is required to meet the criteria. Decisions regarding the actu would follow appropriate review under NEPA and be subject to licensing by NRC. Thi would be implemented so as to minimize impacts on the first repository schedule.

## II COMMENT

Commentors question the schedule for ultimate disposal of spent nuclear fuel with r delays in New Mexico and Nevada, and problems with the associated siting process. state that the proposed repositories are not the answer to spent nuclear fuel mana  
RESPONSE

The repositories to which the commentors apparently refer are the Waste Isolation P New Mexico, for disposal of defense transuranic (TRU) wastes, and the Yucca Mountai for disposal of commercial SNF and high-level wastes. Although the ultimate dispos TRU wastes, and the perceived delays in the availability of associated facilities a EIS, the assumptions used in evaluating alternatives for interim management of SNF wastes at INEL are discussed in this EIS.

As described in Volume 2, section 2.2.7, DOE plans to transport all stored and newl waste that meets the waste acceptance criteria to WIPP. DOE's current schedule is compliance with the disposal requirements as mandated in the WIPP Land Withdrawal A (Publication 102-579) and begin waste disposal operations in 1998. Alternatives fo in the interim are described in Volume 2, section 3.1.3.

Ultimate disposition of DOE SNF is a high priority. For planning purposes, DOE h SNF managed by DOE that is not otherwise dispositioned (e.g., chemically separated, waste being converted into a vitrified glass for repository disposal) is authorized repository. This authorization is subject to the physical and statutory limits of SNF meeting repository acceptance criteria, and payment of fees. As part of its SN DOE would (1) stabilize the SNF as needed to ensure safe interim storage, (2) chara SNF inventory to assess compliance with the first repository's acceptance criteria, processing, if any, is required to meet the criteria. Decisions regarding the actu would follow appropriate review under NEPA and be subject to licensing by NRC. Thi would be implemented so as to minimize impacts on the first repository schedule. See also the response to comment 04.01 (005).

## II COMMENT

The commentor believes that long-term management of spent nuclear fuel is extremel probably beyond human capability.

### RESPONSE

General technologies and practices for managing SNF are discussed in Volume 1, sect Appendix J. The EIS evaluates impacts of SNF management alternatives during the ne SNF can be disposed of in a geologic repository. For the long-term (beyond 40 year and design requirements contained in the Nuclear Regulatory Commission's regulation address issues that span geologic time to ensure safe isolation of this material. technologies for final disposition of SNF cannot be specified in advance of reposit requirements. These requirements are several years from completion and approval. See also the responses to comments 06.05 (007) and 06.05 (010) for additional infor disposition of SNF.

## II COMMENT

The commentor states that based on the overall risks of the spent nuclear fuel mana evaluated in the EIS, DOE should adopt an alternative plan that facilitates accepta

reactor fuels because the risks of doing so would be small compared with the possible United States from the diversion of fuel abroad.

RESPONSE

The policy on acceptance of foreign research reactor fuels is not within the scope response to comment 06.09 (013) for additional discussion.

## II COMMENT

The commentor asks if there is any research going on to find a better plan for spent than encapsulation or vitrification, which the commentor asserts have been shown in RESPONSE

DOE, and others, are researching techniques to process SNF for disposal. Contrary statement, encapsulation and vitrification are viable technologies for certain spent Volume 1, Appendix J. In all Volume 1 alternatives except No Action, research would ensure that there will be a broad base of technologies available, including vitrification multipurpose canisters, for treating SNF to meet the repository acceptance criteria

## II COMMENT

The commentor believes that solutions do not exist for the problems of spent nuclear including commercial sources and low-level waste. The commentor also states that a for dealing with these wastes is needed.

RESPONSE

General solutions for managing SNF are discussed in Volume 1, section 1.1, and tech dealing with the current and future inventories are described in Volume 1, Appendix program for safely managing and storing SNF at each of the sites considered in the Environmental Management Program prepared the DOE-Owned Spent Nuclear Fuel Strategy the safe, reliable, and efficient management of DOE SNF and its preparation for disposal available to the public. All SNF and other wastes will be managed to ensure protection and the health and safety of the public and site employees. While DOE complex-wide wastes is outside the scope of this EIS, the Waste Management Programmatic EIS is prepared to address an integrated national approach for dealing with these wastes. Management Programmatic EIS will be issued for public comment later this year.

## II COMMENT

Commentors express the opinion that DOE is avoiding making a clear choice of a path management of spent nuclear fuel, including a decision as to ultimate disposition, preparation of future documents to clear up the missing pieces to the "ultimate solution" that the experience exists to make a decision now. Commentors express frustration policy has not been established, and long-range plans do not exist. Such policy should include the total energy picture and its associated environmental impacts, the disposition of spent nuclear fuel and nuclear waste, and other "nuclear age" problem solutions are favored over interim solutions. Commentors prefer the No Action alternative nuclear fuel management.

RESPONSE

This EIS addresses the interim programmatic management of DOE SNF nationwide, in addition environmental restoration and waste management activities at INEL. Yucca Mountain the potential site for the first geologic repository. If the site is found suitable commercial SNF is expected to begin in 2010. Although acceptance of DOE high-level for 2015, the date for acceptance of DOE SNF at the repository has not been finalized SNF management is based on the maximum amount of time considered necessary to implement on the ultimate disposition of DOE SNF. DOE, through this EIS, solicited public comment program needs. Regarding INEL activities, this period is indexed to both strategic budget forecasts, as well as looking into the future as far as reasonably foreseeable site-wide programs. The programmatic SNF 40-year period is based on the maximum amount considered necessary to make and implement decisions on the ultimate disposition of define the criteria necessary to implement such disposition, and to have the facilities

repositories) available to implement ultimate disposition. The need for such inter discussed in Volume 1, Chapter 2, and the EIS Summary in greater detail. According a reasonable range of alternatives for the safe and environmentally sound management as the No Action alternative required by law. The programmatic SNF portion of the and updated as necessary.

With respect to establishing an overall national nuclear or energy policy, this EIS strategy for the period required to develop and implement decisions on ultimate disposition of DOE SNF is a high priority. For planning purposes, DOE had determined managed by DOE that is not otherwise dispositioned (e.g., chemically separated, waste being converted into a vitrified glass for repository disposal) is authorized for a repository. This authorization is subject to the physical and statutory limits of SNF meeting repository acceptance criteria, and payment of fees. As part of its SNF DOE would (1) stabilize the SNF as needed to ensure safe interim storage, (2) characterize SNF inventory to assess compliance with the first repository's acceptance criteria, processing, if any, is required to meet the criteria. Decisions regarding the actual would follow appropriate review under NEPA and be subject to licensing by NRC. This would be implemented so as to minimize impacts on the first repository schedule.

National policy with regard to the overall management of DOE's waste is being established preparation of the Waste Management Programmatic EIS, which is on a parallel course. The site-wide management of INEL waste streams is being coordinated with the program which will set the overall strategic approach. Commercial SNF and waste management activities are not within the scope of either this SNF or the Waste Management Program currently being prepared. National energy policy is not within the scope of any of See the response to comment 03.05 (007) regarding alternative sources of energy. See comment 01.01.01.01 (008) regarding preference for the No Action alternative.

## II COMMENT

A commentor states that the ultimate disposition of DOE spent nuclear fuel is general situations facing nuclear power utilities in the United States and other nuclear re the commentor suggests that all spent nuclear fuel be turned over to the International Agency for choice of one final repository. Other commentors suggest that an international taken regarding spent nuclear fuel disposition or storage issues.

### RESPONSE

The scope of the EIS for SNF is discussed in Volume 1, Chapters 1 and 2. These chapters EIS is restricted to considering temporary storage (through 2035) and related international managing only DOE SNF. Consequently, the location and nature of a geologic repository commercial SNF and SNF of international origin, are not included in this EIS because independent actions and the subject of Presidential and Congressional policies. SNF disposition is subject to the Nuclear Waste Policy Act, as amended, which restricts evaluations to the Yucca Mountain site in Nevada. No treaties or other arrangements envisioned as being feasible to combine SNF disposition efforts with those occurring in States. Nevertheless, among the technologies described in Volume 1, Appendix J are would prepare SNF for satisfying eventual repository acceptance requirements. These consistent with SNF disposition approaches being actively pursued or under consideration countries. Also, as outlined in Appendix J, DOE maintains an awareness of international take advantage of any technological advancements elsewhere that would be helpful in

## II COMMENT

The commentor suggests that funds should not be expended on moving toxic waste until known.

### RESPONSE

DOE is committed not only to developing a Federal geologic repository for permanent and high-level waste, but to providing safe interim storage pending availability of facilities. DOE has a program for safely managing and storing radioactive material considered in the EIS. Analyzing transportation of SNF and waste materials is necessary under the alternatives DOE is analyzing for providing safe interim storage and managing waste materials. The alternatives have definite purposes for relocating SNF and waste storing and/or treating similar fuel and waste types within a single secure facility



attempt to balance transportation concerns with other worthy considerations, including worker and public health and safety, and cost effectiveness.

## II COMMENT

The commentor suggests disposing of spent nuclear fuel at sea, enclosed in a submarine repository, or in outer space. Another commentor opposes disposing of spent nuclear fuel.

RESPONSE  
In the late 1970s the Federal Government evaluated a full range of reasonable alternative disposition of SNF and high-level waste. These alternatives included mixed geologic disposal, island disposal, and space disposal. As a result of this evaluation, DOE issued in 1979 by DOE, Congress mandated in the Nuclear Waste Policy Act in 1983 an amendment that geologic repositories be developed for permanent disposal of SNF and that research and development on alternative means and technologies for permanent disposal be continued and accelerated (section 222) and that, in particular, sub-seabed disposal progress be reported periodically to Congress (section 224). DOE is proceeding with the

## II COMMENT

The commentor states that to resolve the overall problem of storage of all types of spent nuclear fuel, DOE should press ahead to determine where the ultimate repository of spent nuclear fuel should be. The commentor urges DOE to complete the EIS process, and adopt the Navy's preferred alternative for Naval fuel.

RESPONSE  
Volume 1, section 3.1 shows the actions that would be undertaken by DOE to the extent of the Navy's preferred alternative. Activities related to the management of SNF, including development activities would be included. DOE is continuing to aggressively pursue candidate repository site at Yucca Mountain in Nevada for disposal of high-level waste. Ultimate disposition of DOE SNF is a high priority. For planning purposes, DOE has SNF managed by DOE that is not otherwise dispositioned (e.g., chemically separated, waste being converted into a vitrified glass for repository disposal) is authorized repository. This authorization is subject to the total quantity of DOE SNF and high-level waste exceeding 10 percent (by weight) of the first repository capacity limit (70,000 metric tons) DOE SNF meeting repository acceptance criteria, and payment of fees. As part of the program, DOE would (1) stabilize the SNF as needed to ensure safe interim storage, existing SNF inventory to assess compliance with the first repository's acceptance criteria, determine what processing, if any, is required to meet the criteria. Decisions regarding disposition of DOE SNF would follow appropriate review under NEPA and be subject to NRC. This path forward would be implemented so as to minimize impacts on the first

## II COMMENT

The commentor suggests that highly enriched spent nuclear fuel may never meet repository acceptance criteria due to criticality and safeguards concerns.

RESPONSE  
DOE agrees that highly enriched SNF is an issue regarding repository disposal, in part due to criticality. This issue is being addressed by DOE and the Nuclear Regulatory Commission. SNF will ultimately be processed to satisfy the repository acceptance criteria, Volume 1, section J of the EIS discusses the available technologies that may be needed for final disposal.

## II COMMENT

The commentor states that until final repository siting and its requirements are ascertained, DOE should consider other elements of the spent nuclear fuel program.

RESPONSE  
As the EIS discusses, interim management of SNF must be addressed for up to the next 100 years.

ultimate disposition. The alternatives identified and evaluated in the EIS represent reasonable alternatives for managing SNF, including the No Action alternative. The impacts of these alternatives, along with other decision factors such as cost, mission, and comment, will be considered in the decision-making process leading to the ROD.

## II COMMENT

The commentor states that DOE's Environmental Management and Office of Civilian Radiation Management should have an integrated approach to discussion of spent nuclear fuel management undertaken to define the ultimate disposal in a geologic repository. The commentor requests the EIS be revised to include a road map that would demonstrate a joint approach.

### RESPONSE

Ultimate disposition of DOE SNF is a high priority. For planning purposes, DOE has SNF managed by DOE that is not otherwise dispositioned (e.g., chemically separated, waste being converted into a vitrified glass for repository disposal) is authorized repository. This authorization is subject to the physical and statutory limits of SNF meeting repository acceptance criteria, and payment of fees. As part of its SNF management, DOE would (1) stabilize the SNF as needed to ensure safe interim storage, (2) characterize SNF inventory to assess compliance with the first repository's acceptance criteria, processing, if any, is required to meet the criteria. Decisions regarding the actual would follow appropriate review under NEPA and be subject to licensing by NRC. This would be implemented so as to minimize impacts on the first repository schedule. As part of this path forward strategy, the Office of Environmental Management and the Radioactive Waste Management have established a working group to provide an integrated identify and address technical, regulatory, and institutional issues regarding disposal in a geologic repository. This working group has made significant progress in defining and establishing work plans to address them.

## II COMMENT

The commentor asks about the potential long-term radiation of DOE's waste, and support Plan alternative for disposal of spent nuclear fuel at the Idaho National Engineering and Environmental Laboratory.

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environmental impacts of all the alternatives considered in this EIS. The analyses show that the impacts are small. While there are differences in the impacts among the alternatives, these differences are not sufficient to distinguish between alternatives.

Volume 1, Chapter 5 summarizes the radiological impacts associated with all the alternatives in this EIS, including using existing facilities and constructing new ones. Volume 1, the cost of alternatives. The health and safety of workers and the public has been evaluated for these alternatives and the identification of a preferred alternative. The evaluation of radiological impacts and facility costs is considered adequate for evaluation of impacts of all the alternatives considered in this EIS.

Volume 1, section 3.1 describes the preferred alternative for SNF management. See comment 04.04 (008). Volume 2, section 3.4 discusses DOE's preferred alternative for environmental restoration, and waste management activities at INEL. See the response to comment 04.11 (011).

## II COMMENT

The commentor states that the repository for spent nuclear fuel will not take DOE's management of Hanford Site materials should not be based on a mythical repository, but assumed that it will remain at Hanford forever. The commentor also states that the Storage negotiations with Indian Tribes have not been successful.

### RESPONSE

Although activities associated with licensing and opening the repositories or monitoring are outside the scope of this EIS, general solutions for managing SNF in the interim are discussed in Volume 1, section 1.1 and Appendix J of the EIS. Monitoring activities are discussed in Volume 1, section 1.1 and Appendix J of the EIS. Monitoring activities are discussed in Volume 1, section 1.1 and Appendix J of the EIS.

descriptions are in the Volume 1 site-specific Appendices A through F. Therein it technologies for final SNF disposition and a geologic repository site cannot be self repository performance requirements and waste acceptance criteria. These requirements several years from completion and approval. The repository must then be constructed could require decades to properly accomplish. Evaluating and opening disposal site materials is time consuming. Yucca Mountain is being studied as the potential site repository. If the site is found suitable, acceptance of commercial SNF is expected. Although acceptance of DOE high-level waste is planned for 2015, the date for acceptance the repository has not been finalized. DOE acknowledges these challenges by allowing suitable repository to become fully operational.

## II COMMENT

The commentor has an opinion on the second geologic repository being used for disposal of reactivity spent nuclear fuel.

### RESPONSE

Under the Nuclear Waste Policy Act, as amended, DOE is not authorized to work on a second repository and is required to report to the President and to Congress between January 2007 and January 2010 on the need for a second repository. Concerns regarding disposal of higher-reactivity (enriched) waste are addressed for the first repository between DOE Office of Environmental Management and the Radioactive Waste Management.

## II 6.6 Interim Management

## II COMMENT

The commentor states that the EIS does not evaluate reasonable, safe alternatives for SNF storage.

### RESPONSE

DOE believes that the alternatives analyzed in this EIS are reasonable and in accordance with Council on Environmental Quality (CEQ) requirements to consider a range of reasonable alternatives. Alternatives range from the No Action alternative, required by law, to an alternative for SNF at one of five sites. Alternatives dismissed are discussed in Volumes 1 and 2. DOE believes the discussion of the basis for dismissing other possible alternatives is adequate.

## II COMMENT

The commentor states that possible future contamination of the Idaho National Engineering and Environmental (ICPP-666) storage pool from spent nuclear fuel transferred from ICPP-603 could pose an environmental impact.

### RESPONSE

Volume 2, Appendix C, SNF4-1 discusses the Canning Characterization Project that could occur under various alternatives at ICPP to keep this situation from occurring.

## II COMMENT

Commentors express opinions that the interim spent nuclear fuel management program is "de facto" permanent storage sites for the nation and perhaps the world, and that the permanent storage site will be delayed. In addition, several commentors express that the interim site may not be suitable for permanent disposal of spent nuclear fuel, but because they are economically weak, the storage sites will be forced on them.

### RESPONSE

It is not DOE's intent to allow interim SNF storage sites to become de facto permanent. In fact, Congress mandated that the Federal Government pursue the development of a geologic repository.

the permanent disposal of SNF and high-level waste and directed DOE to study the Yucca Nevada to determine if it is suitable for this purpose. DOE currently is pursuing commitments to developing facilities for permanent isolation of SNF and high-level waste. Availability of such disposal sites, DOE must provide for safe and environmentally sound management of these materials. The implementation of safe interim storage and final disposition represents the solution that DOE seeks to define with this EIS. This EIS objectively evaluates 10 sites as reasonable alternatives for some level of management activity, without regard to or consideration of political or economic factors. The EIS includes environmental considerations, socioeconomic impacts, potential risk operations and reasonably foreseeable accident conditions, and other environmental factors of options for management of SNF. The EIS concludes that the alternative sites are suitable for management of SNF and that there would be no significant risk to the public environment due to interim management of SNF at any of the 10 sites being considered. DOE considered public comment while preparing the EIS, upon which decisions will be made. The EIS provides DOE with an informed basis for decisionmaking from the perspective of impacts and public comment, decisions will also consider such factors as national security costs. In addition, implementation of decisions is subject to independent processes of Congressional funding and environmental permitting. DOE intends, however, to develop a national SNF management strategy that best serves the overall needs of the nation.

## II COMMENT

The commentor asks DOE to quantify the relationship between the capacity of any new planned and DOE's total inventory of spent nuclear fuel.

### RESPONSE

The storage capacity required to safely manage the existing and projected SNF inventory under the programmatic approach selected by DOE. However, SNF storage facilities, complex-waste to provide the storage capacity required under the programmatic approach selected, availability of qualified existing storage under the specific alternative, at the site.

## II COMMENT

The commentor notes that the transfer of fuels from Idaho Chemical Processing Plant (ICPP-603) to the newer storage pool in ICPP-666 is not assessed in the EIS.

### RESPONSE

As discussed in Volume 2, Chapter 3, phasing out wet storage in ICPP-603 and moving it is part of the No Action, Ten-Year Plan, and Maximum Treatment, Storage, and Disposition. The impacts of moving this fuel have been assessed as a part of the overall impacts described in Volume 2, section 5.1.

## II COMMENT

The commentor notes that the EIS does not discuss the Test Area North Dry Cask Storage Project in Appendix B, section 3.1.1.3, where it would be expected. The commentor states that the spent nuclear fuel to the Idaho Chemical Processing Plant, it could be stored in the Test Area North with less transportation, less handling, and less attendant risk. The commentor states alternatives to moving spent nuclear fuel from Test Area North to Idaho Chemical Processing Plant should be evaluated in the EIS.

### RESPONSE

The Test Area North Dry Cask Storage Project is proceeding as an interim action. Volume 2 discusses two projects related to the use of Test Area North: (1) dry fuel storage canning, characterization and shipping; and (2) Test Area North pool fuel transfer. Volume 2, section 2.2 states that as part of the vulnerability corrective action plan for SNF management activity from Test Area North to a more central location.

## II COMMENT

The commentor states that relocation of spent nuclear fuel will only enlarge the area that will later have to be restored, because both the old storage area from which it be moved and the a new area to which it will be moved will both have to be restored

#### RESPONSE

Volume 1, section 1.3 discusses safe and environmentally sound management of SNF until regarding its ultimate disposition are made and implemented. Storage options at each other storage options are analyzed. The analysis of the storage options of each alternative estimated type and size of a representative storage facility potentially needed at DOE believes that relocation of some SNF could be a beneficial management strategy. would be designed both for safe storage and for more effective restoration when the Removing SNF from older facilities in the near term could serve to reduce future environmental problems and allow restoration activities to begin.

## II COMMENT

The commentor notes that the EIS states: "DOE has scheduled the installation and characterization and canning equipment in the Irradiated Fuel Storage Facility before Decision (1995)." According to Westinghouse Idaho Nuclear Company, the earliest start operating is early to mid-1996, and the location has not been determined.

#### RESPONSE

Installation of new fuel characterization and canning equipment in the Irradiated Fuel Storage Facility is now tentatively scheduled for early 1996. The text of the EIS has been revised to reflect this schedule.

## II COMMENT

The commentor notes that DOE sites predominantly use wet storage, and only limited experience exists within DOE. The commentor recommends that DOE consider commercial alternatives wherever possible to increase regulator and public acceptance, and reduce schedule

#### RESPONSE

Current and projected DOE SNF inventories are considered in this EIS. Existing storage facilities are identified, and their status, capacities, and accident histories are described. SNF integrity, corrosion and corrosion byproducts, storage technologies, and storage facility factors are integrated into the EIS analysis for each alternative. Storage options at the site of storage options are analyzed. The analysis of the storage options for each alternative estimated type and size of representative storage facilities potentially needed at DOE experience is considered whenever possible to increase public acceptance and reduce

## II 6.7 Cost

## II COMMENT

Commentors want cost evaluation to be part of this EIS.

#### RESPONSE

Volume 1, section 3.3.6 summarizes the costs for implementing actions under each alternative. A discussion of the cost evaluation has been prepared for use by decisionmakers. The cost evaluation considers capital costs for upgrades, operations, maintenance, decommissioning, and transportation. The evaluation also addresses additional system disposition.

## II COMMENT

The commentor expresses the opinion that a spent nuclear fuel management facility (



## II COMMENT

The commentor states that "moving nuclear waste around from 'interim site' to 'interim site' is extremely expensive." The commentor also states that funds would be put to better use in organizing individual sites and developing a program to eliminate waste."

### RESPONSE

DOE evaluated the transportation impacts of shipping SNF; the results are presented in Appendix I, and Volume 2, section 5.11. DOE's cost evaluation of the proposed alternative in Volume 1, section 3.3.6, shows that transportation costs do not differ among alternatives. Transportation of the entire DOE inventory between widely dispersed sites was analyzed for a reasonable range of alternatives and associated impacts for decisionmakers to consider. DOE shows that the impacts of all the alternatives considered in this programmatic EIS were within acceptable limits.

## II COMMENT

Commentors state that the cost information presented is not truthful and that the costs are overstated.

### RESPONSE

Volume 1, section 3.3.6 summarizes the cost for implementing actions under each alternative. A reference to the cost evaluation has been added to the EIS. DOE has provided a summary of the cost information in these documents to allow independent evaluation of the costs.

## II COMMENT

The commentor believes that the Federal Government should have to pay a tariff to allow the use of spent fuel.

### RESPONSE

The government provides support to communities that have Federal facilities and promotes the creation of jobs and other associated benefits. Payments to areas willing to accept spent nuclear fuel are determined by Congress or the President and are outside of the scope of this EIS.

## II COMMENT

The commentor raises issues regarding the costs and scheduling of alternatives being considered in the EIS.

### RESPONSE

While cost and schedule issues will be considered in the decisions facilitated by the EIS, administrative issues that are beyond the scope of this EIS. DOE prepared a cost report showing the cost of each of the alternatives under consideration in this EIS. This report is in section 3.3.6 and is available to the public.

## II COMMENT

The commentor states that the "massive" cost estimates for ICPP-666 and dry spent nuclear fuel "lead us to wonder whether the scale of these projects might be a Trojan horse" to allow additional spent nuclear fuel and other nuclear materials.

### RESPONSE

Decisions for storing other nuclear materials will be decided through other NEPA documents, those discussed in Volume 1, section 1.2. Facility costs are based on compliance with regulatory requirements, as identified in DOE Order 6430.1A, General Design Criteria. The size and number of facilities vary by alternative and the proposed amount of SNF to be stored.

## II COMMENT

The commentor states that the cost of each proposed alternative and assumptions should be public and included in the EIS.

### RESPONSE

While cost and scheduling issues will be considered in the decisions facilitated by administrative issues that lie beyond the scope of the EIS itself. DOE prepared a estimates the cost of each of the alternatives under considerations in this EIS. T Volume 1, section 3.3.6 and is available to the public.

## II 6.8 Commercial Spent Nuclear Fuel

## II COMMENT

The commentor requests that DOE accept greater-than-Class-C sealed sources from commercial power plants, and that the EIS be expanded to include acceptance of commercial power high-level waste.

### RESPONSE

Volume 2, section 3.1 discusses acceptance of greater-than-Class C sealed sources for all of the alternatives analyzed except the Maximum Treatment, Storage, and Disposal discussed in Volume 1, Chapter 1 and the Implementation Plan, this EIS does not add power plant SNF or high-level waste, which are the subjects of the Nuclear Waste Policy

## II COMMENT

The commentor raises issues regarding commercial spent nuclear fuel, particularly with compensating communities for storage as is being discussed with tribal governments nuclear fuel monitored retrievable storage.

### RESPONSE

This EIS pertains to the programmatic management of DOE SNF. Issues regarding SNF cognizance of commercial power utilities are beyond the scope of this EIS. Compensation of negotiations between the former Nuclear Waste Negotiator and various entities to SNF monitored retrievable storage facility is likewise beyond the scope of this EIS

## II COMMENT

The commentor states that the Project Summary for Fort St. Vrain spent fuel shipment corrections.

### RESPONSE

The Fort St. Vrain Project Summary has been corrected.

## II COMMENT

The commentor states that there may be a relationship between DOE spent nuclear fuel spent nuclear fuel.

### RESPONSE

Except for a very few special-case situations as described in Volume 1, section 1.1 discuss SNF from commercial power nuclear reactors. Volume 1, Chapter 2 states that must be made to establish an effective program for DOE SNF are a) where to conduct activities, b) the appropriate facilities, capabilities and technologies for SNF management research and development activities to support the SNF management program. See also the response to comment 04.01 (001).



## II COMMENT

The commentor notes that the EIS fails to fully consider that the remaining useful the Fort Ft. Vrain fuel is to be stored will be exceeded by 2015, well before a rep ready for this fuel.

### RESPONSE

In Volume 2, Chapter 3, DOE proposes a dry storage facility in each of the alternat Vrain SNF would be received. Yucca Mountain is being studied as the potential site repository. If the site is found suitable, acceptance of commercial SNF is expected. Although acceptance of DOE high-level waste is planned for 2015, the date for acceptance of the repository has not been finalized. DOE considered that the design life of the before a repository is ready. In the event that engineering studies cannot justify existing facility, Fort St. Vrain SNF would be moved to the new dry storage facility.

## II COMMENT

The commentor urges DOE to consider all contractual obligations to accept spent nuclear fuel specifically to Fort St. Vrain fuel.

### RESPONSE

DOE considered its contractual obligation to accept specific fuels in its identification alternative for programmatic SNF management.

## II COMMENT

The commentor states that DOE is responsible for accepting spent nuclear fuel and commercial power reactors, referring to commercial spent nuclear fuel that DOE is obligated to accept after 1998.

### RESPONSE

As described in Volume 1, Chapter 1, this EIS focuses on DOE SNF from production, research and development reactors, Naval reactors, university and foreign research reactors, and generators. Management of commercial SNF is beyond the scope of this EIS. DOE's responsibility for management of commercial SNF is within DOE's Office of Civilian Radioactive Waste Management.

## II COMMENT

The commentor states that, with respect to the No Action alternative, Public Service would not be able to transport the remaining spent nuclear fuel from the Fort St. Vrain facility for unrestricted use.

### RESPONSE

DOE recognizes that this would be a consequence of the No Action alternative. This is in DOE's identification of a preferred alternative for SNF management.

## II 6.9 Miscellaneous

## II COMMENT

The commentor states that the EIS should also include plans for dealing with errors and provide compensation for damages for those locations that accept spent nuclear fuel.

### RESPONSE

Potential accidents and the impacts associated with these accidents are generally discussed in section 5.1, and treated in more detail in Volume 1, Appendices A through F. DOE is planning for dealing with errors in its planning for new facilities and activities, as well.

facilities and activities. An in-depth analysis of the impacts of operations and p provided for SNF management operations in Volume 1, Appendices A through F, and for and remediation activities in Volume 2. The analyses include potential hazards and possible methods, measures, or controls to be employed to minimize them. The analy risk from SNF management operations is small.

DOE will use the statutory indemnity contemplated by the Price-Anderson Act (42 USC ready and prompt availability of funds to compensate the public for injuries and da nuclear incident arising from activities conducted by indemnified DOE contractors. provided under the Act would cover nuclear incidents at INEL as well as nuclear inc transport of material to and from the site.

Although the Price-Anderson Act is the primary means for compensating the public fo nuclear incidents, other remedies exist for claims not falling within the purview o claims against DOE or its employees may be cognizable under the Federal Tort Claims environmental damage may fall within CERCLA. These and other laws afford an injure mechanisms for seeking recovery for damages related to operation of DOE facilities.

## II COMMENT

The commentor states that spent nuclear fuel does not require treatment prior to di proposing treatment facilities only so that it can remain in the weapons production RESPONSE

Volume 1, section 1.1.3 discusses technologies for managing SNF, and more details o technologies are provided in Volume 1, Appendix J. Therein it is acknowledged that require treatment prior to disposal in a repository. However, there are many diffe widely differing characteristics, which may make treatment necessary. At this time acceptance criteria have not been defined; therefore, the fuels that might require determined at present. Processing is being considered to provide the chemical or m needed for ultimate disposition or to meet limits on size or amount of fissile mate containers.

Many of the SNF treatment technologies being studied do not require separation of u and thus would not be related to the weapons production business. As discussed in Volume 1, Chapter 1, DOE made a policy decision in 1992 that reprocessing of SNF fo production would be phased out. That policy is still in effect.

## II COMMENT

The commentor expresses the opinion that if people are unhappy with the current sit spent nuclear fuel, they should participate in solving the problem, rather than com RESPONSE

Through the public scoping process and the public comment period on the Draft EIS, organizations have participated in developing solutions to the SNF problem.

## II COMMENT

The commentor states that the inset entitled "What is Spent Nuclear Fuel?" in the S inadequate and 'harmless' as far as the lay public is concerned." RESPONSE

This insert was provided to clarify the definition of SNF for the public. It has b accomplish that.

## II COMMENT

Commentors request that DOE address spent nuclear fuel management program prioritie of funding and other limitations. commentors also request that the public be allow priorities.

RESPONSE

The Spent Fuel Working Group Report on Inventory and Storage of the Department's Spent Fuel and Other Reactor Irradiated Nuclear Materials and Their Environmental, Safety Vulnerabilities and associated action plans address some of DOE's immediate SNF management. The Phase I Action Plan was issued in February 1994, the Phase II Action Plan in April, and the Phase III Action Plan in October 1994. These action plans currently are being implemented. Additionally, DOE has issued the DOE-Owned Spent Nuclear Fuel Strategic Plan. This plan addresses the issues associated with the management of SNF and its preparation for disposal. The reading rooms and information locations listed in the EIS.

## II COMMENT

The commentor expresses the opinion that disposal of spent nuclear fuel in a repository is a concentration of valuable resources at a particular site and that it would attract people who would want to mine the resources.

### RESPONSE

The ultimate disposition of SNF is not within the scope of this EIS. This EIS considers only the decisions on ultimate disposition are made. The process that will decide disposition will consider the resource value of SNF and its constituents, along with safety and security concerns, including human intrusion.

## II COMMENT

The commentor, although supportive of options that would help solve the plutonium (and uranium) waste problem, feels the Isaiah Project is not an appropriate option.

### RESPONSE

The Isaiah Project has been proposed to consume excess plutonium as a mixed-oxide fuel for power-production reactors (specifically within the Washington Public Power Supply System). Projects concerning the management of existing special nuclear material, not SNF (which is reprocessed to yield recycled fissile material). The Isaiah Project is not included in this EIS and its adoption or rejection would not change the need for SNF management. Future management of the current inventory of special nuclear material is the forthcoming EIS for Storage and Disposition of Weapons-Usable Fissile Material. This EIS was recently announced in Volume 59 of the Federal Register, pages 31985 through 31990.

## II COMMENT

The commentor questions why the EIS frequently mentions reprocessing activities and that none of the fuel brought to the Idaho National Engineering Laboratory has been reprocessed.

The commentor seems to be referring to Fort St. Vrain SNF. While there has been some SNF at INEL over its 40-year history, no Fort St. Vrain SNF has been reprocessed. Facilities at INEL do not have the capability to reprocess graphite-moderated SNF used at Fort St. Vrain.

## II COMMENT

The commentor asserts that the illustration of a fuel rod and a fuel assembly in the EIS Nuclear Fuel is not representative of the vast majority of spent nuclear fuel managed by the Navy. Additionally, the commentor states that the other illustrations and descriptions in the EIS summaries do not include descriptions of many fuels, including Navy fuel TRIGA fuels.

### RESPONSE

The illustrations and descriptions in the fact sheet and the EIS summaries were chosen to help provide a basic understanding of the typical components of SNF. DOE is not providing detailed descriptions of the numerous SNF types in this EIS; rather, DOE is providing considerations, such as cladding type and condition, that are germane to a general understanding of SNF.

management.

## II COMMENT

The commentor asks DOE to responsibly manage the radioactive materials (including r and spent nuclear fuel) that DOE helped to create and wants to give DOE 2 years to strides in decontamination, stabilizing, and managing radioactive wastes, or replac

### RESPONSE

DOE acknowledges its responsibility for safe management of radioactive materials, i radioactive wastes. DOE is committed to comply with all applicable Federal and sta regulations, DOE Orders, and interagency agreements governing SNF and radioactive a wastes. According to Volume 2, Chapter 2, two programmatic EISs are being prepared level regarding DOE's SNF Program (Volume 1) and DOE's Environmental Restoration an Management Program (a separate forthcoming document). Additionally, DOE prepared a Spent Nuclear Fuel Strategic Plan for the safe, reliable, and efficient management preparation for disposal. This plan is available to the public.

For more discussion on DOE's legal authority and responsibility for managing radioa response to comment 03.04 (010).

## II COMMENT

The commentor states that receipt of spent nuclear fuel from foreign research react existing storage capacities at the Norfolk Naval Shipyard.

### RESPONSE

Volume 1, section 4.6 of the EIS states that the Norfolk Naval Shipyard is being co temporary storage of Naval SNF, not SNF from DOE facilities or international source Virginia, is a potential port of entry being analyzed in the forthcoming EIS on the Weapons Nonproliferation Policy Concerning Foreign Research Reactor Spent Nuclear F

## II COMMENT

One commentor favors keeping foreign spent nuclear fuel out of the United States. need, cost, motive, legality, or reasons behind such returns, especially given the SNF overseas. Another commentor states that this EIS should address what happens b on our shores.

### RESPONSE

Alternatives related to the acceptance of FRR SNF of United States origin, includin analyzed in a separate EIS. This EIS does analyze the impacts of domestic transpor of FRR SNF, which represents less than 1 percent of all SNF addressed in this EIS, return such fuel be made. The environmental impact analyses are designed to produc projection of the upper bound for potential environmental consequences. This requi appropriately conservative assumptions and analytical approaches. In this context, that an assumption or analysis would tend to overpredict, rather than underpredict, However, overly conservative analyses do not provide a useful basis for comparing a not make a final decision on the acceptance of FRR SNF until the EIS on a Proposed Nonproliferation Policy Concerning Foreign Research Reactor Spent Nuclear Fuel and completed.

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environme all the alternatives considered in this EIS. The analyses show that the impacts of small.

## II COMMENT

The commentor discusses the need for cleanup at other sites, including non-DOE site consideration of alternatives that gauge environmental and accidental risks. It wa should address these problems. The commentor states that DOE has willfully and ill

request the funding necessary to cease discharges and implement best available treatment cooling to meet the deadlines in the Hanford Cleanup Agreement. The commentor states lack of confidence for the storage of any additional fuels at Hanford.

#### RESPONSE

Cleanup at sites other than INEL is not within the scope of this EIS, which addresses management of SNF at all sites (Volume 1), and limited to cleanup and waste management at INEL (Volume 2). However, the Secretary of Energy has publicly affirmed that current practice emphasizes safety and environmental considerations above other program goals closely with EPA to remediate and eliminate adverse environmental impacts from past significant potential environmental impacts have been identified for any of the alternatives in the EIS for SNF management.

## II COMMENT

The commentor states that at a public meeting unrelated to the EIS, the commentor's nuclear waste was ignored. The commentor also states that the current administrative policy.

#### RESPONSE

This EIS pertains to the programmatic SNF management and SNF management, environmental and waste management at INEL. SNF reprocessing to recover uranium and plutonium fuel is being phased out as a matter of national policy. As discussed in Volume 1, section 1.2.3, reprocessing is being evaluated for certain types of SNF for the purpose of stabilizing and recovering, fissile materials, which would not eliminate the need for storage and a nuclear policy of this administration is outside the scope of this EIS.

## II COMMENT

The commentor expresses opinions regarding storage or disposition of fissile material if foreign research reactor and spent nuclear fuel are reprocessed in Europe.

#### RESPONSE

These concerns relate to the management of special nuclear material, not DOE SNF, being reprocessed to recycle fissile material. Future management of the current inventory of nuclear material will be addressed in the forthcoming EIS for the Storage and Disposition of Weapons-Usable Fissile Material, as described in Volume 1, section 1.2.3. See also comment 06.09 (013).

## II COMMENT

The commentor expresses an opinion that areas of little or no population and/or where resources are best suited for the management of spent nuclear fuel. The commentor selects the safest site for the management of spent nuclear fuel.

#### RESPONSE

Although SNF management activities can safely coincide with high population or other risks can be higher in such areas. However, public perceptions of risk from DOE activities tend to significantly exceed the risks as presented in this EIS.

The EIS evaluates 10 sites as reasonable alternative sites for some level of SNF management in the EIS includes a number of factors, including the risk to the public from operable foreseeable accident conditions. Discussions on public health and safety can be found in the Public Health and Safety sections in Volume 1 (and its associated site-specific appendix) and in the Health and Safety section in Volume 2. The EIS concludes that the consequences to the environment due to SNF management activities at any of the 10 sites under consideration are small. DOE tries to avoid high-population areas to the extent practicable.

## II COMMENT

The commentor raises questions regarding management of special nuclear materials and

generated by spent nuclear fuel processing, specifically the Actinide Recycle Demonstration Project. The commentor requests that the EIS provide additional information on projects or facilities that are in process.

**RESPONSE**  
The Electrometallurgical Process Demonstration Project (formerly the Actinide Recycle Project) is discussed in Volume 2, section 3.1.1, where SNF activities under the project are discussed. More detailed information is in Volume 2, Appendix C, section SNF8. The demonstration is to investigate electrochemical processing of SNF, to produce a waste potentially suitable for a geologic repository, and to quantify volumetric reduction. The demonstration would produce high-level radioactive waste containing fission products to be processed. Mixed wastes also would emerge because of electrorefiner operation and plus sodium contaminants in the SNF to be used for this demonstration. Management of special nuclear materials such as highly enriched uranium is being covered in the EIS and is outside the scope of this EIS. Specific information is not available for projects or activities that have not been conducted to acquire a valid base. Projects or activities have been included in the EIS as placeholders to present readers with as comprehensive a picture of forthcoming projects as is currently possible. These projects or facilities may require documentation. At such time, accurate information on secondary waste generation will be included in the assessment of impacts on waste management.

## II COMMENT

The commentor states that the period of interim storage addressed in this EIS should be based on repository availability, and that consideration should be given to recycling rather than disposal.

**RESPONSE**  
Decisions as to the ultimate disposition of SNF have not been made, and are outside the scope of this EIS. Ultimate disposition of DOE SNF is a high priority. For planning purposes, DOE has authorized SNF managed by DOE that is not otherwise dispositioned (e.g., chemically separated, waste being converted into a vitrified glass for repository disposal) is authorized for interim storage in a repository. This authorization is subject to the physical and statutory limits of SNF meeting repository acceptance criteria, and payment of fees. As part of its SNF management, DOE would (1) stabilize the SNF as needed to ensure safe interim storage, (2) characterize SNF inventory to assess compliance with the first repository's acceptance criteria, and (3) process SNF, if any, is required to meet the criteria. Decisions regarding the actual disposition of SNF would follow appropriate review under NEPA and be subject to NRC licensing. This path forward would be implemented so as to minimize impacts on repository schedule.

The 40-year period of interim management proposed in this EIS is designed to bound the period of interim management on ultimate disposition to be made and necessary facilities, such as a potential repository, implementation. Yucca Mountain is being studied as a suitable geologic repository. Acceptance of commercial SNF is expected to begin in 2010. Although acceptance of high-level waste is planned for 2015, the date for acceptance of DOE SNF at the repository is not yet finalized.

The current policy of DOE precludes the reprocessing of SNF to recover fissile materials. DOE policies are subject to periodic review, a need for the recovery or recycling of spent nuclear fuel is currently foreseen.

## II COMMENT

The commentor believes that nonfinancial costs of spent nuclear fuel management, to include environmental impacts, resources, and people, should be included in DOE's decision.

**RESPONSE**  
DOE interprets the nonfinancial costs identified by the commentor to mean impacts to the environment. Volumes 1 and 2, Chapter 5 summarize the environmental impacts for all of the alternatives in this EIS. The impacts for all of the alternatives would be small.

## II COMMENT

Commentors express opinions about the history of spent nuclear fuel mismanagement.  
RESPONSE

The condition of SNF management facilities is the result of a number of factors. R factors, it was recognized that the condition of these facilities cast doubt on the manage SNF. Therefore, DOE prepared a report commissioned by the Secretary of Ener vulnerabilities of the current program and has been directed by the Secretary to de long-term SNF program. The SNF vulnerability assessment and associated action pla identified vulnerabilities are identified in the EIS in Volume 1, section 1.1.2 and 2, section 2.5.2. Additional site-specific information is in Volume 1, Appendices Environmental consequences of SNF management for all alternatives are discussed in 5.1, and mitigation measures are discussed in section 5.7. For all alternatives an be small.

## II COMMENT

The commentor indicates that information on the Fort St. Vrain site was incorrect a data.

RESPONSE

These data have been incorporated into Volume 1, section 4.7.3, and Volume 1, Appen section 3.3.2.

## II COMMENT

The commentor suggests minor text revisions to the Fort St. Vrain project summary i Volume 2.

RESPONSE

The EIS has been changed to reflect the commentor's recommendation for text changes summary.

## II COMMENT

The commentor notes that Volume 1, Appendix A, section 2.1.4 fails to mention the B Company and states the opinion that privatization of certain activities may bring a key roles.

RESPONSE

The following sentence was added to the end of Volume 1, Appendix A, section 2.1.4: Bechtel Hanford Company and a team of subcontractors became DOE's environmental res contractor at the Hanford Site." Future contractor arrangements at the Hanford Si of this EIS.

## II COMMENT

The commentor states that the EIS does not solve the current Idaho National Enginee nuclear fuel problems, let alone the ones that will be there in 40 years.

RESPONSE

Volume 1, Chapter 1 discusses DOE's plan to develop an integrated, long-term SNF pr discussion also points out a number of reasons why DOE cannot make all decisions re management for the next 40 years at this time. These reasons include (a) lack of c the interim storage behavior of certain types of SNF and (b) lack of acceptance cri disposition. Volume 1, Appendix J identifies a number of activities currently unde management path and proceed toward ultimate disposition. This EIS is one step in t ultimate disposition.

This EIS is not a decision-making document; rather, it is a tool designed to aid th by evaluating the environmental consequences of proposed actions and their alternat

is used by decisionmakers in conjunction with other information, such as costs and course of action.

## II COMMENT

Commentors request information on the relative merits of wet storage versus dry storage fuel. Commentors state that the EIS does not distinguish the consequences of repro wet storage. Information is also requested on spent nuclear fuel types, costs and short-term activities to fix storage problems, storage facility design, and work-for commentors question why the two storage types are not split into two alternatives.

### RESPONSE

Volume 1, Chapters 1, 4, and 5 and Appendix J, and Volume 2, Chapters 2 and 4 and A the requested information. Volume 1, section 3.3 summarizes the Spent Fuel Management Evaluation Plan. Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize environmental impacts of all the alternatives considered in this EIS. The analyses all alternatives would be small.

## II COMMENT

The commentor expresses a general opinion that engineering has its limitations and account consequences and that the consequences of that engineering bring about and burden on the whole society to solve those problems.

### RESPONSE

The commentor's opinion regarding engineering and its perceived limitations is noted.

## II COMMENT

The commentor wants to keep spent nuclear fuel where it is until "we come up with a better alternative."

### RESPONSE

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environmental impacts of all the alternatives considered in this EIS. The analyses show that the impacts of all alternatives would be small. While there are differences in the impacts among the alternatives, these differences are not sufficient to distinguish between alternatives. Volume 1, section 3.1 describes the alternative for programmatic SNF management. See also the response to comment 04.04 (008).

## II COMMENT

The commentor objects to reactor irradiated nuclear material continuing to be generated because of the serious health effects.

### RESPONSE

Eliminating all current and future generation of DOE SNF would not significantly decrease storage, and final disposition challenges DOE faces. Inventories of DOE SNF are addressed in section 1.1. Approximately 86 percent of the current inventory originated in DOE reactors that have ceased to operate. Another 8 percent was generated in DOE experiments of which have been shut down. According to Volume 1, Table 1-1, the additional SNF over the next 40 years (until 2035) will amount to only a 3-percent increase in the inventory. Eliminating sources of DOE SNF altogether would require halting nuclear Navy operations and research at universities, which is not within the control of DOE and is outside the scope of the EIS.

## II COMMENT

The commentor asks how DOE will correct storage problems and what new designs have been developed for handling and storing spent nuclear fuel. Furthermore, the commentor is of the opinion that DOE should be more proactive in addressing storage problems.



not address the broad issues of permanent storage and availability of mature technology.  
RESPONSE

The potential impacts of storing radioactive materials associated with SNF are discussed in Chapter 5. Environmental consequences of SNF management are presented for all alternatives in section 5.1, and mitigation measures are discussed in section 5.7.

Problems at existing storage facilities have been identified in Spent Fuel Working Group Inventory and Storage of the Department's Spent Nuclear Fuel and Other Reactor Irradiated Materials and Their Environmental, Safety and Health Vulnerabilities. This report, the SNF vulnerability assessment, and associated action plans to resolve identified issues are acknowledged in Volume 1, section 1.1.2 and Appendix J-2. Additional site-specific information is presented in Volume 1, Appendices A through F.

At INEL, there is an ongoing dry storage demonstration project to gain information using commercial dry storage casks for DOE fuels. Dry storage is addressed in Volume 1, Appendix J.

Volumes 1 and 2, and Appendix D discuss the impacts of waste management on INEL and the environment. These impacts would be small under all alternatives considered in the EIS. That the transfer of SNF at INEL from potentially unsafe storage to a newer storage facility is an interim action that can be conducted prior to completion of this EIS. Depending on the decision for this EIS, various projects are proposed for interim storage and treatment and later decision on ultimate disposition. DOE will implement the reflected projects, Appendix C, to address the deficiencies with current storage. The proposed project existing underwater storage facilities to more effectively use their capacities, co storage facilities for longer-term storage, and processing facilities to treat and interim storage. Commercial experience is considered whenever possible to increase reduce costs and schedules. The alternatives analyzed attempt to balance consideration of existing facilities, minimizing transportation, consolidating similar fuels, and other

## II COMMENT

The commentor states that DOE fails to explain how spent nuclear fuel should be properly disposed of.

RESPONSE

Volume 1, section 3.1 describes the preferred alternative for programmatic SNF management in Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environmental impacts of the alternatives considered in this EIS. The analyses show that the impacts of all alternatives are within acceptable limits. Volume 1, Appendix B, Chapter 2, specifically discusses INEL's program objectives. See also the response to comment 04.04 (008).

## II COMMENT

The commentor states that the EIS fails to take an integrated approach to addressing the issues involved in handling and storing spent nuclear fuel.

RESPONSE

Volume 1, Chapter 1 discusses DOE's plan to develop an integrated, long-term SNF management plan. Appendix J identifies a number of activities currently under way to define a management plan toward ultimate disposition. DOE's Plan of Action to Resolve Spent Nuclear Fuel Vulnerabilities identified in the Spent Fuel Working Group Report on Inventory and Storage of the Department's Spent Nuclear Fuel and Other Reactor Irradiated Nuclear Materials and Their Environmental Health Vulnerabilities identified this EIS as a vehicle to address the lack of a permanent disposition of SNF. In addition, this EIS focuses on a programmatic approach to SNF management. Site-specific SNF management issues will be addressed by additional NEPA reviews and

## II COMMENT

The commentor requests that more details and options be provided in the EIS concerning alternatives, storage facility design, types of processing, improved long-term storage, and nuclear fuel disposition. The commentor states that many of these same concerns were discussed in scoping meetings and have not been adequately addressed, including minimization of

transportation.

#### RESPONSE

The EIS has been augmented to include the estimated range of costs for each of the alternatives. The technological options for interim storage, transportation, and site preparation are discussed in Volume 1, Appendix J. Evaluation of specific designs and technologies required at the programmatic NEPA level and will be analyzed in follow-on site-specific documents. DOE received approximately 1,900 comments during on the EIS scoping process and attempted to respond to all the public concerns; however, the ultimate disposition of SNF specifically addressed in this EIS due to unknown future requirements for geologic criteria, which in turn affect stabilization and treatment strategies. The range of options DOE believes may be required for stabilization, treatment, or preparation for ultimate disposition are discussed in Volume 1, Appendix J. Minimization of transportation is a DOE goal for efficient operations.

## II COMMENT

The commentor states that the EIS should evaluate the best storage form, processing and other technical considerations required for long-term management of spent nuclear fuel.

**RESPONSE**  
Volume 1 of the EIS is programmatic in nature and addresses the major management decisions and the groundwork for the more specific technical decisions that the commentor believes such as the best storage form and processing requirements for specific fuel types. The EIS discusses the current SNF management problem and the need for action in Volume 1, C. Technical considerations and solutions to the management problems are addressed in Volume 1, J, including storage options, containers, and processing options.

## II COMMENT

The commentor states that storage is not the optimum strategy for spent nuclear fuel. The commentor also states that plutonium is too readily available for making weapons and should be degraded as rapidly as possible while maintaining the safety of the environment and other valuable constituents.

#### RESPONSE

This EIS does not consider ultimate disposition of SNF; rather it considers interim decisions on ultimate disposition of special nuclear material are made. The process and manner of ultimate disposition, which is the subject of another EIS, will consider SNF and its constituents, as well as the necessary safeguards against diversion of SNF for weapons production. DOE has a program for safely managing and storing SNF and other radioactive materials at each of the sites considered in the EIS. DOE will manage SNF in accordance with applicable federal, state, and local requirements and regulations and DOE Orders in a manner that protects and the health and safety of the public and site employees.

## II COMMENT

The commentor requests that specific corrections or specific additional information management options for the fuel elements at the Veterans Administration Medical Center in Omaha, Nebraska, be included in the EIS.

#### RESPONSE

As described in footnotes to Volume 1, Appendix E, Table 2.1-3, the Veterans Administration Medical Center in Omaha, Nebraska, is a Category 2 Facility that does not routinely generate SNF. SNF is expected to be generated by this facility during the period covered by this EIS. Volume 1, Table 1-2 has been deleted from the EIS because it duplicated Volume 1, Appendix E, Table 2.1-3.

## II COMMENT

The commentor states that Volume 1, Appendix J does not appear to recognize the commentor's concerns regarding the management of spent nuclear fuel.

when spent nuclear fuel is damaged, and that oxidation products on the outside of facility contamination during canning. The commentor states that the EIS should be this omission.

**RESPONSE**

The section of Volume 1, Appendix J to which the commentor refers is intended to provide the reader of the technology options that are available to DOE for use in preparing Technologies would be evaluated for appropriateness for specific fuels prior to use identify potential problems and appropriate mitigation measures. The potential for contaminated fuels is routinely considered in DOE's evaluations of technologies for conditioning of SNF.

## 06.09 (050) Miscellaneous

**COMMENT**

The commentor states that the Project Summary on Fort St. Vrain spent fuel shipment previously prepared environmental assessment that may not be relevant.

**RESPONSE**

The EIS has been corrected in response to this comment.

## 06.09 (051) Miscellaneous

**COMMENT**

The commentor expresses the opinion that the storage of spent nuclear fuel has adverse Americans and will next destroy the institution of the American family farm.

**RESPONSE**

Volume 1, Chapter 5 and Appendix K, and Volume 2, Chapter 5 summarize the environmental all the alternatives considered in this EIS. The analyses show that the impacts of small.

## 06.09 (052) Miscellaneous

**COMMENT**

The commentor notes that in Volume 1, section 4.6.5 the effective dose equivalent is per year, but in Appendix D, effective dose equivalent values are provided in millirem commentor suggests that units be consistent throughout the document.

**RESPONSE**

The EIS has been revised to ensure uniformity in units throughout the document.

## 06.09 (053) Miscellaneous

**COMMENT**

The commentor wants to ensure that mitigation is an integral part of planning and should consider avoidance, minimization, rectification, and compensation.

**RESPONSE**

If necessary, a mitigation action plan will be prepared for this EIS in accordance with 10 CFR 1021. All necessary mitigation is generally noted in this EIS. Volume 1 mitigation measures relative to environmental impacts.



