

April 22, 2002

Mr. D. N. Morey  
Vice President - Farley Project  
Southern Nuclear Operating  
Company, Inc.  
Post Office Box 1295  
Birmingham, Alabama 35201-1295

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 RE: ISSUANCE OF  
AMENDMENTS (TAC NOS. MB2161 AND MB2162)

Dear Mr. Morey:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 155 to Facility Operating License No. NPF-2 and Amendment No. 147 to Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated June 5, 2001.

The amendments revise TS Surveillance Requirement 3.4.14.1 to clarify the frequency of performance with regard to Reactor Coolant System Pressure Isolation Valves in the Residual Heat Removal System flow path. Also, related TS Bases and editorial changes are part of this TS change.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Frank Rinaldi, Project Manager, Section 1  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosures:

1. Amendment No. 155 to NPF-2
2. Amendment No. 147 to NPF-8
3. Safety Evaluation

cc w/encl: See next page

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cc w/encl: See next page

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SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 155  
License No. NPF-2

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated June 5, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 155, are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

John A. Nakoski, Chief, Section 1  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 22, 2002

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 147  
License No. NPF-8

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated June 5, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 147, are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

John A. Nakoski, Chief, Section 1  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 22, 2002

ATTACHMENT TO LICENSE AMENDMENT NO. 155

TO FACILITY OPERATING LICENSE NO. NPF-2

DOCKET NO. 50-348

ATTACHMENT TO LICENSE AMENDMENT NO.147

TO FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Replace the following pages of the Appendix A Technical Specifications and associated Bases with the attached revised pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3.4.14-3  
B 3.4.14-3  
B 3.4.14-6  
B 3.4.14-7  
B 3.4.14-8

Insert

3.4.14-3  
B 3.4.14-3  
B 3.4.14-6  
B 3.4.14-7  
B 3.4.14-8

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 155 TO FACILITY OPERATING LICENSE NO. NPF-2  
AND AMENDMENT NO. 147 TO FACILITY OPERATING LICENSE NO. NPF-8  
SOUTHERN NUCLEAR OPERATING COMPANY, INC., ET AL.  
JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2  
DOCKET NOS. 50-348 AND 50-364

## 1.0 INTRODUCTION

By letter dated June 5, 2001, the Southern Nuclear Operating Company, Inc. (SNC) et al., submitted a request for changes to the Joseph M. Farley Nuclear Plant (Farley), Units 1 and 2 Technical Specifications (TS). The requested changes would revise TS Surveillance Requirement (SR) 3.4.14.1 to clarify the frequency of performance with regard to Reactor Coolant System (RCS) Pressure Isolation Valves (PIVs) in the Residual Heat Removal (RHR) System flow path. Also, related TS Bases and editorial changes are part of this TS change.

## 2.0 BACKGROUND

There is a long docketed history related to PIV testing at Farley. The following references document the review of this issue: (a) June 5, 1987, Alabama Power Company letter from R. P. McDonald to the NRC entitled "Joseph M. Farley Nuclear Plant - Units 1 and 2, Response to Generic Letter 87-06, Periodic Verification of Leak Tight Integrity of Pressure Isolation Valves;" (b) September 22, 1980, Alabama Power Company letter from F. L. Clayton, Jr. responding to NRC letters from R. L. Tedesco dated August 25, 1980, and September 10, 1980, that contained the basis for not testing the RHR suction valves and Low Head Safety Injection Cold Leg valves after seat disturbances due to flow; (c) Section 3.9.4 of Supplement 5 to the Unit 2 safety evaluation report discusses the resolution of this issue at the time that the full power license was issued; and (d) Amendments 50 and 41 for Units 1 and 2, respectively, finalized and standardized the lists for PIV testing in the TS.

As part of the conversion to the Improved Technical Specifications (ITS), an asterisked note was deleted from Table 3.4-1 of the old TS that listed the RCS PIVs. The note was to identify the valves to which the following SR frequency applied: "Following valve actuation due to automatic or manual action or flow through the valve for valves identified in Table 3.4-1 by an asterisk." Performance of this SR at this frequency did not apply to the RCS PIVs in the RHR flow path. In the documentation for conversion from the old TS to the ITS, the change was identified as an administrative change, because the Mode 4 applicability and note 2 to SR 3.4.14.1 effectively provided the same exception. SNC has proposed a TS change that clarifies the fact that the performance of this SR at this frequency does not apply to the RCS PIVs in the



RHR flow path, consistent with the requirements of the old pre-conversion Farley TS. Therefore, approval of the proposed change clarifies a potential source of confusion introduced into the TS during the conversion to the ITS.

### 3.0 EVALUATION

The applicability of limiting condition of operation (LCO) 3.4.14 is as follows: "MODES 1, 2, and 3, and MODE 4, except valves in the RHR flow path when in, or during the transition to or from, the RHR mode of operation." SR 3.4.14.1 is currently required to be performed at two frequencies, as follows: "18 months, prior to entering MODE 2" and "Following valve actuation due to automatic or manual action or flow through the valve." SR 3.4.14.1 is modified by three notes. Note 2 states the following: "Not required to be performed on the RCS PIVs located in the RHR flow path when in the shutdown cooling mode of operation."

As part of the conversion to the ITS, an asterisked note was deleted from the old TS on Table 3.4-1 that listed the RCS PIVs. The purpose of this note was to identify the valves to which the following SR applied: "Following valve actuation due to automatic or manual action or flow through the valve for valves identified in Table 3.4-1 by an asterisk." RCS PIVs in the RHR flow path were not identified by an asterisk and were therefore not subject to the SR at this frequency. In the conversion, this asterisked note was deleted when the RCS PIVs LCO 3.4.14 Mode 4 applicability for valves in the RHR flow path and the LCO 3.4.14 note 2 to SR 3.4.14.1 were adopted. In the section "Discussion of Change," associated with that change, this was identified as an administrative change, because the Mode 4 applicability and note 2 to SR 3.4.14.1 effectively provided the same exception as the asterisked note in the old TS. The NRC Safety Evaluation for the ITS conversion also characterizes this change as administrative. Administrative changes are defined in part as "editorial in nature or involve the reorganization or reformatting of TS requirements without affecting technical content or operational restrictions."

The ITS was implemented on March 1, 2000. Recently, some confusion has arisen concerning the relationship between the RCS PIVs LCO 3.4.14 Mode 4 applicability for valves in the RHR flow path, note 2 to SR 3.4.14.1, and the frequency "Following valve actuation due to automatic or manual action or flow through the valve" as it relates to RCS PIVs located in the RHR flow path. This TS change clarifies, as discussed in the conversion to the ITS, that SR 3.4.14.1 does not apply to the RCS PIVs in the RHR flow path following valve actuation due to automatic or manual action, or flow through the valves.

The staff agrees that during the conversion to the ITS a potential source of confusion was introduced into the TS. The staff has determined that the proposed changes are consistent with the requirements of the old TS and the ITS at Farley. Further, the staff has determined that the changes clarify the applicability of SR 3.4.14.1 to RCS PIVs in the RHR system flow path with regards to the frequency condition, "Following valve actuation due to automatic or manual action or flow through the valve." On this basis, the staff finds the proposed changes to SR 3.4.14.1 frequency acceptable.

### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of Alabama official was notified of the proposed issuance of the amendments. The State official had no comments.

## 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change the surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding [66 CFR 55025]. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Rinaldi

Date: April 22, 2002

Joseph M. Farley Nuclear Plant

cc:

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