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Operated by Nuclear Management Company, LLC

NRC-02-017

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10 CFR 50.46

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Ladies/Gentlemen:

Docket 50-305
Operating License DPR-43
Kewaunee Nuclear Power Plant
2001 Annual Operating Report

Enclosed is the 2001 Kewaunee Nuclear Power Plant (KNPP) Annual Operating Report. This report is being submitted in accordance with Section 6.9.a.2 of the KNPP Technical Specifications.

This submittal of the 2001 KNPP Annual Operating Report also satisfies the reporting requirements of 10 CFR 50.46(a)(3)(ii) (Emergency Core Cooling System evaluation model changes), and KNPP Technical Specification 4.2.b.7.b (steam generator inspection). Also, in accordance with the commitment made by KNPP upon NRC issuance of the turbine valve test frequency Technical Specification amendment, any turbine stop and control valve failures are described.

Sincerely,

Thomas Coutu
Manager-Kewaunee Plant

dak
Enc.

cc - US NRC - Region III
US NRC Senior Resident Inspector
INPO Records Center
REIRS Project Manager, US NRC

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INTRODUCTION

This annual operating report is being submitted to fulfill several reporting requirements contained either in the Kewaunee Nuclear Power Plant (KNPP) Technical Specifications (TS) or in other commitments made by KNPP to the Nuclear Regulatory Commission (NRC).

In response to NUREG-0737, Item II.K.3.3, and in accordance with KNPP Technical Specification (TS) 6.9.a.2.C, Section 1.0 reports challenges to and failures of pressurizer safety and relief valves, if applicable.

Section 2.0 provides a summary of the steam generator eddy current examination in accordance with KNPP TS 4.2.b.7.b.

Personnel exposure and monitoring data is provided in Section 3.0 per Regulatory Guide 1.16, Section C.1.b.(3), and KNPP TS 6.9.a.2.B.

The provisions of 10 CFR 50.46 require the reporting of corrections or changes to the Emergency Core Cooling System (ECCS) evaluation models that are approved for use in performing the loss-of-coolant accident (LOCA) safety analysis. This information, if applicable, is provided in Section 4.0.

1.0 CHALLENGES TO AND FAILURES OF PRESSURIZER SAFETY AND RELIEF VALVES

In response to NUREG-0737, item II.K.3.3, and in accordance with KNPP Technical Specification 6.9.a.2.C, WPSC is committed to reporting challenges to and failures of pressurizer safety and pressurizer power-operated relief valves. There were no challenges to or failures of pressurizer safety or pressurizer power-operated relief valves during 2001.

2.0 SUMMARY OF THE SPRING 2001 STEAM GENERATOR EDDY CURRENT EXAMINATIONS

During 2001 Kewaunee replaced steam generators, therefore, no in-service examinations were performed on the old steam generators.

3.0 PERSONNEL EXPOSURE AND MONITORING REPORT

Table 3.1 presents a tabulation of the total number of individuals for whom monitoring was provided, along with information on total station dose for the year.

Table 3.2 presents a tabulation of the number of station, utility, and other personnel (including contractors) receiving exposures greater than 100 mrem/yr (1.0 mSv/yr) and their associated person-rem exposure according to work and job functions. This table is provided per Regulatory Guide 1.16, Section C.1.b.(3), and Kewaunee Technical Specification 6.9.a.2.B.

Table 3.1
Total Statistics
1/1/01 To 12/31/01

Deep Dose Exposure	Number of Individuals	
Range		in Range
None - Detected		665
Less then .100 rem		470
0.100 rem to 0.249 rem		359
0.250 rem to 0.499 rem		227
0.500 rem to 0.749 rem		41
0.750 rem to 0.999 rem		11
1.000 rem to 1.999 rem		2
Greater then 2.000 rem		0
 Total Monitored Individuals		 1775
 Total Site Deep Dose (NDE + DDE) =		 200.245 rem

TABLE 3.2
U.S.N.R.C. REGULATORY GUIDE 1.16 REPORT
KEWAUNEE NUCLEAR POWER PLANT
FROM 1/1/01 TO 12/31/01

Work and Job Function	Number of Persons > .100 Rem			Total Man-Rem		
	Station	Utility	Contract	Station	Utility	Contract

Inservice Inspection						
Maintenance Person.	11	0	38	0.264	0.000	3.695
Operating Personnel	0	0	1	0.000	0.000	0.025
Health Physics Per.	5	0	15	0.031	0.000	0.635
Supervisory Person.	1	0	0	0.001	0.000	0.000
Engineering Person.	3	0	0	0.239	0.000	0.000
Routine Maintenance						
Maintenance Person.	38	2	40	2.258	0.040	1.248
Operating Personnel	13	0	0	0.512	0.000	0.000
Health Physics Per.	19	0	62	3.504	0.000	13.889
Supervisory Person.	1	0	1	0.120	0.000	0.008
Engineering Person.	1	0	0	0.016	0.000	0.000
Reactor Operations & Surv						
Maintenance Person.	2	0	0	0.004	0.000	0.000
Operating Personnel	14	0	8	2.403	0.000	0.009
Health Physics Per.	0	0	0	0.000	0.000	0.000
Supervisory Person.	0	0	0	0.000	0.000	0.000
Engineering Person.	1	0	0	0.001	0.000	0.000
Refueling						
Maintenance Person.	28	2	15	3.261	0.556	1.644
Operating Personnel	3	0	8	0.369	0.000	2.429
Health Physics Per.	8	0	24	1.340	0.000	0.487
Supervisory Person.	1	0	1	0.162	0.000	0.007
Engineering Person.	2	0	0	0.446	0.000	0.000
Special Maintenance						
Maintenance Person.	41	3	459	7.875	0.441	116.385
Operating Personnel	2	0	3	0.149	0.000	0.317
Health Physics Per.	18	1	55	1.103	0.119	6.274
Supervisory Person.	1	0	3	0.119	0.000	0.315
Engineering Person.	3	0	7	0.336	0.000	1.735
Waste Processing						
Maintenance Person.	16	0	1	0.232	0.000	0.001
Operating Personnel	0	0	0	0.000	0.000	0.000
Health Physics Per.	14	0	14	0.952	0.000	0.178
Supervisory Person.	0	0	0	0.000	0.000	0.000
Engineering Person.	0	0	0	0.000	0.000	0.000
Sub Totals						
Maintenance Person.	136	7	553	13.894	1.037	122.973
Operating Personnel	32	0	20	3.433	0.000	2.780
Health Physics Per.	64	1	170	6.930	0.119	21.463
Supervisory Person.	4	0	5	0.402	0.000	0.330
Engineering Person.	10	0	7	1.038	0.000	1.735
Grand Totals	246	8	755	25.697	1.156	149.281

Total Site Deep Dose (NDE + DDE) for Ind. with >.100 rem = 177.406 rem

** INDIVIDUALS MAY BE LISTED UNDER MORE THAN ONE WORK AND JOB FUNCTION

4.0 CHANGES IN THE EMERGENCY CORE COOLING SYSTEM MODEL

The provisions of 10 CFR 50.46 require the reporting of corrections or changes to the emergency core cooling system (ECCS) models that are approved for use in performing the loss of coolant accident (LOCA) safety analysis.

As reported in the letter to the NRC, dated October 12, 2000, the peak clad temperature (PCT) for a large break LOCA and small break LOCA following steam generator replacement at the Kewaunee plant are:

PCT small break LOCA: 843 degrees F

PCT large break LOCA: 2038 degrees F

5.0 FAILURES OF TURBINE STOP AND CONTROL VALVES

There were no failures of the turbine stop and control valves during 2001.

6.0 MAXIMUM COOLANT ACTIVITY

KNPP TS 6.9.a.2.D requires the documentation of the results of specific activity analysis in which the reactor coolant exceeded the limits of TS 3.1.c.1.A during the past year.

The reactor coolant did not exceed the limits of TS 3.1.c.1.A during 2001.