

April 18, 2002

Mr. A. Christopher Bakken III, Senior Vice President  
and Chief Nuclear Officer  
Indiana Michigan Power Company  
Nuclear Generation Group  
500 Circle Drive  
Buchanan, MI 49107

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF  
AMENDMENTS (TAC NOS. MB4231 AND MB4232)

Dear Mr. Bakken:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 267 to Facility Operating License No. DPR-58 and Amendment No. 248 to Facility Operating License No. DPR-74 for the Donald C. Cook (D. C. Cook) Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated February 22, 2002.

The amendments would relocate TSs 3/4.9.6, "Refueling Operations – Manipulator Crane Operability" and TSs 3/4.9.7 "Refueling Operations - Crane Travel - Spent Fuel Storage Pool Building," with associated Bases to the D. C. Cook updated final safety analysis report.

A copy of our related safety evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

**/RA/**

John F. Stang, Senior Project Manager, Section 1  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosures: 1. Amendment No. 267 to DPR-58  
2. Amendment No. 248 to DPR-74  
3. Safety Evaluation

cc w/encls: See next page

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\*See previous concurrence.

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OFFICIAL RECORD COPY

Donald C. Cook Nuclear Plant, Units 1 and 2

cc:

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INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 267  
License No. DPR-58

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated February 22, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 267, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days, including the relocation to the updated final safety analysis as specified in the licensee's application dated February 22, 2002.

FOR THE NUCLEAR REGULATORY COMMISSION

**/RA/**

L. Raghavan, Chief, Section 1  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: April 18, 2002

ATTACHMENT TO LICENSE AMENDMENT NO. 267

TO FACILITY OPERATING LICENSE NO. DPR-58

DOCKET NO. 50-315

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

X

X

XIII

XIII

3/4 9-6

3/4 9-6

3/4 9-7

3/4 9-7

3/4 9-8

3/4 9-8

B 3/4 9-2

B 3/4 9-2

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 248

License No. DPR-74

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated February 22, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 248, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days, including the relocation to the updated final safety analysis as specified in the licensee's application dated February 22, 2002.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

L. Raghavan, Chief, Section 1  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: April 18, 2002



ATTACHMENT TO LICENSE AMENDMENT NO. 248

FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

X

X

XIII

XIII

3/4 9-6

3/4 9-6

3/4 9-7

3/4 9-7

B 3/4 9-2

B 3/4 9-2

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 267 TO FACILITY OPERATING LICENSE NO. DPR-58  
AND AMENDMENT NO. 248 TO FACILITY OPERATING LICENSE NO. DPR-74  
INDIANA MICHIGAN POWER COMPANY  
DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2  
DOCKET NOS. 50-315 AND 50-316

## 1.0 INTRODUCTION

By application dated February 22, 2002, the Indiana Michigan Power Company (the licensee) requested amendments to the Technical Specifications (TSs) for the Donald C. Cook Nuclear Plant, Units 1 and 2. The proposed amendments would relocate TSs 3/4.9.6, "Refueling Operations – Manipulator Crane Operability," and TSs 3/4.9.7 "Refueling Operations - Crane Travel - Spent Fuel Storage Pool Building," with associated Bases to the D. C. Cook updated final safety analysis report (UFSAR).

## 2.0 BACKGROUND

Section 182a of the Atomic Energy Act of 1954, as amended (the Act) requires applicants for nuclear power plant operating licenses to include the TSs as part of the license. The Commission's regulatory requirements related to the content of TSs are set forth in Section 50.36 of Title 10 of the Code of Federal Regulations (10 CFR). The regulation requires that the TSs include items in specific categories, including: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCOs); (3) surveillance requirements; (4) design features; and (5) administrative controls. The regulation does not specify the particular requirements to be included in the TSs.

The four criteria defined by 10 CFR 50.36(c)(2)(ii) for determining whether particular items are required to be included in the TS LCOs, are as follows:

- (1) installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary;
- (2) a process variable, design feature, or operating restriction that is an initial condition of a design-basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier;

- (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design-basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; and
- (4) a structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

Existing TS LCOs which fall within or satisfy any of the above criteria must be retained in the TSs; those which do not fall within or satisfy these criteria may be relocated to other licensee-controlled documents.

The licensee's submittal stated that the LCOs proposed for relocation do not meet any of the above criteria in 10 CFR 50.36 and that their relocation to the D. C. Cook UFSAR, which is controlled under the requirements of 10 CFR 50.59, would provide additional operational flexibility during refueling outages. The submittal also stated that the proposed changes are consistent with NUREG 1431, "Standard Technical Specifications, Westinghouse Plants," Revision 2, dated June 2001.

### 3.0 EVALUATION

#### 3.1 TS 3/4.9.6 Refueling Operations - Manipulator Crane Operability

TS 3/4.9.6 LCO defines the minimum capacity and overload cutoff limits for the manipulator crane and hoist, used for the movement of fuel assemblies and control rods, to be considered operable during the movement of fuel. The TS bases state that the operability requirements for the manipulator crane ensures that: 1) the manipulator cranes will be used for handling fuel assemblies and control rods; (2) each crane and hoist has sufficient load capacity to lift a control rod or fuel assembly and; (3) the core internals and pressure vessel are protected from excessive lifting force in the event that they are inadvertently engaged during lifting operations.

The relocation of the LCO requirements related to the manipulator crane operability is acceptable as it is not an LCO that satisfies the criteria of 10 CFR 50.36. Further, there is no accident analysis based on the minimum capacity and overload cutoff limits of the cranes.

The Nuclear Regulatory Commission (NRC) staff has reviewed the licensee's submittal and agrees with the licensee's conclusion that the Manipulator Crane LCO does not meet the criteria in 10 CFR 50.36(c)(2)(ii) requiring inclusion of this item as a TS LCO. Based on this review and the preceding evaluation, the staff finds it acceptable to relocate TS 3/4.9.6 to the UFSAR. Any changes to these requirements after relocation to the UFSAR will require a 10 CFR 50.59 evaluation. Under 10 CFR 50.59, sufficient regulatory controls exist to ensure continued protection of public health and safety.

#### 3.2 TS 3/4.9.7 Refueling Operations, Crane Travel - Spent Fuel Storage Building

TS 3/4.9.7 LCO prohibits loads in excess of 2500 pounds from travel over fuel assemblies in the storage pool. This restriction ensures that no more than the contents of one fuel assembly will rupture in the event of a fuel handling accident. The loading restrictions are

within the design capabilities of the associated cranes. The bounding accident analyses demonstrate acceptable radiological consequences even if a fuel assembly was dropped into the spent fuel pool. Consequently, the travel and loading restrictions of the cranes are not necessary to prevent or mitigate fuel handling accidents and, therefore, do not satisfy the TS LCO criteria of 10 CFR 50.36.

The NRC staff has reviewed the licensee's submittal and agrees with the licensee's conclusion that the Crane Travel - Spent Fuel Storage Building LCO does not meet the criteria in 10 CFR 50.36(c)(2)(ii) requiring inclusion of this item as a TS LCO. Based on this review and the preceding evaluation, the staff finds it acceptable to relocate TS 3/4.9.7 to the UFSAR. Any changes to these requirements after relocation to the UFSAR will require a 10 CFR 50.59 evaluation. Under 10 CFR 50.59, sufficient regulatory controls exist to ensure continued protection of public health and safety.

### 3.3 Index and Bases Pages

The licensee's proposed changes to the TS Index and Bases pages are consistent with the proposed relocation of TS Sections 3/4.9.6 and 3/4.9.7.

### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Michigan State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 5.0 ENVIRONMENTAL CONSIDERATION

These amendments change the requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change the surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (67 FR 12603, March 19, 2002). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Stang

Date: April 18, 2002