

February 22, 2002

MEMORANDUM TO: William H. Bateman, Chief  
Materials and Chemical Engineering Branch  
Division of Engineering

FROM: A. Louise Lund, Chief */ra/*  
Component Integrity & Chemical Engineering Section  
Materials and Chemical Engineering Branch  
Division of Engineering

SUBJECT: SUMMARY OF JANUARY 31, 2002, PUBLIC MEETING WITH THE  
NUCLEAR ENERGY INSTITUTE REGARDING THE GENERIC  
IMPLICATIONS OF THE THREE MILE ISLAND UNIT 1 (TMI-1) TUBE  
SEVER EVENT

On January 31, 2002, the Nuclear Regulatory Commission (NRC) staff met with representatives of the Nuclear Energy Institute (NEI) and the nuclear industry at the NRC's office in Rockville, Maryland to discuss the industry's plan for addressing the generic implications of the Three Mile Island Unit 1 (TMI-1) tube severance event. Attachment 1 (ML020420006) is a list of those attending the meeting.

Based on a request from the NRC staff, NEI provided the NRC an assessment of the implications of the TMI-1 tube severance event for once-through and recirculating steam generators by letter dated December 21, 2001, (ML020220355). To prepare for the January 31, 2002, meeting the NRC staff provided questions and comments to NEI in a letter dated January 25, 2002, (ML020250428).

At the meeting the industry discussed the TMI-1 and Oconee Unit 3 inspection results, industry assessment of the issue, industry actions taken to date, the industry action plan for addressing the issue, and a schedule for completion of these activities (Attachment 2 - ML020420009). As discussed in the meeting, the industry will be providing near-term guidance to the once-through steam generator licensees with spring 2002 outages to assist them with their steam generator tube inspections and repairs in response to this event. The industry agreed to provide a copy of this guidance to the NRC. Based on operating experience, engineering evaluations, and risk arguments, the industry concluded that near-term guidance is not required for recirculating steam generators.

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William H. Bateman

- 2 -

The industry is continuing to study this issue and will be developing an action plan in the next five months to address longer term actions that may be necessary for both once-through and recirculating steam generators. The industry will inform the NRC regarding any unexpected spring 2002 inspection results and any unexpected findings from the near-term activities in this area. At the end of the meeting the NRC provided a hand written question to NEI regarding the industry's risk analysis (Attachment 3). The NRC also requested clarification for the reason why various alloy 690 rolled plugs were repaired (refer to Slide 21 of Attachment 2).

Attachments: As stated

cc: Jim Riley, NEI

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Attachments: As stated

cc: Jim Riley, NEI

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Slide 29 indicates that the  $\Delta$ LERF increment is about 10% of the  $\Delta$ CDF increment estimated to be attributable to severed tubes. Explain the factors that exclude 90% of the  $\Delta$ CDF from the LERF category, including both the definition used for large early release and the manner in which the physical attributes of the sequences that were not counted as LERF did not meet the criteria for the LERF category.