



FirstEnergy Nuclear Operating Company

Davis-Besse Nuclear Power Station
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Docket Number 50-346

License Number NPF-3

Serial Number 2764

February 12, 2002

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555-0001

Subject: Supplemental Information Regarding License Amendment Application to
Revise Technical Specification (TS) 3/4.4.5 Steam Generator Tube Repair Roll
Requirements (License Amendment Request No. 01-0004; TAC No. MB2107)

Ladies and Gentlemen:

On May 22, 2001, the FirstEnergy Nuclear Operating Company (FENOC) submitted an application for an amendment to the Davis-Besse Nuclear Power Station (DBNPS), Unit Number 1, Operating License Number NPF-3, Appendix A Technical Specifications, regarding Technical Specification (TS) 3/4.4.5, Steam Generators. The proposed amendment (DBNPS letter Serial Number 2705) would revise the once-through steam generator tube repair roll requirements. This letter provides supplemental information to the license amendment application.

During a discussion between the NRC and DBNPS staffs on February 5, 2002, the DBNPS staff was requested to address the relationship between Topical Report BAW-2374, *Justification for Not Including Postulated Breaks in Large-Bore Reactor Coolant System Piping in the Licensing Basis for Existing and Replacement Once-Through Steam Generators*, dated July 2000, and the license amendment application submitted to the NRC under DBNPS letter Serial Number 2705, dated May 22, 2001.

By letter dated July 7, 2000, the Babcock and Wilcox Owners' Group (BWOG), of which the DBNPS is a member, submitted Topical Report BAW-2374, dated July 2000 (reference: BWOG Letter OG-1792), to the NRC. By letter dated November 27, 2000, the BWOG provided additional information to the NRC regarding this topical report. This topical report, submitted on behalf of the DBNPS, provides the technical basis for excluding the effects of large-break LOCA induced thermal and pressure loads on OTSG tube repair rolls and provides for the acceptability of basing OTSG thermal loads on the

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previously analyzed limiting accidents, which are a loss of coolant accident in reactor coolant system attached pipes and a main steam line break.

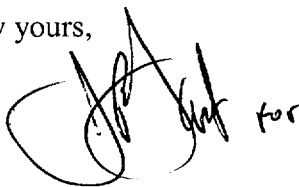
By letter dated March 12, 2001, the BWOOG submitted Topical Report BAW-2374, Revision 1, *Risk-Informed Assessment of Once-Through Steam Generator Tube Thermal Loads Due to Breaks in Reactor Coolant System Upper Hot Leg Large-Bore Piping*, dated March 2001. This topical report was referenced as supporting the technical basis for the license amendment application submitted under DBNPS Serial Number 2705, dated May 22, 2001. Topical Report BAW-2374, Revision 1, is currently being reviewed by the NRC staff.

Topical Report BAW-2374, dated July 2000, provides, the risk-informed basis for the proposed repair roll design specified in Topical Report BAW-2303P, Revision 4, *OTSG Repair Roll Qualification Report*, which was referenced in the license amendment application submitted under DBNPS letter Serial Number 2705. Accordingly, since Topical Report BAW-2374, Revision 1, remains under NRC staff review, Topical Report BAW-2374, dated July 2000, submitted on the behalf of the DBNPS, provides the risk-informed basis for the design specified in Topical Report BAW-2303P, Revision 4, that, in turn, provides the technical basis for this license amendment application.

This information does not affect the conclusions stated in the previously submitted license amendment application that there is no adverse impact on nuclear safety and that the proposed amendment involves no significant hazards consideration.

Should you have any questions or require additional information, please contact Mr. David H. Lockwood, Manager - Regulatory Affairs, at (419) 321-8450.

Very truly yours,

A handwritten signature in black ink, appearing to read 'D. Lockwood', with a large circular flourish to the left.

MAR/s

Enclosures

cc: J. E. Dyer, Regional Administrator, NRC Region III
S. P. Sands, NRC/NRR Project Manager
D. J. Shipley, Executive Director, Ohio Emergency Management Agency,
State of Ohio (NRC Liaison)
C. S. Thomas, NRC Region III, DB-1 Senior Resident Inspector
Utility Radiological Safety Board

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Enclosure 1

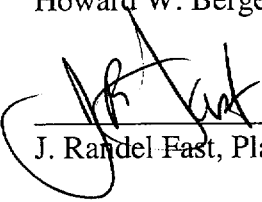
SUPPLEMENTAL INFORMATION IN SUPPORT OF
APPLICATION FOR AMENDMENT
TO
FACILITY OPERATING LICENSE NPF-3
DAVIS-BESSE NUCLEAR POWER STATION
UNIT NUMBER 1

This letter contains supplemental information for Davis-Besse Nuclear Power Station (DBNPS), Unit Number 1 Facility Operating License Number NPF-3, License Amendment Request Number 01-0004 (DBNPS letter Serial Number 2705, dated May 22, 2001).

I, Howard W. Bergendahl, state that (1) I am Vice President - Nuclear of the FirstEnergy Nuclear Operating Company, (2) I am duly authorized to execute and file this certification on behalf of the Toledo Edison Company and The Cleveland Electric Illuminating Company, and (3) the statements set forth herein are true and correct to the best of my knowledge, information and belief.

For: Howard W. Bergendahl, Vice President - Nuclear

By:



J. Randel Fast, Plant Manager

Affirmed and subscribed before me this twelfth day of February, 2002.



Notary Public, State of Ohio

Laura A. Jennison - My Commission Expires August 16, 2006.

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Enclosure 2

COMMITMENT LIST

THE FOLLOWING LIST IDENTIFIES THOSE ACTIONS COMMITTED TO BY THE DAVIS-BESSE NUCLEAR POWER STATION (DBNPS) IN THIS DOCUMENT. ANY OTHER ACTIONS DISCUSSED IN THE SUBMITTAL REPRESENT INTENDED OR PLANNED ACTIONS BY THE DBNPS. THEY ARE DESCRIBED ONLY FOR INFORMATION AND ARE NOT REGULATORY COMMITMENTS. PLEASE NOTIFY THE MANAGER – REGULATORY AFFAIRS (419-321-8450) AT THE DBNPS OF ANY QUESTIONS REGARDING THIS DOCUMENT OR ANY ASSOCIATED REGULATORY COMMITMENTS.

COMMITMENTS

DUE DATE

None

N/A