

Exelon Generation Company, LLC
Dresden Nuclear Power Station
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Nuclear

10 CFR 50.4
10 CFR 50.55a

February 5, 2002

PSLTR #02-0009

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

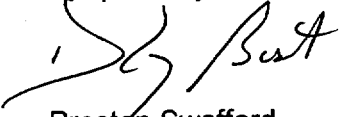
Dresden Nuclear Power Station, Unit 2
Facility Operating License No. DPR-19
NRC Docket No. 50-237

Subject: Inservice Inspection (ISI) Summary Report
Fall 2001 Inservice Inspection Period

Enclosed is the Dresden Nuclear Power Station (DNPS) Unit 2 Post-Outage (90 day) Summary Report for ISI examinations and repair/replacement activities conducted from January 12, 2000 to January 21, 2002. Unit 2 completed its seventeenth refueling outage (D2R17) on November 7, 2001. This report has been submitted to you in accordance with the requirements of American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," Article IWA-6200.

Should you have any questions concerning this letter, please contact Mr. D. F. Ambler, Regulatory Assurance Manager, at (815) 416 - 2800.

Respectfully,



Preston Swafford
Site Vice President
Dresden Nuclear Power Station

Attachment

cc: Regional Administrator – Region III
NRC Senior Resident Inspector, Dresden Station

A047

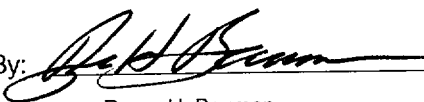
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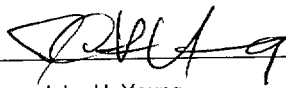
October 2001 Inservice Inspection
Unit No. 2; National Board No. N-137
Commercial Service Date: 6-9-72


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Section I

Introduction

The seventeenth Inservice Inspection (ISI) of Dresden Unit 2 was performed during the D2R17 outage which began on October 20, 2001 and was completed on November 7, 2001. This was the second of two scheduled refuel outages in the third inspection period of the unit's 3rd 10-year ISI Inspection Interval which commenced on March 1, 1992. The third inspection period commenced on October 1st, 1999 and is currently scheduled to end on January 19, 2003. In addition to examinations performed during D2R17, a large number of examinations were performed during on-line operation of Unit 2. This report contains all examinations completed during D2R17 and on line examinations which were not included in the previous Unit 2 Summary Report (dated January 19, 2000).

General Electric was contracted to perform the non-destructive examinations and reactor vessel visual examinations during the refuel outage. Dresden Engineering Programs Group performed the remaining visual and NDE examinations during D2R17 as well as the on line examinations.

The Authorized Nuclear Inservice Inspector's (ANII) services were provided by The Hartford Steam Boiler Inspection Insurance Company of Connecticut. The ANII reviewed procedures, personnel qualifications, instrument and material certifications, and examination results.

All examinations were performed in accordance with the Dresden Station ISI Program, Dresden Station Technical Requirements Manual, the ASME Boiler and Pressure Vessel Code, Section XI, 1989 Edition and 1998 Edition (implemented in accordance with Relief Request MCR-02 for Containment ISI), Generic Letter 88-01, BWRVIP-75, and In-Vessel Visual Examinations in accordance with BWRVIP Inspection and Evaluation Guidelines.

A list of abbreviations used throughout this report can be found in Section IV of this report.

FORM NIS-1 OWNER'S REPORT FOR INSERVICE INSPECTIONS
As required by the Provisions of the ASME Code Rules

1. Owner Exelon Nuclear, P.O. Box 805379, Chicago, IL 60680-5379
(Name and Address of Owner)
2. Plant Dresden Nuclear Power Station, 6500 N. Dresden Road, Morris, IL 60450
(Name and Address of Owner)
3. Plant Unit Two 4. Owner Certificate of Authorization (if required) N/A
5. Commercial Service Date 6-9-72 6. National Board Number for Unit N-137
7. Components Inspected See Section II of this report (report is 85 total pages).

Component or Appurtenance	Manufacturer Or Installer	Manufacturer or Installer Serial No.	State or Province No.	National Board No.
Reactor Vessel	Babcock & Wilcox, Barberton, Ohio	610-0098-51-52	B0082800	N-137
Class 1 & 2 Systems	General Electric-APED Morris, IL	N/A	N/A	N/A

Note: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11½ in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-1 (Back)

8. Examination Dates: 1/12/2000 to 1/21/2002

9. Inspection Period Identification : 3rd Period - From 10/1/1999 to 1/19/2003

10. Inspection Interval Identification : 3rd Interval - From 3/1/1992 to 1/19/2003

11. Applicable Edition of Section XI 1989 Addenda N/A
1998* Addenda N/A

* 1998 edition implemented in accordance with Relief Request MCR-02 for Containment Inservice Inspection.

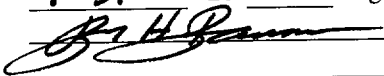
12. Date/Revision of Inspection Plan: 6/1/2001 Rev.6

13. Abstract of Examination and Tests. Include a list of examinations and tests and a statement concerning status of work required for the Inspection Plan. See Attached Sections II and III.

14. Abstract of Results of Examinations and Tests. See Attached Sections II and III.


15. Abstract of Corrective Measures. See Attached Sections II and III.

We certify that a) the statement made in this report are correct, b) the examinations and tests meet Inspection Plan as required by the ASME Code, Section XI, and c) corrective measures taken conform to the ASME Code, Section XI.

Certificate of Authorization No. (if applicable) N/A Expiration Date N/A
Date 1-31 20 02 Signed Exelon Nuclear Dresden Nuclear Power Station.
By  Dresden Station ISI Coordinator
Owner

CERTIFICATE OF INSERVICE INSPECTION

I, the under signed, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois And employed by Hartford Steam Boiler of CT have inspected the components described in the Owner's Report during the period 1/12/2000 To 1/21/2002, and state that to the best of my knowledge and belief, the Owner has performed examinations and tests and taken corrective measures described in this Owner's Report in accordance with the Inspection Plan and as required by the ASME Code, Section XI. By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations, tests, and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.


Inspector's Signature

Commissions NB7742NISB, IL932
National Board, State, Province, and Endorsements

Date 2-1 20 02

Section II

Scope of Inspection

Abstract of ISI and Augmented Examinations

ISI and Augmented Examinations

Table A contains a list of components examined prior to, during and after the D2R17 refuel outage, to satisfy the requirements of the Dresden Station ISI Program, Dresden Station Technical Requirements Manual, the ASME Boiler and Pressure Vessel Code, Section XI, 1989 Edition and 1998 Edition (implemented in accordance with Relief Request MCR-02 for Containment ISI), Generic Letter 88-01 and BWRVIP-75. Those items which were examined and meet the acceptance standards of ASME Section XI or Generic Letter 88-01 are identified as "Acceptable" under the results column. Those items that required further evaluation are identified with "Section III" in the results column and are further discussed in Section III of this report.

Dresden Station requested deferral of thirty-three weld overlays which do not meet the requirements of Open Item 3.4 of BWRVIP-75 with regard to the number of inspections required prior to implementing BWRVIP-75 inspection frequencies. This deferral was approved by the NRC permitting deferral until October 2003 or until completion of the NRC staff review and approval of the BWRVIP-75 report, whichever comes first.

Snubber Examinations (Technical Requirements Manual 3.7.H)

All Section XI Class 1, 2 and 3 safety-related snubbers are visually (VT-3) examined in accordance with Dresden Technical Requirements Manual 3.7.H. A sample population of snubbers are functionally tested every outage. Table A includes all the snubbers functionally tested during D2R17 and any visual examined snubbers that required further evaluation. Snubbers that required further evaluation are identified with "Section III" in the results column and are further discussed in Section III of this report.

Summary of Reactor Vessel Weld (Category B-A) Examinations

The Unit 2 reactor vessel shell welds are examined as required by 10CFR50.55(a) and ASME Section XI. These welds are addressed under Examination Category B-A, Item numbers B1.11, Circumferential Shell Welds; Item B1.12, Longitudinal Sheel Welds; and Item B1.30, Head to Flange Weld. Two relief requests were associated with these inspections. First, relief was requested, and subsequently granted, from requirements to examine welds under item B1.11. The technical basis for this request is established in BWRVIP-05. Second, a schedule exemption was requested, and subsequently approved, to allow performance of these inspections over an additional refueling outage. Otherwise, the entire inspection would have been required during D2R17, the last outage in the third inspection interval. The basis of this request was to allow the station to attempt to take advantage of a new inspection technology, the AIRIS 21 system, to achieve an increased coverage without significantly impacting the outage.

Summary of Vessel Interior Examinations

Attachment A contains a summary of examinations performed to satisfy the requirements of ASME Section XI categories B-N-1, B-N-2, and various special examination requirements. Details of the examinations, results, and corrective measures are included.

Section II Scope of Inspection

Abstract of ISI and Augmented Examinations

Current Interval Status

As of the date of this report, the percentages listed in the table below represent the status of inspections for the 3rd inservice inspection interval. All remaining examinations, with the exception of reactor vessel shell welds are scheduled to be completed prior to the end of the inspection interval. The remaining reactor vessel shell welds (Examination Category B-A) are scheduled to be completed during refueling outage D2R18 at the start of the 4th inservice inspection interval in accordance with an approved exemption request.

Examination Category	Deferral is permissible in accordance with Table IWX-2500-1	Inspected in accordance with Inspection Program B
B-A	73%	100%
B-D	N/A	100%
B-E	100%	N/A
B-G-1	80%	N/A
B-G-2	100% ¹	100%
B-M-1	0%	N/A
B-M-2	100% ¹	N/A
B-N-1	N/A	100%
B-N-2	100%	N/A
B-O	100%	N/A
C-A	N/A	50%
C-B	N/A	83% ²
C-C	N/A	64%
D-B	N/A	78%
F-A	N/A	80%
R-A	N/A	85% ³

1. Percentage represents components subject to inspection only when disassembled for maintenance, repair, or volumetric examination.
2. Percentage does not include components subject to VT-2 examination each inspection period. VT-2 examinations during the 3rd inspection period are scheduled for completion prior to the end of the inspection interval.
3. Percentage does not include socket welds subject to VT-2 examination each inspection period. VT-2 examinations during the 3rd inspection period are complete.

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October 2001 Inservice Inspection
Unit No. 2; National Board No. N-137
Commercial Service Date: 6-9-92

Section II

Scope of Inspection

Abstract of ISI and Augmented Examinations

Category	Item	Augment	System	Line	Component	Type	Exam	Credit	Results
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC1A-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC1B-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC1D-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC2D-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC3A-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC3B-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC3C-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC3D-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC3E-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC4A-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC4B-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC4C-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC4D-VERT	LONG	UT	XI	Acceptable
B-A	B1.12	N/A	RPV	RPV SHELL	2-SC4E-VERT	LONG	UT	XI	Acceptable
B-A	B1.30	N/A	RPV	RPV SHELL	2-SC4-FLGA	RPV-FLG	UT	XI	Acceptable
B-A	B1.30	N/A	RPV	RPV SHELL	2-SC4-FLGB	RPV-FLG	UT	XI	Acceptable
B-A	B1.30	N/A	RPV	RPV SHELL	2-SC4-FLGC	RPV-FLG	UT	XI	Acceptable
B-A	B1.30	N/A	RPV	RPV SHELL	2-SC4-FLGD	RPV-FLG	UT	XI	Acceptable
B-A	B1.30	N/A	RPV	RPV SHELL	2-SC4-FLGE	RPV-FLG	UT	XI	Acceptable
B-A	B1.30	N/A	RPV	RPV SHELL	2-SC4-FLGF	RPV-FLG	UT	XI	Acceptable
B-D	B3.100	N/A	RPV	RPV SHELL	N3A-1	NIR	UT	XI	Acceptable
B-D	B3.100	N/A	RPV	RPV SHELL	N3B-1	NIR	UT	XI	Acceptable
B-D	B3.100	N/A	RPV	RPV SHELL	N3C-1	NIR	UT	XI	Acceptable
B-D	B3.100	N/A	RPV	RPV SHELL	N3D-1	NIR	UT	XI	Acceptable
B-D	B3.100	N/A	RPV	RPV SHELL	N5A-1	NIR	UT	XI	Acceptable
B-D	B3.100	N/A	RPV	RPV UPP HD	N18A-1	NIR	UT	XI	Acceptable
B-D	B3.100	N/A	RPV	RPV UPP HD	N18B-1	NIR	UT	XI	Acceptable
B-D	B3.100	N/A	RPV	RPV UPP HD	N8-1	NIR	UT	XI	Acceptable
B-D	B3.90	N/A	RPV	RPV LWR HD	N12-2	NOZ-RPV	UT	XI	Acceptable
B-D	B3.90	N/A	RPV	RPV SHELL	N3A-2	RPV-NOZ	UT	XI	Acceptable
B-D	B3.90	N/A	RPV	RPV SHELL	N3B-2	RPV-NOZ	UT	XI	Acceptable
B-D	B3.90	N/A	RPV	RPV SHELL	N3C-2	RPV-NOZ	UT	XI	Acceptable
B-D	B3.90	N/A	RPV	RPV SHELL	N3D-2	RPV-NOZ	UT	XI	Acceptable
B-D	B3.90	N/A	RPV	RPV SHELL	N5A-2	RPV-NOZ	UT	XI	Acceptable
B-D	B3.90	N/A	RPV	RPV UPP HD	N18A-2	NOZ-RPV	UT	XI	Acceptable
B-D	B3.90	N/A	RPV	RPV UPP HD	N18B-2	RPV-NOZ	UT	XI	Acceptable
B-D	B3.90	N/A	RPV	RPV UPP HD	N8-2	RPV-NOZ	UT	XI	Acceptable
B-G-1	B6.10	N/A	RPV	RPV UPP HD	HD NUTS (92)	FLGBLT	VT-1	XI	Acceptable
B-G-1	B6.20	N/A	RPV	RPV UPP HD	HD STUDS IN PLC (92)	FLGBLT	UT	XI	Acceptable
B-G-1	B6.50	N/A	RPV	RPV UPP HD	WSHR/BSHG (92)	FLGBLT	VT-1	XI	Acceptable

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Commercial Service Date: 6-9-72

Section II

Scope of Inspection

Abstract of ISI and Augmented Examinations

Category	Item	Augment	System	Line	Component	Type	Exam	Credit	Results
B-G-2	B7.50	N/A	MSA	3001A-6	SV-2-203-4A	FLGBLT	VT-1	XI	Acceptable
B-G-2	B7.50	N/A	MSA	3001A-6	SV-2-203-4B	FLGBLT	VT-1	XI	Acceptable
B-G-2	B7.50	N/A	MSC	3001C-6	ERV-2-203-3C	FLGBLT	VT-1	XI	Acceptable
B-G-2	B7.50	N/A	MSC	3001C-6	SV-2-203-4E	FLGBLT	VT-1	XI	Acceptable
B-G-2	B7.50	N/A	MSC	3001C-6	SV-2-203-4F	FLGBLT	VT-1	XI	Acceptable
B-G-2	B7.50	N/A	MSD	3001D-6	ERV-2-203-3D	FLGBLT	VT-1	XI	Acceptable
B-G-2	B7.50	N/A	MSD	3001D-6	SV-2-203-4G	FLGBLT	VT-1	XI	Acceptable
B-G-2	B7.50	N/A	MSD	3001D-6	SV-2-203-4H	FLGBLT	VT-1	XI	Acceptable
B-M-2	B12.50	N/A	MSA	3001A-6	TRV-2-203-3A	VLV	VT-3	XI	Acceptable
B-N-1	B13.10	N/A	RPV	RPV SHELL	VESSEL INT	RPV	VT-3	XI	See Attachment A
B-N-2	B13.20	N/A	RPV	RPV SHELL	IN-BELTLINE ATT	IWA	VT-1	XI	See Attachment A
B-N-2	B13.30	N/A	RPV	RPV SHELL	OUT-BELTLINE AT	IWA	VT-3	XI	See Attachment A
B-N-2	B13.40	N/A	RPV	RPV SHELL	CORE SUPPORT	IWA	VT-3	XI	See Attachment A
B-O	B14.10	N/A	RPV	RPV LWR HD	M2-4607	P-FLG	PT	XI	Acceptable
B-P	B15.XX	N/A	RR	TEST BLOCK	2RC01	N/A	VT-2	XI	See Section III
B-P	B15.XX	N/A	SBLC	TEST BLOCK	2SC01	N/A	VT-2	XI	Acceptable
C-B	C2.21	N/A	ISCOCR	1303A-8	8-9	SHL-NOZ	MT UT	XI	Acceptable
C-B	C2.21	N/A	ISCOCR	1303B-8	8-8	SHL-NOZ	MT UT	XI	Acceptable
C-B	C2.21	N/A	ISCOSS	1302A-12	12-9	NOZ-SHL	MT UT	XI	Acceptable
C-B	C2.21	N/A	ISCOSS	1302B-12	12-8	NOZ-SHL	MT UT	XI	Acceptable
C-B	C2.31	N/A	ECCS	1501-20	20-6	SDL-SHL	MT	XI	Acceptable
C-B	C2.31	N/A	ECCS	1501-20	20-8	SDL-SHL	MT	XI	Acceptable
C-C	C3.20	N/A	ECCS	1501-24	M-3202-01	IWA	MT	XI	Acceptable
C-C	C3.20	N/A	ECCS	1501-24	M-3202-02	IWA	MT	XI	Acceptable
C-C	C3.20	N/A	ECCS	1501-24	M-3202-09	IWA	MT	XI	Acceptable
C-C	C3.20	N/A	ECCS	1501-24	M-3202-10	IWA	MT	XI	Acceptable
C-C	C3.20	N/A	ECCS	1501-24	M-3202-11	IWA	MT	XI	Acceptable
C-C	C3.20	N/A	ECCS	1501-24	M-3202-12	IWA	MT	XI	Acceptable
C-C	C3.20	N/A	HPCIPD	2304-14	M-1151D-10	IWA	MT UT	XI	See Section III
C-C	C3.20	N/A	ISCOSS	1302-14	M-1163D-261	IWA	PT	XI	Acceptable

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Section II

Scope of Inspection

Abstract of ISI and Augmented Examinations

Category	Item	Augment	System	Line	Component	Type	Exam	Credit	Results
C-H	C7.ST	N/A	HPCI	TEST BLOCK	2HP03	N/A	VT-2	XI	Acceptable
C-H	C7.XX	N/A	HPCI	TEST BLOCK	2HP01	N/A	VT-2	XI	Acceptable
C-H	C7.XX	N/A	RPV	TEST BLOCK	2NB01	N/A	VT-2	XI	Acceptable
C-H	C7.XX	N/A	RR	TEST BLOCK	2RC01	N/A	VT-2	XI	See Section III
C-H	C7.XX	N/A	SBLC	TEST BLOCK	2SC01	N/A	VT-2	XI	Acceptable
C-H	C7.XX	N/A	SBLC	TEST BLOCK	2SC02	N/A	VT-2	XI	Acceptable
C-H	C7.XX	N/A	SBLC	TEST BLOCK	2SC03	N/A	VT-2	XI	Acceptable
D-B	D2.IA	N/A	SRVDA	3019A-12	M-1177D-1	IWA	VT-3	XI	Acceptable
D-B	D2.IA	N/A	SRVDB	3019B-12	M-1177D-4	IWA	VT-3	XI	Acceptable
D-B	D2.IA	N/A	SRVDC	3019C-12	M-1177D-7	IWA	VT-3	XI	Acceptable
D-B	D2.IA	N/A	SRVDD	3019D-12	M-1177D-10	IWA	VT-3	XI	Acceptable
D-B	D2.IA	N/A	SRVDE	3019E-12	M-1177D-13	IWA	VT-3	XI	Acceptable
D-B	D2.XX	N/A	ISCO	TEST BLOCK	2IC01	N/A	VT-2	XI	Acceptable
D-B	D2.XX	N/A	MS	TEST BLOCK	2MS01	N/A	VT-2	XI	Acceptable
E-A	E1.11	N/A	CONT	N/A	DW 502 LINER	DW LINER	GV	XI	See Section III
E-A	E1.12	N/A	CONT	N/A	TOR B10 SUB	TOR LINER	VT-1	BL	See Section III
E-A	E1.12	N/A	CONT	N/A	TOR B14 SUB	TOR LINER	VT-1	BL	See Section III
E-A	E1.30	N/A	CONT	N/A	DW BASE MBARR	MBARR	GV	XI	See Section III
F-A	F1.10	N/A	MSC	3001C -20	M-564G SHT 1	CL 1 SNB	F.TEST	OTHR	Acceptable
F-A	F1.10	N/A	MSD	3001D-20	M-564H SHT 1	CL 1 SNB	F.TEST	OTHR	Acceptable
F-A	F1.10	N/A	RHS	0304-2.5	M-1167D-1	CL 1 SUP	VT-3	AD	See Section III
F-A	F1.10	N/A	RHS	0304-2.5	M-1167D-2	CL 1 SUP	VT-3	XI	See Section III
F-A	F1.10	N/A	RHS	0304-2.5	M-1167D-3	CL 1 SUP	VT-3	AD	See Section III
F-A	F1.20	N/A	CSBD	1404-12	M-3209-20 (1/3)	CL 2 SNB	F.TEST	OTHR	Acceptable
F-A	F1.20	N/A	ECCS	1501-24	M-3202-02	CL 2 SNB	VT-3/4	XI	Acceptable
F-A	F1.20	N/A	ECCS	1501-24	M-3202-09	CL 2 SNB	VT-3/4	XI	Acceptable
F-A	F1.20	N/A	ECCS	1501-24	M-3202-23	CL 2 SNB	VT-3/4	XI	Acceptable
F-A	F1.20	N/A	ECCS	1501-24	M-3202-26	CL 2 SNB	F.TEST	OTHR	Acceptable
F-A	F1.20	N/A	HPCISS	2305-10	2305-G-201	CL 2 SUP	VT-3	XI	Acceptable
F-A	F1.20	N/A	HPCISS	2305-10	M-1151D-103	CL 2 SUP	VT-3	XI	Acceptable
F-A	F1.20	N/A	HPCISS	2305-10	M-1151D-132	CL 2 SUP	VT-3	XI	Acceptable
F-A	F1.20	N/A	HPCISS	2305-10	M-1151D-277	CL 2 SUP	VT-3	XI	Acceptable
F-A	F1.20	N/A	HPCISS	2305-10	M-1151D-292	CL 2 SUP	VT-3	XI	Acceptable
F-A	F1.20	N/A	HPCISS	2305-10	M-1151D-294	CL 2 SUP	VT-3	XI	Acceptable
F-A	F1.20	N/A	HPCITE	2306-24	M-3212-06	CL 2 SUP	VT-3/4	XI	Acceptable
F-A	F1.20	N/A	ISCOSS	1302B-12	M-1163D-82	CL 2 SUP	VT-3	XI	Acceptable
F-A	F1.20	N/A	LPCIAD	1504-16	M-3213-12	CL 2 SNB	F.TEST	OTHR	Acceptable
F-A	F1.20	N/A	LPCIBD	1509-18	M-3214-06	CL 2 SUP	VT-3	XI	Acceptable
F-A	F1.20	N/A	LPCITR	1517-14	M-3208-06 (2/2)	CL 2 SNB	F.TEST	OTHR	Acceptable
F-A	F1.30	N/A	SRVDC	3019C-8	M-564G SHT 8	CL 3 SNB	F.TEST	OTHR	Acceptable

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Section II

Scope of Inspection

Abstract of ISI and Augmented Examinations

Category	Item	Augment	System	Line	Component	Type	Exam	Credit	Results
F-A	F1.40	N/A	RPV	RPV LWR HD	M-1175D-5	CL 1 SUP	VT-3	XI	Acceptable
F-A	F1.40	N/A	RRAS	PMP 2A-0202	M-1135 SHT 13	CL 1 SNB	VT-3	OTHR	See Section III
F-A	F1.40	N/A	RRAS	PMP 2A-0202	M-1135 SHT 8	CL 1 SNB	F.TEST	OTHR	Acceptable
F-A	F1.40	N/A	RRBS	PMP 2B-0202	M-1135 SHT 10	CL 1 SNB	F.TEST	OTHR	Acceptable
F-A	F1.40	N/A	RRBS	PMP 2B-0202	M-1135 SHT 15	CL 1 SNB	F.TEST	OTHR	Acceptable
R-A	R1.11	N/A	FW	3204B-18	18-1	VLV-TEE	MT UT-E	XI	Acceptable
R-A	R1.11	N/A	HPCI	2306-24	24-16	EL-P	MT UT-E	XI	Acceptable
R-A	R1.11	N/A	MS	3007-2	MSD2-1(A)	SWR-SWT	VT-2	XI	Acceptable
R-A	R1.11	N/A	MS	3007-2	MSD2-14	SWV-P	VT-2	XI	Acceptable
R-A	R1.11	N/A	MS	3007-2	MSD2-18	P-SWE	VT-2	XI	Acceptable
R-A	R1.11	N/A	MS	3007-2	MSD2-24	P-SWV	VT-2	XI	Acceptable
R-A	R1.11	N/A	MS	3007-2	MSD2-3(A)	SWT-P	VT-2	XI	Acceptable
R-A	R1.11	N/A	MS	3007-2	MSD2-9(A)	SWT-P	VT-2	XI	Acceptable
R-A	R1.11	N/A	RPV	UVLB	UVLB2-2	P-SWR	VT-2	XI	Acceptable
R-A	R1.11	N/A	RVBD	1265-2	2-10	P-SWE	VT-2	XI	Acceptable
R-A	R1.11	N/A	SBLC	1102-1.5	SLC1.5-10	SWT-P	VT-2	XI	Acceptable
R-A	R1.11	N/A	SBLC	1102-1.5	SLC1.5-11	RED-SWT	VT-2	XI	Acceptable
R-A	R1.11	N/A	SDC	1001A-16	16-2B	P-EL	MT UT-E	XI	Acceptable
R-A	R1.11	N/A	SDC	1001B-16	16-2B	P-P	MT UT-E	XI	Acceptable
R-A	R1.20	GL88-01 A	HPCI	2305-14	N5A-3	NOZ-SE	UT-E	XI	Acceptable
R-A	R1.20	N/A	MS	3001A-20	20-K6	EL-P	MT UT-E	XI	Acceptable
R-A	R1.20	N/A	MS	3001B-20	20-K10	P-EL	MT UT-E	XI	Acceptable
R-A	R1.20	N/A	MS	3001D-20	20-K10	P-EL	MT UT-E	XI	Acceptable
R-A	R1.20	GL88-01 A	RR	0203A-4	SPM-45-25(A)	SWP-CAP	UT-E	XI	Acceptable
R-A	R1.20	GL88-01 A	RR	0203A-4	SPM-45-7(A)	SWP-RED	UT-E	XI	Acceptable
R-A	R1.20	N/A	SBLC	1102-1.5	SLC1.5-31	SWE-P	VT-2	XI	Acceptable
R-A	R1.20	N/A	SBLC	1102-1.5	SLC1.5-32	P-SWE	VT-2	XI	Acceptable
R-A	R1.20	N/A	SBLC	1102-1.5	SLC1.5-39	SWE-P	VT-2	XI	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302-14	14-6	P-P	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302-14	14-7	P-TEE	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302-14	14-8	TEE-RED	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302-14	14-9	TEE-RED	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302A-12	12-1	RED-P	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302A-12	12-2	P-P	UT	AG	Acceptable

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Scope of Inspection

Abstract of ISI and Augmented Examinations

Category	Item	Augment	System	Line	Component	Type	Exam	Credit	Results
N/A	N/A	GL88-01 C	ISCO	1302A-12	12-3	P-EL	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302A-12	12-4	EL-P	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302A-12	12-4A	P-P	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302A-12	12-5	P-EL	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302A-12	12-6	EL-P	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302A-12	12-7	P-SE	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302B-12	12-1	RED-P	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302B-12	12-2	P-EL	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302B-12	12-3	EL-P	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302B-12	12-3A	P-P	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302B-12	12-4	P-EL	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302B-12	12-5	EL-P	UT	AG	Acceptable
N/A	N/A	GL88-01 C	ISCO	1302B-12	12-6	P-SE	UT	AG	Acceptable
N/A	N/A	GL88-01 F	RR	0201A-28	PD1A-D14	EL-P	UT	AG	Acceptable
N/A	N/A	GL88-01 F	RR	0202B-28	PS2-TEE/202-4B	TEE-VLV	UT	AG	Acceptable

Section II Scope of Inspection

Attachment A Summary of Vessel Interior Examinations

Examination Category B-N-1

Item B13.10, reactor vessel interior surfaces, were VT-3 examined between the closure flange and the top of the shroud over 360° of the reactor vessel circumference. This exam is required each period. A lower steam separator alignment guide rod at 200° was found bent similar to what was identified during D3R16 at 20°. All of the upper and lower guide rod sets were examined in this scope. No other recordable indications were noted.

Examination Category B-N-2

Item B13.20, "Interior Attachments within the Beltline", Accessible Welds, were VT-1 examined as required this interval. These examinations are required to be performed once each interval. All component attachments within the beltline were examined this outage and included the upper and lower vessel metal surveillance coupon mounting brackets at all six locations and the twenty Jet Pump riser brace attachments. No recordable indications were noted at the attachment points to the reactor vessel. The upper leaf on the Jet Pump number nine side of the riser brace to Jet Pumps nine and ten was found cracked at the block end of the leaf. This indication is discussed later in this report.

Item B13.30, "Interior Attachments beyond the Beltline", Accessible welds, were VT-3 examined as required this interval. These examinations are required to be performed once each interval. Several were performed during the third interval in outages prior to D2R17. The locations examined this outage included all four Steam Dryer Wall Brackets, the upper two Moisture Separator guide rod mounting lugs, and the upper and lower (four) steam dryer guide rod lug attachments.

Item B13.40, "Core Support Structures", Accessible Welds, were VT-3 examined as required this interval. These examinations are required to be performed once each interval. Several were performed during the third interval in outages prior to D2R17. The locations examined this outage included the shroud vertical welds beyond the beltline region. No recordable indications were noted.

Core Spray System Examinations

The Core Spray piping internal to the reactor vessel was examined using ultrasonic or EVT-1 enhanced visual inspection in accordance with BWRVIP-18, "BWR Vessels and Internals Projects, BWR Core Spray Internals Inspection and Flaw Evaluation Guidelines" under WO# 99137500-01. Previously identified cracks on welds 1P5, 2P8a, 3P4c, 3P4d, 3P4d, 3P8a, and 4P8a were sized ultrasonically and evaluated for flaw growth. These flaws were previously evaluated after D2R15 under Sargent & Lundy Flaw Analysis SL-5197 in accordance with ASME Section XI and BWRVIP-18. At that time, the flaws were deemed to be acceptable for two cycles. Since D2R15 there has been no increase in flaw length on welds with demonstrated ultrasonic techniques. However, there have been changes in flaw lengths associated with flaws on undemonstrated (P8a). Specifically, the flaw at 2P8a has grown from 5.36" to 5.8" and the flaw at 4P8a, which was never visually confirmed, has been determined to be free of flaws. The flaws were again evaluated this outage in General Electric report GENE-B13-02136-00 and determined to be acceptable for another two cycles of operation. The P8a locations are of minimal concern since they do not impact the ECCS LOCA leakage analysis.

Section II Scope of Inspection

Attachment A Summary of Vessel Interior Examinations

Also, in accordance with BWRVIP-18, the Core Spray sparger "target weld" set identified under part 3.2.3 was inspected. The sparger was baseline inspected during D2R15. The D2R17 scope consisted of an EVT-1 of all S-1, S-2 and S-4 welds and a VT-1 of 50% of the S-3 welds and was performed under WO# 99137499-01. No indications were identified on sparger welds.

Jet Pump Examinations

The current six-year inspection cycle for Jet Pump component welds specified in BWRVIP-41 began during D2R15. During the D2R16 inspections of medium priority wall brace welds, the adjacent ASME Section XI B-N-2 vessel attachment welds were also inspected for eleven of the twenty attachments. No flaws were identified at those locations. This outage, the remaining nine locations were inspected to satisfy B-N-2 requirements. There were no flaws identified on the B-N-2 attachment welds. However, a fatigue crack was identified on an adjacent weld at one upper leaf on the Jet Pump number 9 side of the 9/10 riser brace leaf to block weld (BWRVIP weld designation RB-4b) at the toe of the weld across the leaf. The eleven braces previously examined during D2R16 were then re-examined so that all twenty locations were examined this outage. In addition, a review of D2R14 examination records for set screw clearances identified that a gap may exist at the restrainer gates of Jet Pumps 9 and 10 and at this location only. This was of particular interest to GE since wear at set-screws of BWR3 plants had not been thought to be a potential problem. Closer examination at this location confirmed that no actual gap existed.

GE prepared an evaluation of continued operation with a wall brace only partially intact. An important conclusion of this analysis is that the failure of a wall brace is not a safety issue, but is instead an asset preservation issue. This position has previously been accepted in an SER to the BWRVIP-28 report. The GE analysis addressed recirc pump vane pass frequency (VPF) induced leaf resonance and single loop operation (SLO) flow induced vibration. Prior to D2R17, studies had been performed to assess leaf resonance during increased pump speed at end-of-cycle Extended Power Uprate (EPU) conditions. These studies concluded that operation above 100% rated speed would not result in resonance. However, it was also concluded that resonance could occur at speeds much lower than expected, between 70 and 80%, and recommended that restrictions be imposed over this "exclusion zone". Also, a review of original start-up testing data revealed that increased flow in the operating loop during SLO would quickly accumulate fatigue usage of a partially intact wall brace in the exclusion zone. Recommendations for maximum speed have been in place for SLO since 1970. This test data was reviewed to evaluate the affect of running pump speeds from full rated down through the exclusion zone. Additional mock-up testing was also performed to predict wall brace resonance with one of the four leaves failed.

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Section II

Scope of Inspection

Attachment A

Summary of Vessel Interior Examinations

Based on the recommendations of this report, an operability evaluation was prepared that implements a method for limiting and monitoring fatigue usage. Operating procedures have been revised to restrict plant operation in these regions.

All other high and medium priority Jet Pump riser welds as required have been inspected during previous outages in accordance with BWRVIP-41. Of these welds, a single RS-1 weld on the riser to Jet Pump pair 15/16 was identified to contain a 1.5" long flaw during D2R15. It was evaluated at that time to be acceptable for a two cycles of operation and therefore was examined again this outage under WO# 99241679-01. There was no change in the length of this flaw since D2R15.

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Section III

Abstract of Corrective Measures

The findings and subsequent measures taken to correct the findings demonstrate that all components examined are functional and in compliance with the Dresden Station ISI Program, Dresden Station Technical Requirements Manual, the ASME Boiler and Pressure Vessel Code, Section XI, 1989 Edition and 1998 Edition (implemented in accordance with Relief Request MCR-02 for Containment ISI), Generic Letter 88-01, and BWRVIP-75.

The following is a summary of corrective measures taken as a result of examination findings.

Section III

Abstract of Corrective Measures

Category	Item	Augment	System	Line	Component	Type
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B-P	B15.XX		RR	TEST BLOCK	2RC01	N/A
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During the D2R17 system leakage test a small number of recordable indications were discovered. CR 84257 was initiated to document discrepancies. The following recordable indications were noted and addressed per the provisions of Relief Requests PR-18 and PR-20 or subsequent corrective maintenance: bonnet leaks on valves 2-0203-2A, 2-0203-2B, 2-0203-2C (corrective measures under WO 99124181-14); flange leaks on control rod drives C10, C11, J3, and K15 (corrective measures in accordance with relief request PR-20); bonnet leak on pump 2-0202B (corrective measures in accordance with WO 99124181-13).

C-C	C3.20		HPCIPD	2304-14	M-1151D-10	IWA
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While performing a MT examination, two indications were observed adjacent to one of the shear lugs. The indications exceeded the acceptance criteria of ASME Section XI Table IWB-3512-1 for surface indications. Therefore, a supplemental UT examination was performed to characterize the indications by taking pre and post UT thickness readings while performing additional surface preparation on the area in order to evaluate the indications in accordance with IWC-3100. Upon completion of the additional surface preparation the indications had been removed. Removal was verified by performance of an as left MT. The indications were very shallow and were contributed to processing during fabrication of the piping. In accordance with IWB-3514.2 the indications were evaluated against the flaw standards for volumetric examination. Based on this evaluation the indications were well within the acceptance standards of IWC-3000 (IWB-3514). Therefore, no repair or expansion was required.

C-H	C7.XX		RR	TEST BLOCK	2RC01	N/A
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During the D2R17 system leakage test a small number of recordable indications were discovered. CRs 84257 and 80704 were initiated to document discrepancies. The following recordable indications were noted and addressed per the provisions of Relief Request PR-18 or subsequent corrective maintenance: bonnet leak on control rod drive hydraulic control unit valves 3-0305-101(G7), 3-0305-102(B8), 3-0305-101(F11), 3-0305-120/123(C5), 3-0305-120/123(F10), 3-0305-127(G2), 3-0305-127(C3), 3-0305-101(K12), 3-0305-101(P12), 3-0305-102(M10), 3-0305-127(K15), 3-0305-102(M2), 3-0305-120(K9), 3-0305-120(K8), 3-0305-102(H6), 3-0305-112(K3), 3-0305-112(L10), 3-0305-112(K12). CRD corrective measures were performed under WOs 99124181-10, 11, 375020-01, 375117-01, and 375119-01.

E-A	E1.11		CONT	N/A	DW 502 LINER	DW LINER
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During the containment general visual examination, an area that varied from two to four inches wide encompassing almost 360 degrees around the DW shell adjacent to the basement floor immediately above the moisture barrier exhibited areas of missing coating and primer resulting in corrosion of the base metal. A detailed visual exam was performed and UT thickness readings were taken in the area of the basement floor/moisture barrier. UT thickness readings demonstrated that as found degradation was well within the corrosion allowance for the drywell liner, therefore this condition is considered acceptable. Work Order 00376342 was initiated to repair the coating in this area to prevent further long term degradation. Two other areas that exhibited coating separation and rusting were at penetrations X-103C and X-115A at their associated penetration sleeve adjacent to the DW shell. A detailed visual exam was performed which revealed surface corrosion was present but no wastage was detected visually. Based upon the additional observations, these areas were considered acceptable. The conditions had WOs initiated to apply Service Level 1 coating (ref. WOs 00377187 and 00377188) in order to prevent any long term degradation which would impact the structural integrity of the containment structure. The entire drywell liner was inspected during this inspection. Therefore, no areas exist in which additional examinations could be performed.

Section III

Abstract of Corrective Measures

Category	Item	Augment	System	Line	Component	Type
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E-A	E1.12		CONT	N/A	TOR B10 SUB	TOR LINER
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During visual inspection of underwater coating in the suppression pool, an area requiring repair at location 1-1 had an identified pit depth of 104 mils. This pit exceeded 10% of the torus bottom plate thickness which is 0.653" thick. The pitting was a result of isolated coating failures exposing the base metal to suppression pool water. An evaluation of the condition was performed and found to be acceptable without further repair. This area was buffed to a bright metal finish and repair coating applied to prevent further degradation. Each refuel outage the entire under water torus coating is examined resulting in identification of similar isolated coating failures allowing pits to develop in the base metal. Visual examination of the entire underwater surface each refuel outage has identified coating deficiencies prior to the occurrence of significant base metal degradation. Evaluation of any pitting is performed and coating is repaired to ensure the structural integrity of containment is maintained. Therefore, all of the components in which this type of degradation could occur have been inspected and evaluated, and no additional examinations were performed.

E-A	E1.12		CONT	N/A	TOR B14 SUB	TOR LINER
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During visual inspection of underwater coating in the suppression pool, an area requiring repair at location 4-3 had an identified pit depth of 68 mils. This pit exceeded 10% of the torus bottom plate thickness which is 0.653" thick. The pitting was a result of isolated coating failures exposing the base metal to suppression pool water. An evaluation of the condition was performed and found to be acceptable without further repair. This area was buffed to a bright metal finish and repair coating applied to prevent further degradation. Each refuel outage the entire under water torus coating is examined resulting in identification of similar isolated coating failures allowing pits to develop in the base metal. Visual examination of the entire underwater surface each refuel outage has identified coating deficiencies prior to the occurrence of significant base metal degradation. Evaluation of any pitting is performed and coating is repaired to ensure the structural integrity of containment is maintained. Therefore, all of the components in which this type of degradation could occur have been inspected and evaluated, and no additional examinations were performed.

E-A	E1.30		CONT	N/A	DW BASE MBARR	MBARR
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During performance of the containment general visual examination, the drywell basement moisture barrier was found severely degraded and completely missing in several areas. Base metal was exposed with areas of surface corrosion present. The condition of the existing moisture barrier was unacceptable (reference CR 0079639). The existing moisture barrier was replaced under WO 99211609-01 and a detailed visual inspection was performed prior to installation of the new moisture barrier material. In addition, another detailed visual inspection was also performed along with a supplemental UT to determine the extent of corrosion to the DW shell. UT thickness readings were taken at two areas that witnessed the greatest level of corrosion. The thinnest location was at approximate azimuth of 50 degrees measured 0.968". The second location was at approximate azimuth 225 degrees and was 1.001", while the adjacent metal shell not affected by any corrosion was varied in thickness of 1.03" to 1.05". Both degraded areas are within the allowable corrosion tolerance of 1/4". A visual examination was performed on the newly installed moisture and found acceptable.

F-A	F1.10		RHS	0304-2.5	M-1167D-1	CL 1 SUP
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During VT-3 examination, a loose lock nut on the strut was discovered. Initiated CR 79613 to document discrepancy. All adjacent supports have been expanded to and no additional supports of the same type and function exist on the system. Therefore, no further expansion is required. Support is removed each refuel outage during reactor disassembly. Lock nuts were tightened during reactor reassembly, support reinspected and found acceptable.

F-A	F1.10		RHS	0304-2.5	M-1167D-2	CL 1 SUP
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During VT-3 examination, a loose lock nut on the strut was discovered. Initiated CR 79613 to document discrepancy. Expanded to adjacent supports M-1167D-1 and M-1167D-3. No additional supports of the same type and function exist on the system. Therefore, no further expansion is required. Support is removed each refuel outage during reactor disassembly. Lock nuts were tightened during reactor reassembly, support reinspected and found acceptable.

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Section III

Abstract of Corrective Measures

Category	Item	Augment	System	Line	Component	Type
F-A	F1.10		RHS	0304-2.5	M-1167D-3	CL 1 SUP
F-A	F1.40		RRAS	PMP 2A-0202	M-1135 SHT 13	CL 1 SNB

During VT-3 examination, a loose lock nut on the strut was discovered. Initiated CR 79613 to document discrepancy. All adjacent supports have been expanded to and no additional supports of the same type and function exist on the system. Therefore, no further expansion is required. Support is removed each refuel outage during reactor disassembly. Lock nuts were tightened during reactor reassembly, support reinspected and found acceptable.

During the initial D2R17 100% snubber inspection it was discovered that a PSA Mechanical Snubber (EPN: 2A-0202-12) exterior was covered with an affixed foreign material with concern that this material may have entered the snubber interior working mechanisms, CR 079637 was written to disposition the conditions. The PSA-35 snubber was cleaned/removed and tested during D2R17 under WO 0372768-01. The snubber passed the as-found testing, but was replaced with a new snubber to alleviate any potential that the material did work it's way to the insides causing a future test failure.

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Section IV Abbreviations

Augment

GL88-01	Generic Letter 88-01 Exam, Weld Category
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Exam

MT	Magnetic Particle
PT	Liquid Penetrant
UT	Ultrasonic
UT-E	Ultrasonic-Risk ISI Expanded Volume
VT-1, 2, or 3	Visual Examination

Exam Selection

AD	Additional Examination (expansion)
AG	Augmented Requirement
BL	Baseline
OTHR	Other
SU	Successive Examination
XI	ASME Section XI Requirement

System

ECCS	Emergency Core Cooling System Ring Header
FW	Feedwater
HPCI	High Pressure Coolant Injection
HPCIPD	High Pressure Coolant Injection, Pump Discharge
HPCISS	High Pressure Coolant Injection, Steam Supply
ISCO	Isolation Condenser
ISCOCR	Isolation Condenser, Condensate Return
ISCOSS	Isolation Condenser, Steam Supply
LPCIBD	Low Pressure Coolant Injection "B", Pump Discharge
MS	Main Steam
MSA	Main Steam "A"
MSB	Main Steam "B"
MSC	Main Steam "C"
MSD	Main Steam "D"
RHS	Reactor Head Spray
RPV	Reactor Pressure Vessel
RR	Reactor Recirculation
RVBD	Reactor Vessel Bottom Drain
SBLC	Standby Liquid Control
SDC	Shutdown Cooling

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Section IV

Abbreviations

SRVDA	Safety Relief Valve Discharge "A"
SRVDB	Safety Relief Valve Discharge "B"
SRVDC	Safety Relief Valve Discharge "C"
SRVDD	Safety Relief Valve Discharge "D"
SRVDE	Safety Relief Valve Discharge "E"

Type

BELLOW	Bellows
BLTCONN	Bolted Connection
BPC	Branch Pipe Connection
BPCS	Branch Pipe Connection Saddle
CAP	Pipe Cap
COND	Condenser
CRO	Cross
EL	Elbow
ELS	Elbow Longitudinal Seam
F	Flued Head
FLG	Flange
FLGBLT	Flange Bolt
FLS	Fitting Longitudinal Seam
GASKET	Gasket
HTEX	Heat Exchanger
IWA	Integral Welded Attachment
MBARR	Moisture Barrier
NIR	Nozzle Inner Radius
NOZ	Nozzle
P	Pipe
PG	Penetration Guide
PLS	Piping Longitudinal Seam
PMP	Pump
PMPBLT	Pump Bolting
RED	Reducer
REDE	Reducing Elbow
RPV	Reactor Pressure Vessel
SDL	Saddle
SE	Safe-end
SEAL	Seal
SHL	Shell
SURF	Containment Surface
SWC	Socket Welded Coupling
SWCP	Socket Welded Pipe Cap
SWE	Socket Welded Elbow
SWF	Socket Welded Flange
SWP	Sweep-O-Let, Weld-O-Let, Etc.
SWR	Socket Welded Reducer

Exelon Nuclear
P.O. Box 805379, Chicago, IL 60680-5379

Dresden Nuclear Power Station
6500 N. Dresden Road, Morris, IL 60450

October 2001 Inservice Inspection
Unit No. 2; National Board No. N-137
Commercial Service Date: 6-9-72

Section IV

Abbreviations

SWT	Socket Welded Tee
SWV	Socket Welded Valve
TBSHT	Tubesheet
TEE	Tee
VB	Vacuum Breaker
VLV	Valve
VLVBLT	Valve Bolting

Section V

Repairs and Replacements Since the Preceding Summary Report

Several ASME Section XI repairs and replacements have taken place at Dresden Unit 2 since the previous summary report was issued. A review of the Dresden Station Section XI Repair Program Log was conducted in order to identify the various repairs and replacements. Although not required per IWA-6210(c), Class 3 repairs and replacements are also included in this report.

Copies of the NIS-2 forms associated with all of the Section XI repairs and replacements conducted since the previous summary report have been included in this section. The NIS-2 forms provide an abstract of the repairs and replacements and outline the examinations and tests performed in conjunction with them. Code Data Reports are not included in this report, but are available for review at Dresden Station.

A listing of NIS-2 forms is included in this section in order of repair/replacement plan number followed by the associated work request number.

Exelon Nuclear
P.O. Box 805379, Chicago, IL 60680-5379

Dresden Nuclear Power Station
6500 N. Dresden Road, Morris, IL 60450

October 2001 Inservice Inspection
Unit No. 2; National Board No. N-137
Commercial Service Date: 6-9-72

Section V

Repairs and Replacements Since the Preceding Summary Report

NIS-2 No.	Work Request
2-00-003	990169728-01
2-00-005	990111436-01
2-00-006	990116325-01
2-00-007	990202253-01
2-01-001	990020585-01
2-01-002	990117771-01
2-01-003	99231860
2-01-008	990232288-01
2-01-009	990014750-01
2-01-014	970110711-01
2-01-015	970110713-01
2-01-016	990051247-01
2-01-017	990051249-01
2-01-018	990063638-01
2-01-019	990111436-03
2-01-020	990116325-03
2-01-021	990062741
2-01-022	990062742
2-01-023	990062743
2-01-024	990062744
2-01-025	990253002-01
2-01-026	990231572
2-01-027	990120951-01
2-01-029	99259544
2-01-031	970072186-01
2-01-032	990269493
2-01-033	99250494
2-01-036	321079
2-01-045	99102788, 91
2-01-046	990102788
2-01-047	99248274
2-01-048	PO 23108
2-01-049	99248723
2-01-053	99235806
2-01-060	99222115
2-01-061	99222112
2-01-062	372768
2-01-063	374695
2-01-064	99117771
2-01-065	99225126
2-01-066	381568-01
2-02-001	400429-01
2-99-002	970047223-01

Exelon Nuclear
P.O. Box 805379, Chicago, IL 60680-5379

Dresden Nuclear Power Station
6500 N. Dresden Road, Morris, IL 60450

October 2001 Inservice Inspection
Unit No. 2; National Board No. N-137
Commercial Service Date: 6-9-72

Section V
Repairs and Replacements Since the Preceding Summary
Report

NIS-2 No.	Work Request
2-99-037	PO 367668
2-99-038	PO 000239
2-99-043	990004356-01
2-99-054	990053038-01

CATEGORY 3

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 08

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-27-2000

Sheet: 1 Of 1

Unit: 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WR 990169728-01 (PLAN 2-00-003)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 CCSW

5.(a) Construction Code USAS B31.1.0-1967, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
2D CCSW Pump Discharge Flange-to-Elbow Weld	Not Applicable	N/A	N/A	Line 2-1514D-10"-D	N/A	Repair	No

7. Description of work: Repaired pinhole leak in weldneck flange-to-elbow weld.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 185 psig Test Temperature 95.6 °F

9. Remarks: VT-2 examination performed during CCSW Operating Surveillance DOS 1500-02 on 6/10/2000: No leakage observed.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPAIR Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey ISI COORDINATOR 6-27, 2000
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPAIR described in this report on 6-27, 2000 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-27-00 Inspector: Paul T. Ramsey Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-02-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: G.N. Venture (Name)
Same as Above (Address)

WO 99111436-01 PLAN 2-00-005
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (LPCI)

5. (a) Construction Code ASME Sect.III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
2A CCSW/LPCI Hx Tubes.	Unknown for the tubes.	Unknown	N/A	EPN: 2-1503A	N/A	Repair/Replaced	No
Tube Plugs for 3/4" tube X 15-22 Gauge Tube (10 plugs).	Thomas Wilson	None identified	N/A	Cat ID-27487, UTC-2402007, 2003730.	N/A	Installed to Repair	No

7. Description of work: Based upon eddy current results, repaired tubes by plugging (10 tubes plugged). No tubes were replaced.

8. Test Conducted: Hydrostatic [X] Pneumatic [] Nominal Operating Pressure [] Not Applicable []

Test Pressure 375 psig Test Temperature 75.2 °F

9. Remarks: No leakage identified by VT-2 during pressurization on 10/31/01.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed : John H. Young ISI COORDINATOR Jan. 15, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by Hartford Steam Boiler of CT having inspected the **REPAIR/REPLACEMENT** described in this report on 1-13, 20 02. By and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-13-02 Inspector: Robert T. Rainey Robert T Rainey Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-2-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: G.N. Venture (Name)
Same as Above (Address)

WO 99116325-01 PLAN 2-00-006
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (LPCI)

5. (a) Construction Code ASME Sect.III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
2B CCSW/LPCI Hx Tubes.	Unknown for the tubes.	Unknown	N/A	EPN: 2-1503B	N/A	Repair/Replaced	No
Tube Plugs for 3/4" tube X 15-22 Gauge Tube (50 plugs).	Thomas Wilson	None identified	N/A	Cat ID-27487, UTC-2402007	N/A	Installed to Repair	No
3/4" X 18 BWG ASME SB-111 Tubes (55 tubes).	Unknown	None identified	N/A	Cat ID-42456, UTC's-2622489, 2622573	N/A	Replacement	No

7. Description of work: Based upon eddy current results, repaired tubes by plugging (50 tubes plugged) or replaced tubes with new tubes (55 tubes replaced).
A hydro of the tube sheets was performed prior to reassembling the Hx with no leakage observed..

8. Test Conducted: Hydrostatic ☒ Pneumatic ☐ Nominal Operating Pressure ☐ Not Applicable ☐

Test Pressure 436 psig Test Temperature 84 °F

9. Remarks: No leakage identified by VT-2 during pressurization on 10/31/01.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan. 12, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-15, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-15-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 08

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)
2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)
3. Work Performed By: Same as Above (Name)
Same as Above (Address)

Date: 9-13-2000

Sheet: 1 Of 1

Unit: 2

WR 990202253-01(PLAN 2-00-007)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 CCSW/LPCI

- 5.(a) Construction Code ASME Section III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
2B CCSW/LPCI Heat Exchanger Tubes (Inlet Tube Numbers 20-9 and 22-14, Outlet Tubes 2-24, 30-3, 16-25, 8-20, and 17-10)	Not Identified	Unknown	N/A	Inlet: 20-9,22-14 Outlet: 2-24, 30-3, 16-25, 8-20, 17-10	N/A	Repair	No
14 Heat Exchanger Tube Plugs	Unknown	Unknown	N/A	Catalog ID 27487	N/A	Replacement	No

7. Description of work: Chemistry testing results discovered tube leak on 2B CCSW/LPCI heat exchanger. Heat exchanger was disassembled and tubes were pressure tested and select tubes were eddy current tested. Seven tubes were plugged based on pressure test and eddy current results.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Exempt from hydrostatic testing per IWA-4700(b)(2). Maintenance pressure test was performed to verify no leakage at newly installed tubes (no leakage observed).

Certificate of Compliance

We certify that the statements made in this report are correct and this REPAIR conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey ISI COORDINATOR 9-13, 2000
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPAIR described in this report on 9-14, 2000 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 9-14-00 Inspector: Purt J. Riney Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-22-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Sheet: 1 Of 1

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 99020585-01 PLAN 2-01-001
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1400 (Core Spray)

5. (a) Construction Code USAS B31.1-0/ASME Sect.VIII, 19 67/65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Relief valve at EPN 2-1402-28B	Consolidated	252336-1-EJ	N/A	None	Unknown	Replaced	No
Relief valve at EPN 2-1402-28B	Consolidated-Dresser	TC25708	N/A	Cat ID-004180, UTC-2066269, P/N: 1910HC-1-XMC368.	Unknown	Replacement	No

7. Description of work: Replace the flanged relief valve with a new/refurbished one.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 390 psig Test Temperature 84 °F

9. Remarks: Satisfactory VT-2 performed.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan. 11: 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-15, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-15-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-4-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: General Electric (Name)
Same as Above (Address)

WO 99117771-01 PLAN 2-01-002
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0200 Reactor Pressure Vessel

5. (a) Construction Code ASME Sect.III, 19 65 Edition, NO Addenda, Code Cases 1335
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Reactor Pressure Vessel Head Closure Studs (SA-320 Grade L42 and Code Case 1335, five total.	Babcock and Wilcox	Not recorded	N/A	Stud Position Numbers 68, 69, 70, 71, and 72.	N/A	Replaced	No
Reactor Pressure Vessel Head Closure Studs (SA-320 Grade L42 and Code Case 1335, five total.	Babcock and Wilcox	61-111-68, 69, 70, 71, & 72.	N/A	Cat ID-38457, WO's: 99052144-01 & 99126507-01.	N/A	Replacement	No

7. Description of work: Remove/reinstall with rotated stock, the five "cattle chute" related Rx studs.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Not Applicable ☒

Test Pressure psig Test Temperature °F

9. Remarks: The Replacement Rx studs came from D3R16 under WO 99052144-01. They were deconed/cleaned/NDE performed under WO 99126507-01 and staged back on the refuel floor for D2R17.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: John H. Young ISI COORDINATOR Jan. 12, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by Hartford Steam Boiler of CT having inspected the **REPAIR/REPLACEMENT** described in this report on 1-18, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: Robert T. Rainey Robert T. Rainey Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-5-2001

Sheet: 1 Of 3

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

DCP 9900395/WO 99231860/PLAN 2-01-003
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (CCSW/LPCI)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Pipe line 2-1558A-2"-D.	Unknown	None	N/A	Associated with Hx 2-1503-A.	N/A	Replaced	No
Pipe line 2-1558A-2"-D, pipe sch.80.	Unknown	HT#-720254	N/A	Cat. ID-24507, UTC-2066926.	N/A	Replacement	No
Pipe line 2-1558A-2"-D, flange set, 2" s.w., 300lb.	Unknown	Not Recorded	N/A	Cat ID-7480, UTC-2600427.	N/A	Replacement	No
5/8"-11 threaded rod for flange set on 2-1558A-2"-D.	Unknown	Not Recorded	N/A	Cat ID-39725, UTC-2616440.	N/A	Replacement	No
5/8"-11 heavy hex nuts for flange set on 2-1558A-2"-D.	Unknown	Not Recorded	N/A	Cat ID-37029, UTC-2005510.	N/A	Replacement	No

7. Description of work: Replaced existing globe valves with gate valves and flanged piping to facilitate Hx maintenance. Affected WO Tasks 01 & 02 (Hx 2-1503A/pipe 2-1558A-2"), Tasks 06, & 07 (Hx 2-1503B/pipe 2-1558-2").

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []
Test Pressure *** psig Test Temperature Ambient °F

9. Remarks: ***227 psig on 2-1558A-2" and 221 psig on 2-1558B-2" portion of piping. A VT-2 was performed during DOS 1500-12 for both affected trains. Other than a packing leak noted on valve 2-1599-65B, no leakage was detected.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPAIR/REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR 1/7, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPAIR/REPLACEMENT described in this report on 1-14, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-14-02 Inspector: [Signature] Commissions: IL932, NB7742NIBB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-5-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 2 Of 3

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

DCP 9900395/WO 99231860/PLAN 2-01-003
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (CCSW/LPCI)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
2" X 1" s.w. reducing insert on pipe 2-1558A-2"-D.	Unknown	None	N/A	Line #2-1558A-2"	N/A	Replaced	No
2" X 1" s.w. 3000 psi, reducing insert.	Unknown	HT#021J	N/A	Cat ID-38510, UTC-2615426.	N/A	Replacement	No
2" s.w. tee on pipe line 2-1558A-2"-D.	Unknown	None	N/A	Line # 2-1558A-2"	N/A	Replaced	No
2" s.w. 3000 psi, tee.	Unknown	Not Recorded	N/A	Cat ID-7361, UTC-2018116.	N/A	Replacement	No
2" s.w. globe valve 2-1599-65A.	Hancock	Model 5500W1	N/A	EPN: 2-1599-65A	N/A	Replaced	No
2" s.w. cl 800 gate valve 2-1599-65A.	Vogt	Model SW12111 2"	N/A	Cat ID-412907, UTC-2602417.	N/A	Replacement	No
Pipe line 2-1558B-2"-D.	Unknown	None	N/A	Associated with Hx 2-1503-B.	N/A	Replaced	No
Pipe line 2-1558B-2"-D, pipe sch.80.	Unknown	HT#-720254	N/A	Cat ID-24507, UTC-2066926.	N/A	Replacement	No
Pipe line 2-1558B-2"-D, flange set, 2" s.w., 300lb.	Unknown	Not Recorded	N/A	Cat ID-7480, UTC-2600427.	N/A	Replacement	No
5/8"-11 threaded rod for flange set on 2-1558B-2"-D.	Unknown	None	N/A	Cat ID-39725, UTC-2616440.	N/A	Replacement	No
5/8"-11 heavy hex nuts for flange set on 2-1558B-2"-D.	Unknown	HT#021J	N/A	Cat ID-37029, UTC-2005510.	N/A	Replacement	No
2" X 1" s.w. reducing insert on pipe 2-1558B-2"-D.	Unknown	None	N/A	Line #2-1558B-2"	N/A	Replaced	No
2" X 1" s.w. 3000 psi, reducing insert.	Unknown	HT#021J	N/A	Cat ID-38510, UTC-2615426.	N/A	Replacement	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-5-2001

Sheet: 3 Of 3

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

DCP 9900395/WO 99231860/PLAN 2-01-003
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (CCSW/LPCI)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
2" s.w. tee on pipe line 2-1558B-2"-D.	Unknown	None	N/A	Line # 2-1558B-2"	N/A	Replaced	No
2" s.w. 3000 psi, tee.	Unknown	Not Recorded	N/A	Cat ID-7361, UTC-2018116.	N/A	Replacement	No
2" s.w. globe valve 2-1599-65B.	Hancock	Model 5500W1	N/A	EPN: 2-1599-65B	N/A	Replaced	No
2" s.w. cl 800 gate valve 2-1599-65B.	Vogt	Model SW12111 2"	N/A	Cat ID-412907, UTC-2602417.	N/A	Replacement	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-4-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

DCP 9900514 W/O 99232288 (PLAN 2-01-008)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2500 (ACAD)

5. (a) Construction Code ASME Section III, 1974 Edition, Summer 1976 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 1998* Edition, NO Addenda, Code Cases NONE

*1998 Edition implemented in accordance with Relief Request MCR-02

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No.	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Cap on line 2-2502A-1" through Containment Penetration X-202V	Unknown	None	N/A	Cat ID 38871	Unknown	Replacement	No
Cap on line 2-2502B-1" through Containment Penetration X-204B	Unknown	None	N/A	Cat ID 38871	Unknown	Replacement	No
Cap on line 2-2503A-1" through Containment Penetration X-316A	Unknown	None	N/A	Cat ID 38871	Unknown	Replacement	No
Cap on line 2-2503B-1" through Containment Penetration X-316B	Unknown	None	N/A	Cat ID 38871	Unknown	Replacement	No
Cap on line 2-2504-1" through Containment Penetration X-125	Unknown	None	N/A	Cat ID 38871	Unknown	Replacement	No

7. Description of work: Removed ACAD system, cut and capped lines at containment penetrations.

8. Test Conducted: Hydrostatic [] Pneumatic [X] Nominal Operating Pressure [] Not Applicable []

Test Pressure 49.5 psig Test Temperature Ambient °F

9. Remarks: Pneumatic test performed in accordance with 10 CFR 50 Appendix J and ASME Section XI IWE-5240

Certificate of Compliance

We certify that the statements made in this report are correct and this REPAIR/REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR 11-5, 2001
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPAIR/REPLACEMENT described in this report on 11-9, 2001 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 11-5-01 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-24-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 99014750-01 PLAN 2-01-009
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2300 (High Pressure Coolant Injection)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Relief valve at EPN 2-2301-53	Consolidated-Dresser	TC41364	N/A	P/N: 1905-30 PC/SP	N/A	Replaced	No
Relief valve at EPN 2-2301-53	Consolidated-Dresser	TL96055	N/A	Cat ID-46883, UTC-2056244, P/N: 1905NC-2-1 A-MC324.	N/A	Replacement	Yes

7. Description of work: Replace the flanged relief valve with a new/refurbished one.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 60 psig Test Temperature Ambient °F

9. Remarks: Satisfactory VT-2 performed.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan. 11, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-18, 20 02 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-22-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Sheet: 1 Of 1

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 97110711-01 PLAN 2-01-014
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (Low Pressure Coolant Injection)

5. (a) Construction Code ASME Sect.VIII, 1965 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Relief valve at EPN 2-1599-13C	Consolidated-Dresser	TF74326	N/A	P/N: 1905FC-1-XYM19.	N/A	Replaced	No
Relief valve at EPN 2-1599-13C	Consolidated-Dresser	TM67812	N/A	Cat ID-44534, UTC-2622132, P/N: 1905F-XLA2-NC3155.	N/A	Replacement	No

7. Description of work: Replace the flanged relief valve with a new/refurbished one.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 7 psig Test Temperature Ambient °F

9. Remarks: Satisfactory VT-2 performed.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan. 11, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-18, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-22-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 97110713-01 PLAN 2-01-015
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (Low Pressure Coolant Injection)

5. (a) Construction Code ASME Sect.VIII 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Relief valve at EPN 2-1599-13D	Consolidated-Dresser	TK52699	N/A	P/N: 1905FC-1-XMC368.	N/A	Replaced	No
Relief valve at EPN 2-1599-13D	Consolidated-Dresser	TM67809	N/A	Cat ID-44534, UTC-2622130, P/N: 1905F-XLA2-NC3155.	N/A	Replacement	No

7. Description of work: Replace the flanged relief valve with a new/refurbished one.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒ Not Applicable ☐

Test Pressure 7 psig Test Temperature Ambient °F

9. Remarks: Satisfactory VT-2 performed.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPAIR/REPLACEMENT conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan.11, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPAIR/REPLACEMENT described in this report on 1-18, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: [Signature] Commissions: IL932, NB7742NIBS
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-31-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 99051247-01 PLAN 2-01-016
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code **ASME Section III, 1965 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

** General Electric APED Specification 21A5744.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Electromatic Relief Valve at EPN: 2-0203-3B.	Dresser Industries	S/N: BK7002	N/A	Type 1525VX	Unknown	Replaced	Yes
Electromatic Relief Valve to EPN: 2-0203-3B.	Dresser Industries	S/N: BK7079	N/A	Cat ID-42845, UTC-2622214	Unknown	Replacement	Yes

7. Description of work: Removed existing ERV and replaced with a rebuilt spare ERV.

8. Test Conducted: Hydrostatic ☒ Pneumatic ☐ Nominal Operating Pressure ☐ Not Applicable ☐

Test Pressure 1005 psig Test Temperature > 135 / < 212 °F

9. Remarks: Satisfactory VT-2 examination was recorded during the D2R17 10 Yr Hydro under WO 99124181-09.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: SCDU-g ISI COORDINATOR Jan.08, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-18, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: R. T. Rainey Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-30-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 99051249-01 PLAN 2-01-017
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code **ASME Section III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

** General Electric APED Specification 21A5744.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Electromatic Relief Valve at EPN: 2-0203-3D.	Dresser Industries	S/N: BK7053	N/A	Type 1525VX	Unknown	Replaced	Yes
Electromatic Relief Valve to EPN: 2-0203-3D.	Dresser Industries	S/N: BK7050	N/A	Cat ID-42845, UTC-2622215	Unknown	Replacement	Yes

7. Description of work: Removed existing ERV and replaced with a rebuilt spare ERV..

8. Test Conducted: Hydrostatic ☒ Pneumatic ☐ Nominal Operating Pressure ☐ Not Applicable ☐

Test Pressure 1005 psig Test Temperature > 135 / < 212 °F

9. Remarks: Satisfactory VT-2 examination was recorded during the D2R17 10 Yr Hydro under WO 99124181-09.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan.08, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-19, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-19-02 Inspector: Rust T. Ruviny Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-26-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 99063638-01 PLAN 2-01-018
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 8500 (N2 Inerting and D/W O2 Sample)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 98* Edition, NO Addenda, Code Cases NONE

*1998 Edition implemented in accordance with Relief Request MCR-02

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Relief valve at EPN 2-8526.	Unknown	Not recorded	N/A	None	N/A	Replaced	No
Relief valve at EPN 2-8526.	Dresser	TG13751	N/A	Cat ID-35597	N/A	Replacement	No

7. Description of work: Replace the flanged relief valve with a new/refurbished one. Replaced relief valve will be tested under WO 99229335-01..

8. Test Conducted: Hydrostatic [] Pneumatic [X] Nominal Operating Pressure [] Not Applicable []

Test Pressure 50 psig Test Temperature Ambient °F

9. Remarks: Pneumatic test performed in accordance with 10 CFR 50 Appendix J and ASME Section XI IWE-5240.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan. 11, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-18, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-28-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: G.N. Venture (Name)
Same as Above (Address)

WO 99111436-03 PLAN 2-01-019
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (LPCI)

5. (a) Construction Code ASME Sect.III, 1965 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
2A CCSW/LPCI Hx Upper and Lower Channels.	Unknown for the tubes.	05036-3	3006	EPN: 2-1503A	1967	Repair	Yes

7. Description of work: Weld repair/grind/blend pitted areas on upper and lower channels on 2A CCSW/LPCI Hx during maintenance.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 227 psig Test Temperature Ambient °F

9. Remarks: No leakage identified by VT-2 during DOS 1500-12 on 10/31/01.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan.11, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-18, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-01-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 1 Of 1

3. Work Performed By: G.N. Venture (Name)
Same as Above (Address)

WO 99116325-03 PLAN 2-01-020
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (LPCI)

5. (a) Construction Code ASME Sect.III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
2B CCSW/LPCI Hx Upper and Lower Channels.	Unknown for the tubes.	05036-4	3007	EPN: 2-1503B	1967	Repair	Yes

7. Description of work: Weld repair/grind/blend pitted areas on upper and lower channels on 2B CCSW/LPCI Hx during maintenance.

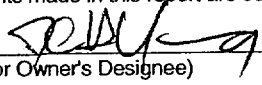
8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 221 psig Test Temperature Ambient °F

9. Remarks: No leakage identified by VT-2 during DOS 1500-12 on 11/5/01.


Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: John H. Young  ISI COORDINATOR Jan. 15, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by Hartford Steam Boiler of CT having inspected the **REPAIR/REPLACEMENT** described in this report on 1-18, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: Robert T. Rainey  Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-28-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 1 Of 1

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 99062741-01 PLAN 2-01-021
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code **ASME Section III, 19 65 Edition, S/66 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

** General Electric APED Specification 21A5743, rev.1.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Consolidated Safety Valve at EPN: 2-0203-4A.	Consolidated	S/N: BK6299	N/A	Model: 3777-QA-RT-21-0S110	Unknown	Replaced	Yes
Consolidated Safety Valve at EPN: 2-0203-4A.	Consolidated	S/N: BK6527	N/A	Cat ID-30366, UTC-2065265	Unknown	Replacement	Yes

7. Description of work: Removed existing safety valve and replaced with a rebuilt/tested spare safety valve.

8. Test Conducted: Hydrostatic ☒ Pneumatic ☐ Nominal Operating Pressure ☐ Not Applicable ☐

Test Pressure 1005 psig Test Temperature > 135 / < 212 °F

9. Remarks: Satisfactory VT-2 examination was recorded during the D2R17 10 Yr Hydro under WO 99124181-09.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan.09, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-19, 20 02 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-19-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)
2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)
3. Work Performed By: Same as Above (Name)
Same as Above (Address)
4. Identification of System: 0203 (Main Steam)
5. (a) Construction Code **ASME Section III, 19 65 Edition, S/66 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE
- Date: 10-28-2001
Sheet: 1 Of 1
Unit: 2
WO 99062742-01 PLAN 2-01-022
Repair Organization P.O. No., Job No. etc.
- ** General Electric APED Specification 21A5743, rev.1.
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Consolidated Safety Valve at EPN: 2-0203-4B.	Consolidated	S/N: BK6263	N/A	Model: 3777-QA-RT-21-0S110	Unknown	Replaced	Yes
Consolidated Safety Valve at EPN: 2-0203-4B.	Consolidated	S/N: BK7162	N/A	Cat ID-30446, UTC-2065262	Unknown	Replacement	Yes

7. Description of work: Removed existing safety valve and replaced with a rebuilt/tested spare safety valve.
8. Test Conducted: Hydrostatic [X] Pneumatic [] Nominal Operating Pressure [] Not Applicable []
Test Pressure 1005 psig Test Temperature > 135 / < 212 °F
9. Remarks: Satisfactory VT-2 examination was recorded during the D2R17 10 Yr Hydro under WO 99124181-09.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan.09, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-18, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-28-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 1 Of 1

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 99062743-01 PLAN 2-01-023
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code **ASME Section III, 19 65 Edition, S/66 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

** General Electric APED Specification 21A5743, rev.1.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Consolidated Safety Valve at EPN: 2-0203-4C.	Consolidated	S/N: BK6526	N/A	Model: 3777-QA-RT-21-0S110	Unknown	Replaced	Yes
Consolidated Safety Valve at EPN: 2-0203-4C.	Consolidated	S/N: BK6304	N/A	Cat ID-30404, UTC-2065264	Unknown	Replacement	Yes

7. Description of work: Removed existing safety valve and replaced with a rebuilt/tested spare safety valve.

8. Test Conducted: Hydrostatic [☒] Pneumatic [☐] Nominal Operating Pressure [☐] Not Applicable [☐]

Test Pressure 1005 psig Test Temperature > 135 / < 212 °F

9. Remarks: Satisfactory VT-2 examination was recorded during the D2R17 10 Yr Hydro under WO 99124181-09.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan.09, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-18-02 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-28-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 99062744-01 PLAN 2-01-024
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code **ASME Section III, 19 65 Edition, S/66 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

** General Electric APED Specification 21A5743, rev.1.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Consolidated Safety Valve at EPN: 2-0203-4D.	Consolidated	S/N: BK6232	N/A	Model: 3777-QA-RT-21-0S110	Unknown	Replaced	Yes
Consolidated Safety Valve at EPN: 2-0203-4D.	Consolidated	S/N: BK6282	N/A	Cat ID-30446, UTC-2065261	Unknown	Replacement	Yes
1-3/8"-12 inlet flange nut for EPN: 2-0203-4D.	Unknown		N/A	None	Unknown	Replaced	No
1-3/8"-12 inlet flange nut for EPN: 2-0203-4D.	Dresser	Ht code: S01	N/A	Cat ID-34748, UTC-2056302	Unknown	Replaced	No

7. Description of work: Removed existing safety valve and replaced with a rebuilt/tested spare safety valve.

8. Test Conducted: Hydrostatic ☒ Pneumatic ☐ Nominal Operating Pressure ☐ Not Applicable ☐

Test Pressure 1005 psig Test Temperature > 135 / < 212 °F

9. Remarks: Satisfactory VT-2 examination was recorded during the D2R17 10 Yr Hydro under WO 99124181-09.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan.09, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-18-02, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 1-27-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 1 Of 1

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WR 990253002-01 (PLAN 2-01-025)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 3900 Service Water (Diesel Generator Cooling Water)

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Line 2/3-39248-3"	Unknown	N/A	N/A	Service water return line from EPN 2/3-9400-102	N/A	Repair	No

7. Description of work: Excavated pin hole leak to remove flaw by grinding and performed weld repair.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 125 psig Test Temperature Ambient °F

9. Remarks: VT-2 examination performed at nominal operating pressure, no evidence of leakage noted.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR** Conforms to Section XI of the ASME Code.

Signed: *D. H. Bann* ISI COORDINATOR 1/29, 2001
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR** described in this report on 1-29, 2001 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-29-01 Inspector: *Paul T. King* Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-25-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 1 Of 3

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

DCP 9900486 WO 99231572 PLAN 2-01-026
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 (Sram Discharge Volume)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89* Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
2" Blind flange from line 2-0384-8".	Unknown	None	N/A	P & ID M-34-Sheet 2	Unknown	Replaced	No
2" 1500# blind flange.	Unknown	None	N/A	Cat ID-1230213, UTC-2622669.	Unknown	Replacement	No
7/8" dia. bolting for blind flange from line 2-0384-8".	Unknown	None	N/A	Bolting consisting of threaded rod & nuts.	Unknown	Replaced	No
7/8"-9 threaded rod.	Unknown	Not recorded.	N/A	Cat ID's-39851, 35357, 1200180; UTC's-2610872, 2021857, 2607188.	Unknown	Replacement	No
7/8"-9 heavy hex nuts.	Unknown	Not recorded.	N/A	Cat ID-37033, UTC-2616435.	Unknown	Replacement	No

7. Description of work: Removed/replaced existing blind flanges and bolting. New blind flanges have ports to facilitate hydrolazing.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 1525 psig Test Temperature Ambient °F

9. Remarks: An acceptable VT-2 accomplished during the performance of DOS 0300-12.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: SOB U-a (Owner or Owners Designee) ISI COORDINATOR (Title) 1/7, 20 02 (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-16, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-15-02 Inspector: Paul T. Rawley Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-25-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Sheet: 2 Of 3

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

DCP 9900486 WO 99231572 PLAN 2-01-026
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 (Sram Discharge Volume)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
2" Blind flange from line 2-0385-8".	Unknown	None	N/A	P & ID M-34-Sheet 2	Unknown	Replaced	No
2" 1500# blind flange.	Unknown	None	N/A	Cat ID-1230213, UTC-2622669.	Unknown	Replacement	No
7/8" dia. bolting for blind flange from line 2-0385-8".	Unknown	None	N/A	Bolting consisting of threaded rod & nuts.	Unknown	Replaced	No
7/8"-9 threaded rod.	Unknown	Not recorded.	N/A	Cat ID's-39851, 35357, 1200180; UTC's-2610872, 2021857, 2607188.	Unknown	Replacement	No
7/8"-9 heavy hex nuts.	Unknown	Not recorded.	N/A	Cat ID-37033, UTC-261435.	Unknown	Replacement	No
2" Blind flange from line 2-0386-8".	Unknown	None	N/A	P & ID M-34-Sheet 2	Unknown	Replaced	No
2" 1500# blind flange.	Unknown	None	N/A	Cat ID-1230213, UTC-2622669.	Unknown	Replacement	No
7/8" dia. bolting for blind flange from line 2-0386-8".	Unknown	None	N/A	Bolting consisting of threaded rod & nuts.	Unknown	Replaced	No
7/8"-9 threaded rod.	Unknown	Not recorded.	N/A	Cat ID's-39851, 35357; UTC's-2610872, 2021857.	Unknown	Replacement	No
7/8"-9 heavy hex nuts.	Unknown	Ht cd-QJS	N/A	Cat ID-37033, UTC-261435.	Unknown	Replacement	No
2" Blind flange from line 2-0387-8".	Unknown	None	N/A	P & ID M-34-Sheet 2	Unknown	Replaced	No
2" 1500# blind flange.	Unknown	None	N/A	Cat ID-1230213, UTC-2622669.	Unknown	Replacement	No
7/8" dia. bolting for blind flange from line 2-0387-8".	Unknown	None	N/A	Bolting consisting of threaded rod & nuts.	Unknown	Replaced	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-25-2001

Sheet: 3 Of 3

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

DCP 9900486 WO 99231572 PLAN 2-01-026
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 (Sram Discharge Volume)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Continued from Sht. 2.							
7/8"-9 threaded rod.	Unknown	Not recorded.	N/A	Cat ID's-39851, 35357; UTC's-2610872, 2021857.	Unknown	Replacement	No
7/8"-9 heavy hex nuts.	Unknown	Ht cd-QJS	N/A	Cat ID-37033, UTC-261435.	Unknown	Replacement	No
1-1/2" Blind flanges (qty-2) from line 2-0408A-6".	Unknown	None	N/A	P & ID M-34-Sheet 2	Unknown	Replaced	No
1-1/2" 1500# blind flanges (qty-2).	Unknown	None	N/A	Cat ID-1200176, UTC-2607424.	Unknown	Replacement	No
1" dia. bolting for blind flange from line 2-0408A-6".	Unknown	None	N/A	Bolting consisting of threaded rod & nuts.	Unknown	Replaced	No
1"-8 threaded rod.	Unknown	Not recorded.	N/A	Cat ID's-35220, 1200181; UTC's-2005170, 2607191.	Unknown	Replacement	No
1"-8 heavy hex nuts.	Unknown	Ht cd-HDF	N/A	Cat ID-37034, UTC-2057524.	Unknown	Replacement	No
1-1/2" Blind flanges (qty-2) from line 2-0408B-6".	Unknown	None	N/A	P & ID M-34-Sheet 2	Unknown	Replaced	No
1-1/2" 1500# blind flanges (qty-2).	Unknown	None	N/A	Cat ID-1230213, UTC-2622669.	Unknown	Replacement	No
1" dia. bolting for blind flange from line 2-0408B-6".	Unknown	None	N/A	Bolting consisting of threaded rod & nuts.	Unknown	Replaced	No
1"-8 threaded rod.	Unknown	Not recorded.	N/A	Cat ID's-35220, 1200181; UTC's-2005170, 2607191.	Unknown	Replacement	No
1"-8 heavy hex nuts.	Unknown	Ht cd-HDF	N/A	Cat ID-37034, UTC-2059784.	Unknown	Replacement	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-29-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 99120951-01 PLAN 2-01-027
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code **ASME Section III, 19 68 Edition, W/68 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

** General Electric APED Specification 21A9206, rev.6.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Target Rock Relief Valve.	Target Rock Corp.	S/N: 121	N/A	EPN: 2-0203-3A	Unknown	Replaced	Yes
Target Rock Relief Valve.	Target Rock Corp.	S/N: 223	N/A	Cat ID-008070, UTC-2062534	Unknown	Replacement	Yes
Class 3, outlet flange stud, 1"-8 X 8" SA-193 Gr. B7.	Unknown	N/A	N/A	Pipe line 2-3019A-8"	Unknown	Replaced	No
1"-8 SA-193 Gr. B7 all-thread rod cut 8" for replacement stud.	Nova Machine Products	N/A	N/A	Cat ID-45397, UTC-2545147	Unknown	Replacement	No

7. Description of work: Removed existing relief valve and replaced with a rebuilt/tested spare safety valve. Also replaced one outlet flange stud (Construction Code for this stud is USAS B31.1 1967 Ed.).

8. Test Conducted: Hydrostatic [☒] Pneumatic [☐] Nominal Operating Pressure [☐] Not Applicable [☐]

Test Pressure 1005 psig Test Temperature > 135 / < 212 °F

9. Remarks: Satisfactory VT-2 examination was recorded during the D2R17 10 Yr Hydro under WO 99124181-09. Reference Repair/Replacement Plan 2-99-038 that recorded the refurbishment of Replacement valve S/N 223 under P.O. 000239 Release 000003.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: John H. Young ISI COORDINATOR Jan. 22, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by Hartford Steam Boiler of CT having inspected the **REPAIR/REPLACEMENT** described in this report on 1-23, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-23-02 Inspector: Robert T. Rainey Robert T. Rainey Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 7-16-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

W/O 99259544 (PLAN 2-01-029)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 3900 (DGSW Keep Fill Supply to CCSW)

5. (a) Construction Code USAS B31.1.0, 1967 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
DGSW Keep Fill Supply to CCSW Check Valve	Edwards Valves	Unknown	N/A	EPN 2-3999-634	N/A	Replaced	No
DGSW Keep Fill Supply to CCSW Check Valve	Edwards Valves	Unknown	N/A	EPN 2-3999-636	N/A	Replaced	No
DGSW Keep Fill Supply to CCSW Check Valve	Velan Valve Corp.	Unknown	N/A	EPN 2-3999-634	N/A	Replacement	No
DGSW Keep Fill Supply to CCSW Check Valve	Velan Valve Corp.	Unknown	N/A	EPN 2-3999-636	N/A	Replacement	No

7. Description of work: Replaced CCSW check valves with new check valves.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 183 psig Test Temperature Ambient °F

9. Remarks: _____

Certificate of Compliance

We certify that the statements made in this report are correct and this REPAIR/REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR 7/16, 2001
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPAIR/REPLACEMENT described in this report on 7-19, 2001 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 7-19-01 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-05-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 1 Of 1

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 972072186-01 PLAN 2-01-031
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code USAS B31.1-0, 1967 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Main Steam Isolation Valve Plug/Stem assembly from 2-0203-2D.	Crane		N/A	None	Unknown	Replaced	No
Main Steam Isolation Valve Plug/Stem for 2-0203-2D.	Crane		N/A	Cat ID-1200124, UTC-2622599.	Unknown	Replacement	No

7. Description of work: Replace the valve plug with a refurbished one.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [] Not Applicable [X]

Test Pressure ___ psig Test Temperature ___ °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan. 11, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-15, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-15-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 5-14-2001

Sheet: 1 Of 1

Unit: 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WR 990269493 (PLAN 2-01-032)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 3900 (Service Water Supply ECCS Room Coolers)

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Service water to ECCS check valve	Powel	Unknown	N/A	2-3999-252	N/A	Replaced	No
Service water to ECCS check valve	Velan Valve	025099-VLV-02/03/04	N/A	2-3999-252	N/A	Replacement	No

7. Description of work: Removed existing check valve and replaced new stainless steel swing check valve.

Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 86 psig Test Temperature 80 °F

9. Remarks: _____

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR 5/14, 20 01
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 5-14, 2001, and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-15-01 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-27-2001

Sheet: 1 Of 6

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Unit: 2

3. Work Performed By: General Electric (Name)
Same as Above (Address)

WO 99250494-05 thru 24 PLAN 2-01-033
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 (Control Rod Drive)

5. (a) Construction Code ASME Sect. III, 19 65 Edition, W/65 Addenda, Code Cases 1335-2, 1361, 1352
(b) Edition of Section XI used for Repair/Replacement 19 89* Edition, NO Addenda, Code Cases N207**, 1361-2**

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Control Rod Drive	General Electric	2027	*	Location: 58-27	***	Replaced	Yes
Control Rod Drive	General Electric	A8924	*	Cat. ID-00413, UTC's-2059461 & 2063125	1992	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location:	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-PJK	N/A	Cat. ID-42416, UTC's-2059461, 2063125	N/A	Replacement	No

7. Description of work: Replace existing control rod drive assemblies and associated flange cap screws with new control rod drive assemblies and flange cap screws. The removed cap screws did not receive a VT-1 examination with application of Dresden Station Third Interval Relief Request CR-20. *See Code Data Report on file for specific information. **Code Cases referenced on replacement CRD's obtained from Perry Nuclear Power Plant.

8. Test Conducted: Hydrostatic ☒ Pneumatic ☐ Nominal Operating Pressure ☐ Not Applicable ☐

Test Pressure 1005 psig Test Temperature > 135 / < 212 °F

9. Remarks: Satisfactory VT-2 examination was recorded during the D2R17 10 Yr Hydro under WO 99124181-09. ***Year Built on these CRD's was not recorded prior to placement into their shipping containers.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan. 10, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co./of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-23, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-23-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-27-2001

Sheet: 2 Of 6

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: General Electric (Name)
Same as Above (Address)

WO 99250494-05 thru 24 PLAN 2-01-033
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 (Control Rod Drive)

5. (a) Construction Code ASME Sect. III, 19 65 Edition, W/65 Addenda, Code Cases 1335-2, 1361, 1352
(b) Edition of Section XI used for Repair/Replacement 19 89* Edition, NO Addenda, Code Cases N207**, 1361-2**

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Control Rod Drive	General Electric	602C	*	Location: 42-35	1967	Replaced	Yes
Control Rod Drive	General Electric	895	*	Cat. ID-00413, UTC-2621571	1969	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 42-35	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C192	N/A	Cat. ID-42416, UTC-2600344	N/A	Replacement	No
Control Rod Drive	General Electric	1052	*	Location: 42-11	1969	Replaced	Yes
Control Rod Drive	General Electric	966	*	Cat. ID-00413, UTC-2621572	1969	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 42-11	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C192	N/A	Cat. ID-42416, UTC-2600344	N/A	Replacement	No
Control Rod Drive	General Electric	A6523	*	Location: 14-07	***	Replaced	Yes
Control Rod Drive	General Electric	608C	*	Cat. ID-00413, UTC-2621567	1967	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 14-07	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C192	N/A	Cat. ID-42416, UTC-2600344	N/A	Replacement	No
Control Rod Drive	General Electric	1080	*	Location: 26-15	1980	Replaced	Yes
Control Rod Drive	General Electric	588	*	Cat. ID-00413, UTC-2621568	1969	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 26-15	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C192	N/A	Cat. ID-42416, UTC-2600344	N/A	Replacement	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-27-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Sheet: 3 Of 6

3. Work Performed By: General Electric (Name)
Same as Above (Address)

Unit: 2
WO 99250494-05 thru 24 PLAN 2-01-033
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 (Control Rod Drive)

5. (a) Construction Code ASME Sect. III, 19 65 Edition, W/65 Addenda, Code Cases 1335-2, 1361, 1352
(b) Edition of Section XI used for Repair/Replacement 19 89* Edition, NO Addenda, Code Cases N207**, 1361-2**

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Control Rod Drive	General Electric	1085	*	Location: 50-35	1969	Replaced	Yes
Control Rod Drive	General Electric	**A8736	*	Cat. ID-00413, UTC-2621564	1989	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 50-35	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-2617049	N/A	Replacement	No
Control Rod Drive	General Electric	A4040	*	Location: 34-19	***	Replaced	Yes
Control Rod Drive	General Electric	**A8098	*	Cat. ID-00413, UTC-2621554	1986	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 34-19	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC- 2617049	N/A	Replacement	No
Control Rod Drive	General Electric	93	*	Location: 18-39	***	Replaced	Yes
Control Rod Drive	General Electric	188	*	Cat. ID-00413, UTC-2621570	1967	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 18-39	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-2617049	N/A	Replacement	No
Control Rod Drive	General Electric	A5115	*	Location: 22-55	***	Replaced	Yes
Control Rod Drive	General Electric	6101	*	Cat. ID-00413, UTC-2621558	1974	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 22-55	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-2617049	N/A	Replacement	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-27-2001

Sheet: 4 Of 6

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: General Electric (Name)
Same as Above (Address)

WO 99250494-05 thru 24 PLAN 2-01-033
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 (Control Rod Drive)

5. (a) Construction Code ASME Sect. III, 19 65 Edition, W/65 Addenda, Code Cases 1335-2, 1361, 1352
(b) Edition of Section XI used for Repair/Replacement 19 89* Edition, NO Addenda, Code Cases N207**, 1361-2**

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Control Rod Drive	General Electric	126	*	Location: 10-31	1968	Replaced	Yes
Control Rod Drive	General Electric	888A	*	Cat. ID-00413, UTC-2621566	1969	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 10-31	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-2617049	N/A	Replacement	No
Control Rod Drive	General Electric	A5444	*	Location: 30-23	***	Replaced	Yes
Control Rod Drive	General Electric	**A4054	*	Cat. ID-00413, UTC-2621573	1992	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 30-23	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-2617049	N/A	Replacement	No
Control Rod Drive	General Electric	A3700	*	Location: 18-51	***	Replaced	Yes
Control Rod Drive	General Electric	599	*	Cat. ID-00413, UTC-2621557	1968	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 18-51	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-2617049	N/A	Replacement	No
Control Rod Drive	General Electric	962	*	Location: 14-55	***	Replaced	Yes
Control Rod Drive	General Electric	8731	*	Cat. ID-00413, UTC-2621555	1989	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 14-55	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-2617049	N/A	Replacement	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-27-2001

Sheet: 5 Of 6

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: General Electric (Name)
Same as Above (Address)

WO 99250494-05 thru 24 PLAN 2-01-033
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 (Control Rod Drive)

5. (a) Construction Code ASME Sect. III, 19 65 Edition, W/65 Addenda, Code Cases 1335-2, 1361, 1352
(b) Edition of Section XI used for Repair/Replacement 19 89* Edition, NO Addenda, Code Cases N207**, 1361-2**

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Control Rod Drive	General Electric	613C	*	Location: 34-43	***	Replaced	Yes
Control Rod Drive	General Electric	1101	*	Cat. ID-00413, UTC-2621575	1969	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 34-43	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-2617049	N/A	Replacement	No
Control Rod Drive	General Electric	A3556	*	Location: 18-47	***	Replaced	Yes
Control Rod Drive	General Electric	700C	*	Cat. ID-00413, UTC-2621556	1967	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 18-47	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-2617049	N/A	Replacement	No
Control Rod Drive	General Electric	1932	*	Location: 46-43	***	Replaced	Yes
Control Rod Drive	General Electric	1541	*	Cat. ID-00413, UTC-2621565	1970	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 46-43	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC- 2617049	N/A	Replacement	No
Control Rod Drive	General Electric	A5513	*	Location: 42-55	***	Replaced	Yes
Control Rod Drive	General Electric	909	*	Cat. ID-00413, UTC-2621569	1969	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 42-55	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-2617049	N/A	Replacement	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-27-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 6 Of 6

Unit: 2

3. Work Performed By: General Electric (Name)
Same as Above (Address)

WO 99250494-05 thru 24 PLAN 2-01-033
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 (Control Rod Drive)

5. (a) Construction Code ASME Sect. III, 19 65 Edition, W/65 Addenda, Code Cases 1335-2, 1361, 1352
(b) Edition of Section XI used for Repair/Replacement 19 89* Edition, NO Addenda, Code Cases N207**, 1361-2**

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Control Rod Drive	General Electric	7020	*	Location: 38-27	1978	Replaced	Yes
Control Rod Drive	General Electric	**A9117	*	Cat. ID-00413, UTC-2621561	1992	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 38-27	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-4617049	N/A	Replacement	No
Control Rod Drive	General Electric	600C	*	Location: 26-47	***	Replaced	Yes
Control Rod Drive	General Electric	9173	*	Cat. ID-00413, UTC-2621574	1978	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 26-47	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-2617049	N/A	Replacement	No
Control Rod Drive	General Electric	509C	*	Location: 38-59	***	Replaced	Yes
Control Rod Drive	General Electric	272	*	Cat. ID-00413, UTC-2621560	1967	Replacement	Yes
Control Rod Drive Flange Cap Screws.	Unknown	Unknown	N/A	Location: 38-59	N/A	Replaced	No
Control Rod Drive Flange Cap Screws.	Unknown	Ht. Code-C475	N/A	Cat. ID-42416, UTC-2617049	N/A	Replacement	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 5-21-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

W/O 321079 (PLAN 2-01-036)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (CCSW to ECCS Room Coolers)

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
CCSW Supply to ECCS Room Coolers Check Valve	KEROTEST	Unknown	N/A	EPN 2-1599-131B	N/A	Repair	No

7. Description of work: Excavated flaw from upstream weld by grinding and performed weld repair.


8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 180 psig Test Temperature Ambient °F

9. Remarks: Surface examinations of excavation, root pass and final weld performed to ensure removal of flaw and to comply with Code Case N-416-1.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR** Conforms to Section XI of the ASME Code.

Signed:  ISI COORDINATOR 5/21, 20 01
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 5-21, 20 01 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-21-01 Inspector:  Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-5-2001

Sheet: 1 Of 2

Unit: 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

EC8074 W/O 99102788, 91 (PLAN 2-01-045)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1600 (Containment)

5. (a) Construction Code ASME Section III, 1965 Edition, Summer 1965 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 1998 Edition, NO Addenda, Code Cases NONE

*1998 Edition implemented in accordance with Relief Request MCR-02

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Containment Penetration X-124 Expansion Bellows	Pathway	Unknown	N/A	EPN 2-1600-X-124	Unknown	Replaced	Yes
Containment Penetration X-124 Expansion Bellows (Inner Ply)	Pathway	H73510-2-2	N/A	EPN 2-1600-X-124 Catalog ID 1220657	2001	Replacement	Yes
Containment Penetration X-124 Expansion Bellows (Outer Ply)	Pathway	H73510-2-3	N/A	EPN 2-1600-X-124 Catalog ID 1220659	2001	Replacement	Yes
X-124 Stand Off Rings	Ecker-Erhardt Co, Inc.	None	N/A	EPN 2-1600-X-124 Catalog ID 1226701	N/A	Replacement	No
Containment Penetration X-116B Expansion Bellows	Pathway	Unknown	N/A	EPN 2-1600-X-116B	Unknown	Replaced	Yes
Containment Penetration X-116B Expansion Bellows (Inner Ply)	Pathway	H73510-1-2	N/A	EPN 2-1600-X-116B Catalog ID 1220656	2001	Replacement	Yes
Containment Penetration X-116B Expansion Bellows (Outer Ply)	Pathway	H73510-1-3	N/A	EPN 2-1600-X-116B Catalog ID 1220658	2001	Replacement	Yes
X-116B Stand Off Rings	Ecker-Erhardt Co, Inc.	None	N/A	EPN 2-1600-X-116B Catalog ID 1226700, 1226699	N/A	Replacement	No

7. Description of work: Removed existing two ply bellows and replaced with inner and outer single ply bellows.

8. Test Conducted: Hydrostatic [] Pneumatic [X] Nominal Operating Pressure [] Not Applicable []

Test Pressure 49.5 psig Test Temperature Ambient °F

9. Remarks: Pneumatic test performed in accordance with 10 CFR 50.50 Appendix J and ASME Section XI IWE-5240

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-5-2001

Sheet: 2 Of 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

EC8074 W/O 99102788, 91 (PLAN 2-01-045)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1600 (Containment)

5. (a) Construction Code ASME Section III, 19 65 Edition, Summer 1965 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR 11-5, 2001
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 10-6, 2001 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 11-6-01 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-5-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

EC8074 W/O 99102788-04 (PLAN 2-01-046)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (LPCI)

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
X-116B Flued Head Anchor Support Members	Unknown	N/A	N/A	2-1600-X-116B	N/A	Replaced	No
X-116B Flued Head Anchor Support Members	Unknown	N/A	N/A	Cat ID 24992	N/A	Replacement	No
5" Seamless Pipe (A-106 Grade B)							
X-116B Flued Head Anchor Support Members	Unknown	N/A	N/A	Cat ID 673433 Heat B1Y6861	N/A	Replacement	No
8" x 8" x 3/8" Plate (A36)							

7. Description of work: Temporarily removed support members to facilitate bellows replacement.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: VT-3 baseline examination performed.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR 11-8, 20 01
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 11-8, 2001 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 11-8-01 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-2-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 1 Of 2

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

DCP 9900425 WO 99248274 PLAN 2-01-047
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 3000 (Main Steam)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89* Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Support M-564E-Sht.4 on line 2-3001A-20". (Task 03)	Pacific Scientific/NPS	S/N: 17630, P/N: PSA-10	N/A	Mechanical snubber w/ adaptor, Item #1 of original support assembly.	Unknown	Replaced	Yes
Support M-564E-Sht.4 on line 2-3001A-20". (Task 03)	Unknown	None	N/A	2" Sch.80 pipe used as extension pc., Item #7 of original support assembly.	Unknown	Replaced	No
Support M-564E-Sht.4 on line 2-3001A-20". (Task 03)	NPS	P/N: SMTT	N/A	Transition kit, Item #9 of original support assembly.	Unknown	Replaced	No
Support M-564E-Sht.4 on line 2-3001A-20". (Task 03)	Lisega	S/N: 612653/22, P/N: 307256	N/A	Cat ID-1218470, UTC-2605440.	Unknown	Replacement	No

7. Description of work: Modify Main Steam piping support (snubber assembly) to facilitate station EPU.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [] Not Applicable [X]

Test Pressure psig Test Temperature Ambient °F

9. Remarks: Installation task is 03. Satisfactory VT-3/4 performed on final installation. Snubber functional test performed under Task 04.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPAIR/REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR 1/7, 20 02
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPAIR/REPLACEMENT described in this report on 1-15, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-15-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

As Required by the Provisions of ASME Code Section XI

[illegible]

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-06-2001

Sheet: 1 Of 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: NWS Technologies (Name)
131 Venture Blvd. Spartanburg, SC 29301 (Address)

PO 23108-Rel.001 PLAN 2-01-048
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code **ASME Section III, 19 68 Edition, W/68 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

**** General Electric APED Specification 21A9206, rev.6.**

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Target Rock Relief Valve.	Target Rock Corp.	S/N: 130A	N/A	Body S/N: 222	Unknown	Repair	Yes
Target Rock Relief Valve.	Target Rock Corp.	S/N: 130A	N/A	Cat ID-008070, UTC-2062536, Installed into body S/N: 115.	Unknown	Repaired	Yes
Target Rock Relief Valve 2 nd stage disc.	Target Rock Corp.	Not recorded	N/A	P/N: 200243-1	Unknown	Replaced	No
Target Rock Relief Valve 2 nd stage disc.	Target Rock Corp.	S/N: 78	N/A	SI-570E28, QRI-D93-02504	Unknown	Replacement	No

7. Description of work: NWS to refurbish/test set pressure spare TRV. OEM parts used were items drawn/sent from Dresden Station Stores Dept.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [] Not Applicable [X]

Test Pressure psig Test Temperature °F

9. Remarks: Hardcopy documents contained in Receipt #63277.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: John H. Young [Signature] ISI COORDINATOR Jan. 22, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by Hartford Steam Boiler of CT having inspected the **REPAIR/REPLACEMENT** described in this report on 1-23-02, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-23-02 Inspector: Robert T. Rainey [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-06-2001

Sheet: 2 Of 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: NWS Technologies (Name)
131 Venture Blvd. Spartanburg, SC 29301 (Address)

PO 23108-Rel.001 PLAN 2-01-048
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code **ASME Section III, 19 68 Edition, W/68 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

** General Electric APED Specification 21A9206, rev.6.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Target Rock Relief Valve pilot disc.	Target Rock Corp.	Not recorded	N/A	P/N: 200242-1	Unknown	Replaced	No
Target Rock Relief Valve pilot disc.	Target Rock Corp.	S/N: 452	N/A	SI-570D56, QRI-292703	Unknown	Replacement	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-26-2001

Sheet: 1 Of 1

Unit: 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

DCP 9900842 WO 99248723 PLAN 2-01-049
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1400 (Core Spray) and 1500 (LPCI)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89* Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Support M-3214-23 on line 2-1508-18". (Task 03)	Unknown	None	N/A	Plate steel, Item #5 of original support assembly.	Unknown	Replaced	No
Support M-3214-23 on line 2-1508-18". (Task 03)	Unknown	None	N/A	Cat ID-38329, UTC-2401080.	Unknown	Replacement	No
Support M-3214-23 on line 2-1508-18". (Task 03)	Hilti	None	N/A	Anchor bolts, Item #6 of original support assembly, (qty-4).	Unknown	Replaced	No
Support M-3214-23 on line 2-1508-18". (Task 03)	Drillco Devices, LTD	P/N: MB-750-16-1/2-11-1/2	N/A	Cat ID-35650, UTC-2618398.	Unknown	Replacement	No

7. Description of work: Modify four piping supports to facilitate station EPU.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [] Not Applicable [X]

Test Pressure psig Test Temperature Ambient °F

9. Remarks: WO tasks affected are 03 thru 06. Satisfactory VT-3/4 performed on final installations.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR 1/7, 20 02
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-16, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-16-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-26-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 2 Of 1

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

DCP 9900842 WO 99248723 PLAN 2-01-049
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1400 (Core Spray) and 1500 (LPCI)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89* Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Support M-3381 on line 2-1527-3". (Task 04)	Unknown	None	N/A	U-bolt, Item #5 of original support assembly.	Unknown	Replaced	No
Support M-3381 on line 2-1527-3". (Task 04)	Unknown	None	N/A	Nuts for U-bolt, Item #6 of original support assembly.	Unknown	Replaced	No
Support M-3381 on line 2-1527-3". (Task 04)	ITT Grinnel	P/N: 137SN	N/A	Cat ID-1218452, UTC-2607161. (U-bolt w/nuts.)	Unknown	Replacement	No
Support M-3209-24 on line 2-1404-12". (Task 05)	Unknown	None	N/A	U-bolt w/nuts, Item #8 of the original support assembly (qty-2).	Unknown	Replaced	No
Support M-3209-24 on line 2-1404-12". (Task 05)	Consolidated Power Supply	P/N: 137SN	N/A	Cat ID-1218446, UTC-2607162. (U-bolt w/nuts.)	Unknown	Replacement	No
Support M-3209-24 on line 2-1404-12". (Task 05)	Unknown	None	N/A	Support members consisting of Item's 9 and 10 from dwg. BOM.	Unknown	Replaced	No
Support M-3209-24 on line 2-1404-12". (Task 05)	Unknown	None	N/A	Cat ID-641125, UTC-2606198.	Unknown	Replacement	No
Support M-3209-24 on line 2-1404-12". (Task 05)	Consolidated Power Supply	P/N: 180471, Ht. #U1192	N/A	Cat ID-514702, UTC-2606197	Unknown	Replacement	No
Support M-3209-24 on line 2-1406-8". (Task 05)	Elcen	P/N: Fig.420, Type 1, Size 5	N/A	Sway strut, originally as Item #1 part of support assembly.	Unknown	Replaced	No
Support M-3209-24 on line 2-1406-8". (Task 05)	Lisega	P/N: 305253RE3, S/N: 01615080/075	N/A	Cat ID-1218469, UTC-2622171.	Unknown	Replacement	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-5-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 1 Of 1

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

EC 330565 WO 99235806 PLAN 2-01-053
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1600 (Containment)

5. (a) Construction Code ASME Section III, 19 65 Edition, Summer 1965 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 98* Edition, NO Addenda, Code Cases NONE

*1998 Edition implemented in accordance with Relief Request MCR-02

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Electrical Containment Penetration X-202F	General Electric	Unknown	N/A	EPN: X-202F	N/A	Replaced	No
Electrical Containment Penetration X-202F	Conax Nuclear, Inc.	6160	6160	Cat ID-1220565, UTC-2621983,	2001	Replacement	Yes

7. Description of work: Removed existing electrical containment penetration head from the sleeve and welded new head on the existing sleeve.

8. Test Conducted: Hydrostatic [] Pneumatic [X] Nominal Operating Pressure [] Not Applicable []

Test Pressure 49.5 psig Test Temperature Ambient °F

9. Remarks: Pneumatic test performed in accordance with 10 CFR 50 Appendix J and ASME Section XI IWE-5240

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan. 10, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-18, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-26-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 99222115-01 PLAN 2-01-060
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0200 (Recirc.)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89* Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Support M-1135-Sht.15, EPN 2-0202-03-B.	Pacific Scientific	S/N: 6811, P/N: PSA-35	N/A	Mechanical snubber.	Unknown	Replaced	Yes
Support M-1135-Sht.15, EPN 2-0202-03-B.	Pacific Scientific	S/N: 5425, P/N: PSA-35	N/A	Cat ID-40805, UTC-2622819, BWD WR A32811.	1980	Replacement	Yes

7. Description of work: Remove/test/replace (if necessary)/reinstall PSA-35 mechanical snubber.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [] Not Applicable [X]

Test Pressure psig Test Temperature °F

9. Remarks: Satisfactory VT-3/4 performed on final installation and Snubber functional test per DTS 0020-02. Removed snubber had degraded performance

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan. 08, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-14-02 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-14-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-27-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 99222112-01 PLAN 2-01-061
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 3000 (Main Steam)

5. (a) Construction Code USAS B31.1-0, 1967 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 1989* Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Support M-564-Sht.1, EPN 2-3001-C-50.	Pacific Scientific	S/N: 8229, P/N: PSA-35	N/A	Mechanical snubber.	Unknown	Replaced	Yes
Support M-564-Sht.1, EPN 2-3001-C-50.	Pacific Scientific	S/N: 11817, P/N: PSA-35	N/A	Cat ID-40805, UTC-2622817, BWD WR A32405.	1983	Replacement	Yes

7. Description of work: Remove/test/replace (if necessary)/reinstall PSA-35 mechanical snubber.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [] Not Applicable [X]

Test Pressure psig Test Temperature °F

9. Remarks: Satisfactory VT-3/4 performed on final installation and Snubber functional test per DTS 0020-02. Removed snubber had degraded performance

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan. 08, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-14, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-14-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-27-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 0372768-01 PLAN 2-01-062
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 3000 (Main Steam)

5. (a) Construction Code USAS B31.1-0, 1967 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 1989* Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Support M-1135-Sht.13, EPN 2-0202-12-A.	Pacific Scientific	S/N: 6322, P/N: PSA-35	N/A	Mechanical snubber.	Unknown	Replaced	Yes
Support M-1135-Sht.13, EPN 2-0202-12-A.	Pacific Scientific	S/N: 11036, P/N: PSA-35	N/A	Cat ID-40805, UTC-2622820, BWD WR A32388.	1983	Replacement	Yes

7. Description of work: Test and replace PSA-35 mechanical snubber that was discovered was foreign material covering it, ref. AP/CR 079637.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Not Applicable ☒

Test Pressure psig Test Temperature °F

9. Remarks: Satisfactory VT-3/4 performed on final installation and Snubber functional test per DTS 0020-02.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan. 07, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co./of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-14, 2001 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-14-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 10-27-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Sheet: 1 Of 1

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 0374695-01 PLAN 2-01-063
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 3900 (CCSW)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Pipe line 2/3-39251-3"-DX.	Unknown	None	N/A	P & ID M-3121, associated with 2/3-9400-102 Refrigeration Condensing Unit.	N/A	Replaced	No
Pipe line 2/3-39251-3"-DX, sch.80, A-106 Gr.B	Unknown	HT#-Y67155	N/A	Cat. ID-385642, UTC-2048190.	N/A	Replacement	No

7. Description of work: Replaced a section of vertical pipe that developed a leak.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []
Test Pressure 178 psig Test Temperature 71.4 °F

9. Remarks: VT-2 was performed, and no leakage was detected.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR 1/7, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-14, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-14-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-4-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 1 Of 1

Unit: 2

3. Work Performed By: General Electric (Name)
Same as Above (Address)

WO 99117771-01 PLAN 2-01-064
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0200 Reactor Head Vent

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Studs (7/8"-9 A/SA-193 Gr. B7) for flange set HV2-12-FLG. on pipe line 2-0215-2"-B.	Unknown	N/A	N/A	HV2-12-FLG, Isometric dwg. ISI-126.	N/A	Replaced	No
Studs (7/8"-9 A/SA-193 Gr. B7 cut all-thread bar stock) for flange set HV2-12-FLG. on pipe line 2-0215-2"-B.	NOVA MACHINE, INC.	Ht. Cd. - NWN	N/A	Cat ID - 037096, UTC - 2045537, Issue Tkt - 5453000	N/A	Replacement	No

7. Description of work: Replace the flange studs that were lost during D2R17.

8. Test Conducted: Hydrostatic ☒ Pneumatic ☐ Nominal Operating Pressure ☐ Not Applicable ☐

Test Pressure 1005 psig Test Temperature > 135 / < 212 °F

9. Remarks: Satisfactory VT-2 examination was recorded during the D2R17 10 Yr Hydro under WO 99124181-09. ***Year Built on these CRD's was not recorded prior to placement into their shipping containers..

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: John H. Young [Signature] ISI COORDINATOR Jan. 16, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by Hartford Steam Boiler of CT having inspected the **REPAIR/REPLACEMENT** described in this report on 1-18, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-18-02 Inspector: Robert T. Rainey [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-5-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

DCP 9900485 W/O 99225126 (PLAN 2-01-065)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 4700 (Instrument Air)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 98* Edition, NO Addenda, Code Cases NONE

*1998 Edition implemented in accordance with Relief Request MCR-02

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Cap on line 2-47320-1" through Containment Penetration X-139D	Unknown	None	N/A	Cat ID 7442	Unknown	Replacement	No

7. Description of work: Removed Instrument Air line, cut and capped line at containment penetration.

8. Test Conducted: Hydrostatic [] Pneumatic [X] Nominal Operating Pressure [] Not Applicable []

Test Pressure 49.5 psig Test Temperature Ambient °F

9. Remarks: Pneumatic test performed in accordance with 10 CFR 50 Appendix J and ASME Section XI IWE-5240.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR 11-5, 20 01
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 11-5, 2001 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 11-5-01 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 11-5-2001

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 00381568-01 PLAN 2-01-066
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (LPCI)

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Pipe line 2-1510-8"-D weld immediately downstream of pump 2-1501-44B.	Unknown	N/A	N/A	P & ID: M-29-Sht. 2	N/A	Repaired	No
Pipe line 2-1510-8"-D weld immediately downstream of pump 2-1501-44B.	Unknown	N/A	N/A	P & ID: M-29-Sht. 2	N/A	Repair	No

7. Description of work: Weld repair through wall leakage on 8" elbow to flange weldment.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 190 psig Test Temperature Ambient °F

9. Remarks: Acceptable VT-2 baseline examination after repairs.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR 1/7, 2001
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-15, 2001 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-15-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 01-21-2002

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 1 Of 1

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 0400429-01 PLAN 2-02-001
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 (LPCI) and 5700 (HVAC)

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Unit 2 CCSW Vault Room Cooler.	Aerofin	Model: C-12AE-20.6X70.5-10-1	N/A	EPN: 2-5700-30A	N/A	Replaced	No
Unit 2 CCSW Vault Room Cooler, isolated with blind flanges, qty-2 (see work description below).	A & A EQUIPMENT & SUPPLY CO.	HT# 988WNT	N/A	Cat. Id-005013, UTC- 2032914. Pipe lines: 2-15100B-2" and 2-15101B-2".	N/A	Replacement	No

7. Description of work: Under Temporary Modification 334930 remove the cooler and place blind flanges on associated pipe flanges until new replacement cooler is installed under WO 0400171.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 194.2 psig Test Temperature Ambient °F

9. Remarks: Acceptable VT-2 was performed.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: John H. Young ISI COORDINATOR Jan.28, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by Hartford Steam Boiler of CT having inspected the **REPAIR/REPLACEMENT** described in this report on 1-29, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-29-02 Inspector: Robert T. Rainey IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 08-03-2001

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 1 Of 1

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WO 972047223 PLAN 2-99-002
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code USAS B31.1-0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Spare Main Steam Isolation Valve Disc removed from 3-0203-2D.	Crane	S/N: 25, Ht. #5-21207	N/A	None	Unknown	Repair	No
Spare Main Steam Isolation Valve Disc removed from 3-0203-2D.	Crane	S/N: 25, Ht. #5-21207	N/A	None	Unknown	Repaired	No
Pilot seat welded to MSIV disc removed from 3-0203-2D.	Crane	None	N/A	None		Replaced	No
Pilot seat welded into spare MSIV disc.	Crane	P/N: 1552	N/A	Cat ID-07995, UTC-2059412.		Replacement	No

7. Description of work: Weld repair Stellite surfaces such as seating and guides along with replacing pilot seat welded to spare disc.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [] Not Applicable [X]

Test Pressure psig Test Temperature °F

9. Remarks: NDE (PT surface) performed on disc excavations and final machined/welds.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR Jan.08, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 1-14, 20 02 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-14-02 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: NWS Technologies (Name)
131 Venture Blvd. Spartanburg, SC 29301 (Address)

Date: 09-10-1999

Sheet: 1 Of 1

Unit: 2

PO 367668-Rel.DR2 PLAN 2-99-037
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code **ASME Section III, 19 68 Edition, W/68 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

** General Electric APED Specification 21A9206, rev.6.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Target Rock Relief Valve.	Target Rock Corp.	S/N: 214	N/A	Body S/N: 223	Unknown	Repair	Yes
Target Rock Relief Valve.	Target Rock Corp.	S/N: 214	N/A	Cat ID-008070, UTC-2042359	Unknown	Repaired	Yes

7. Description of work: NWS to refurbish/test set pressure spare TRV. Repair is actually seat(s) weld removal to facilitate seat replacement.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [] Not Applicable [X]

Test Pressure ___ psig Test Temperature ___ °F

9. Remarks: Satisfactory VT-2 examination was recorded during the D3R16 during the Rx pressure test.. Reference Repair/Replacement Plan 3-99-043 that recorded Replacement valve system S/N 214 into the system under WO 99046067-01.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: John H. Young ISI COORDINATOR Jan.22, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by Hartford Steam Boiler of CT having inspected the **REPAIR/REPLACEMENT** described in this report on 1-23, 2002 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-23-02 Inspector: Robert T. Rainey Robert T. Rainey Commissions: IL932, NB7742NIBS
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 08-28-2000

Sheet: 1 Of 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL, 60450 (Address)

Unit: 2

3. Work Performed By: NWS Technologies (Name)
131 Venture Blvd. Spartanburg, SC 29301 (Address)

PO 000239-Rel.0003 PLAN 2-99-038
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code **ASME Section III, 19 68 Edition, W/68 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

** General Electric APED Specification 21A9206, rev.6.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Target Rock Relief Valve.	Target Rock Corp.	S/N: 223	N/A	Body S/N: 214	Unknown	Repair	Yes
Target Rock Relief Valve.	Target Rock Corp.	S/N: 223	N/A	Cat ID-008070, UTC-2062534	Unknown	Repaired	Yes
Target Rock Relief Valve 2 nd stage disc.	Target Rock Corp.	Not recorded	N/A	P/N: 200243-1	Unknown	Replaced	No
Target Rock Relief Valve 2 nd stage disc.	Target Rock Corp.	S/N: 872	N/A	Cat ID-08081, UTC-2073446	Unknown	Replacement	No

7. Description of work: NWS to refurbish/test set pressure spare TRV. OEM parts used were items drawn/sent from Dresden Station Stores Dept.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Not Applicable ☒

Test Pressure psig Test Temperature °F

9. Remarks: Satisfactory VT-2 examination was recorded during the D2R17 10 Yr Hydro under WO 99124181-09. Reference Repair/Replacement Plan 2-01-027 that recorded Replacement valve system S/N 223 into the system under WO 99120951-01.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: John H. Young [Signature] ISI COORDINATOR Jan.22, 2002
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by Hartford Steam Boiler of CT having inspected the **REPAIR/REPLACEMENT** described in this report on 1-23, 20 02 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-23-02 Inspector: Robert T. Rainey [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)

Date: 08-28-2000

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Sheet: 2 Of 2

Unit: 2

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

PO 000239-Rel.0003 PLAN 2-99-038
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 (Main Steam)

5. (a) Construction Code **ASME Section III, 19 68 Edition, W/68 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

** General Electric APED Specification 21A9206, rev.6.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
Target Rock Relief Valve pilot disc.	Target Rock Corp.	Not recorded	N/A	P/N: 200242-1	Unknown	Replaced	No
Target Rock Relief Valve pilot disc.	Target Rock Corp.	S/N: 601	N/A	Cat ID-08086, UTC-2049404	Unknown	Replacement	No

CATEGORY 3

FORM NIS-2 (OWNER'S REPORT OF REPAIR OR REPLACEMENT As Required by the Provisions of ASME Code Section XI)

DAP 11-18
REVISION 09

1. Owner United Company (Name)
One First National Plaza, Chicago IL, 60601 (Address)

Date 11-10-99

2. Plant Dresden Nuclear Power Station (Name)
6501 North Dresden Road, Morris IL, 60450 (Address)

Sheet 1 of 1

Unit 1

3. Work Performed By Same as Above (Name)

W.R. 990001156-01 (PLAN 2-99-021)
Report Organization P.O. No. Job No. etc.

Same as Above (Address)

4. Identification of System 0203 Main Steam

5.(a) Construction Code ASME Section III 19 65 Edition, NO Addenda Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda Code Cases NONE

6. Identification of Components Required or Replaced and Replacement Components

Name of Components	Name of Manufacturer	Mfrs. Serial No.	Tag Brd No.	Other ID	Yr Bld	Repair, Replaced or Replacement	Code Stamped Yes/No
Electronic Relief Valve (Serial Number BXN1295) Inlet Flange Bolting	Consolidated Dwyer	BXN1295 (Valve)	N/A	None	N/A	Replaced	No
Electronic Relief Valve Inlet Flange Stud Hex Nuts (6 Total)	Consolidated Dwyer	Hex Code QT74	N/A	SI 8700478	N/A	Replacement	No

7. Description of work During valve rebuild, MMID discovered inlet flange hex nuts were damaged and required replacement. Nuts were later provided in Unit 2 order W970099982-01 (Repair/Replacement Plan 2-99-013).

8. Test Conducted Hydraulics | | Pressures | | Normal Operating Pressure | | Not Applicable | X |

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks None

Certificate of Compliance

We certify that the statements made in this report are correct and that REPLACEMENT conforms to Section XI of the ASME Code

Signed: Bryan J. Casey (Owner or Qualified Designer) 11-17-99 (Date)

ES/COORDINATOR (Title)

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88-1/460

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspection and the State of Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 11-21-99, and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 11-21-99 Inspector: Robert J. Allen Commission: 0022-9874288
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT FOR REPAIR OR REPLACEMENTS
As Required by the Provisions of ASME Code Section XI

1. Owner: Exelon Nuclear (Name)
P.O. Box 805379, Chicago IL, 60680-5379 (Address)
2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)
3. Work Performed By: Same as Above (Name)
Same as Above (Address)
4. Identification of System: 3900 Service Water (Diesel Generator Cooling Water)
5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE
6. Identification of Components Repaired or Replaced and Replacement Components

Date: 1-24-2001
Sheet: 1 Of 1
Unit: 2

WR 990053038-01 (PLAN 2-99-054)
Repair Organization P.O. No., Job No. etc.

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Year Built	Repair, Replaced or Replacement	Code Stamped Yes/No
8" X 6" X 6" Concentric Expansion Joint	Unknown	Unknown	N/A	EPN 2-3903	N/A	Replaced	No
5/8" Bolting (Bolts, Nuts and Washers)	Unknown	Unknown	N/A	EPN 2-3903	N/A	Replaced	No
3/4" Bolting (Bolts, Nuts and Washers)	Unknown	Unknown	N/A	EPN 2-3903	N/A	Replaced	No
8" X 6" X 6" Concentric Expansion Joint	Proco Products, Inc.	Unknown	N/A	Cat ID 37883	N/A	Replacement	No
5/8" Bolting (Bolts, Nuts and Washers)	Unknown	Unknown	N/A	Cat ID 3192, 37029, and 5227	N/A	Replacement	No
3/4" Bolting (Bolts, Nuts and Washers)	Unknown	Unknown	N/A	Cat ID 39295, 37031, and 30154.	N/A	Replacement	No

7. Description of work: Replaced existing expansion joint on suction piping of Unit 2 Diesel Generator Cooling Water pump with new expansion joint as preventative maintenance. Flange bolting replaced rather than cleaning and reinstalling existing material.

8. Test Conducted: Hydrostatic [] Pneumatic [] Nominal Operating Pressure [X] Not Applicable []

Test Pressure 3.5 psig Test Temperature Ambient °F

9. Remarks: VT-2 examination performed at nominal operating pressure, no evidence of leakage noted.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** conforms to Section XI of the ASME Code.

Signed: [Signature] ISI COORDINATOR 1/24, 2001
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 1-25, 2001 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-25-01 Inspector: [Signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)