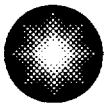
P.O. Box 63
Lycoming, New York 13093



Constellation
Nuclear

Nine Mile Point
Nuclear Station

A Member of the
Constellation Energy Group

January 11, 2002
NMP1L 1637

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

RE: Nine Mile Point Unit 1
Docket No. 50-220
DPR-63

Subject: ***Secondary Containment Leakage Testing Summary Report***

Gentlemen:

Nine Mile Point Nuclear Station, LLC (NMPNS) is submitting the attached "Secondary Containment Leakage Testing Summary Report" for Nine Mile Point Unit 1 (NMP1). This report covers testing performed in April 2001 during refueling outage number 16 to satisfy the requirements of Section 4.4.1, "Leakage Rate," of the NMP1 Technical Specifications (TS). As indicated in the report, the leakage rate criteria of TS 4.4.1 were met.

TS 6.9.3.f required this report to be submitted within 3 months after completion of testing, i.e. by July 11, 2001. However, due to the lack of specific procedural guidance, the need for this report was not properly identified. NMPNS has entered this occurrence into its corrective action program.

Very truly yours,

LA Hopkins

Lawrence A. Hopkins
Unit 1 Plant General Manager

LAH/IAA/cld

cc: Mr. H. J. Miller, NRC Regional Administrator, Region I
Mr. G. K. Hunegs, NRC Senior Resident Inspector
Records Management

A017

ATTACHMENT

SECONDARY CONTAINMENT LEAKAGE TESTING SUMMARY REPORT

APRIL 2001

1.0 PURPOSE

To assure the capability of the secondary containment to maintain leakage within allowable limits per Technical Specification (TS) 4.4.1, Leakage Rate.

2.0 PROCEDURE DESCRIPTION

These tests were performed using Revision 14 of Surveillance Test Procedure N1-ST-C5, "Secondary Containment and Reactor Building Emergency Ventilation System Operability Test." This procedure is on file at the facility.

3.0 TEST DATA

	April 9, 2001 #11 System	April 11, 2001 #12 System
Test Conditions Wind Speed	8.5 mph*	3.5 mph*
Acceptance Criteria Minimum Required Reactor Building Internal Differential Pressure per Section 4.4.1 and Figure 3.4.1 of the TS	-0.275 inches of water	-0.26 inches of water
Reactor Building Leakage Rate	1600 cfm** or less	1600 cfm** or less
Test Results Reactor Building Internal Differential Pressure	-0.30 inches of water	-0.31 inches of water
Ventilation Flow	1550 cfm**	1575 cfm**

4.0 EVALUATION

These test results demonstrated that the reactor building (secondary containment) leakage rate met the acceptance criteria of Section 4.4.1 and Figure 3.4.1 of the TS.

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* miles per hour
** cubic feet per minute