



Entergy Nuclear Generation Co.
Pilgrim Station
600 Rocky Hill Road
Plymouth, MA 02360

William J. Riggs
Director, Nuclear Assessment

January 15, 2002

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

SUBJECT: Entergy Nuclear Generation Company
Pilgrim Nuclear Power Station
Docket No. 50-293

DECEMBER 2001 Monthly Operating Report

LETTER NUMBER: 2.02.003

Dear Sir or Madam:

In accordance with Pilgrim Nuclear Power Station Technical Specification 5.6.4, the operational status summary for Pilgrim Nuclear Power Station is provided in the attachment for your information and planning. Should you have questions or comments concerning this report, please contact Jeff Rogers at (508) 830-8110.

Sincerely,

A handwritten signature in cursive script, appearing to read "W. J. Riggs".

W. J. Riggs

JLR/

Attachment: DECEMBER 2001 Monthly Operating Report

cc: Mr. Hubert Miller
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

IE24

Attachment

OPERATING DATA REPORT

DOCKET NO. 50-293
 NAME: Pilgrim
 COMPLETED BY: J. L. Rogers
 TELEPHONE: (508) 830-8110
 REPORT MONTH: DECEMBER 2001

OPERATING STATUS**NOTES**

- | | |
|--|---------------|
| 1. Unit Name | PILGRIM 1 |
| 2. Reporting Period | DECEMBER 2001 |
| 3. Licensed Thermal Power (MWt) | 1998 |
| 4. Nameplate Rating (Gross MWe) | 678 |
| 5. Design Electrical Rating (Net MWe) | 655 |
| 6. Maximum Dependable Capacity (Gross MWe) | 680 |
| 7. Maximum Dependable Capacity (Net MWe) | 653 |
| 8. If Changes Occur in Capacity Ratings (Items 3 through 7) Since Last Report, Give Reasons: | No changes. |
-
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A
-

| | <u>THIS MONTH</u> | <u>YR-TO-DATE</u> | <u>CUMULATIVE</u> |
|--|-------------------|-------------------|-------------------|
| 11. Hours in Reporting Period | 744.0 | 8,760.0 | 254,760.0 |
| 12. Hours Reactor Critical | 675.1 | 7,972.2 | 174,475.8 |
| 13. Hours Reactor Reserve Shutdown | 0.0 | 0.0 | 0.0 |
| 14. Hours Generator On-Line | 662.3 | 7,886.0 | 169,255.8 |
| 15. Hours Unit Reserve Shutdown | 0.0 | 0.0 | 0.0 |
| 16. Gross Thermal Energy Generated (MWH) | 1,280,220.0 | 15,307,647.1 | 308,090,228.4 |
| 17. Gross Electrical Energy Generated (MWH) | 448,060.0 | 5,347,530.0 | 104,936,494.0 |
| 18. Net Electrical Energy Generated (MWH) | 431,382.9 | 5,144,037.2 | 100,906,427.0 |
| 19. Unit Service Factor | 89.0 | 90.0 | 66.4 |
| 20. Unit Availability Factor | 89.0 | 90.0 | 66.4 |
| 21. Unit Capacity Factor (Using MDC Net) | 88.8 | 89.9 | 60.7 |
| 22. Unit Capacity Factor (Using DER Net) | 88.5 | 89.7 | 60.5 |
| 23. Unit Forced Outage Rate | 11.0 | 2.4 | 9.9 |
| 24. Shutdowns, Scheduled Outages Over Next 6 Months (Type, Date, Duration) | None. | | |

25. If Shutdown at End of Report Period,
 Estimated Date of Start-Up: Unit Operating

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OPERATIONAL SUMMARY

The plant entered the reporting period at 100 percent core thermal power (CTP) until December 14 at 2030 hours when a power reduction to approximately 55 percent CTP was commenced to perform a condenser thermal backwash. Reactor power was restored to 100 percent CTP on December 16 at 0637 hours. A power reduction was commenced on December 17 at 1147 hours to perform a control rod pattern adjustment. Reactor power was restored to 100 percent CTP at 2010 hours where it was maintained until an automatic reactor scram occurred on December 27 at 1321 hours. The plant automatically shut down when an unexpected trip of both reactor recirculation pumps occurred during a routine surveillance test of the Anticipated Transient Without Scram (ATWS) circuitry. The trip was apparently due to a mechanical failure of a test switch within the ATWS actuation logic. The reactor was made critical on December 30 at 1014 hours, and was synchronized to the grid on December 30 at 2303 hours. At the end of the reporting period, the reactor was at approximately 95 percent CTP.

UNIT SHUTDOWNS

| NO. | DATE | TYPE 1 | DURATION (HOURS) | REASON 2 | METHOD OF SHUTTING DOWN REACTOR 3 | CAUSE/CORRECTIVE ACTION/COMMENTS |
|-----|----------|-----------|---------------------|-------------|--|--|
| 3 | 12/27/01 | F | 81.7 | A | 3 | Automatic scram caused by an unexpected trip of both reactor recirculation pumps during a routine, scheduled surveillance test. LER 2001-007-00 will contain additional information. |

1

F - Forced
S - Scheduled

2

A - Equip Failure
B - Main or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training &
License Examination
F - Admin
G - Operation Error
H - Other

3

1 - Manual
2 - Manual Scram
3 - Auto Scram
4 - Continuation
5 - Other